

NRC Workshop: July 18, 2024

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Agenda

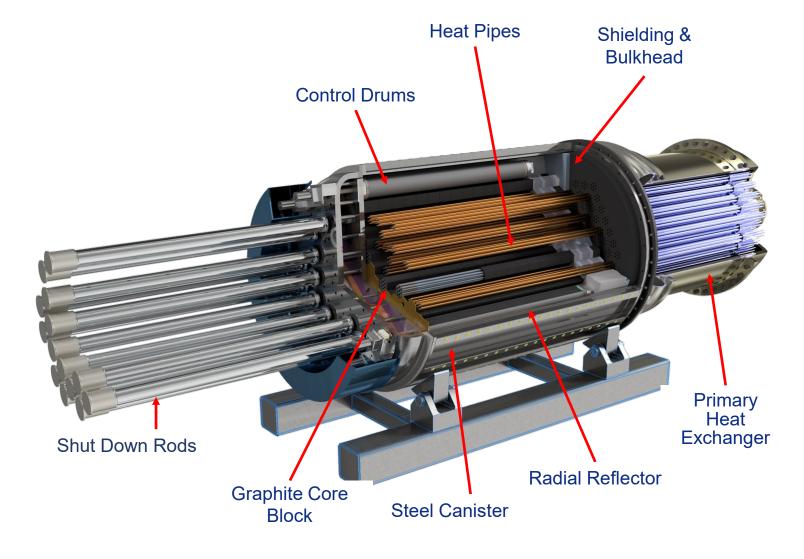
- eVinci Microreactor Design Overview
- Nuclear Test Reactor (NTR) Plan Overview and Interface with INL/DOME
- NTR PRA model development to date
- Licensing Approach and Challenges with NEI 18-04
 - (-) Cumulative risk targets not well defined
 - (-) Reliability data not readily available for advanced reactor components
 - (-) Division of responsibility between PRA and thermal hydraulic analysis
 - (+) Importance of interviews with other functional areas (SY, POS, RI, ESQ, SC)
 - (+) Databases for model configuration control



The eVinci Microreactor Design Overview

Safety through passive heat pipe technology, enabling a very low-pressure reactor

Parameter	eVinci
Power	15 MWt
Fuel Cycle	8 years
Fuel (Enrichment)	TRISO (19.75%)
Coolant	Heat Pipes
Reactor Pressure	~1 atm
Moderator	Graphite
Power Conversion	Open-Air Brayton
Efficiency	34%
Decay Heat Removal	Radial Conduction





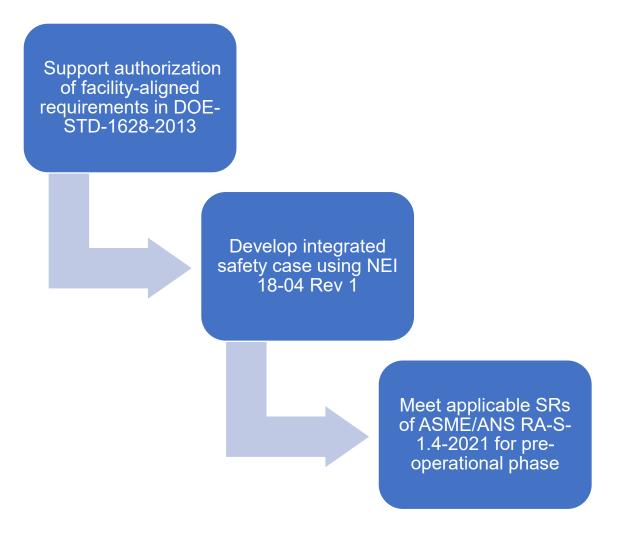
eVinci Microreactor Design Overview

Key Design Features for PRA

- Reactivity control provided by banks of control drums with neutron absorbing material
 - Shutdown rod assemblies provide defense-in-depth shutdown capability
- Heat flows from core through heat pipes to primary heat exchanger (PHX) and then through power conversion system (PCS) to environment
 - Passive heat removal system (PHS) and normal residual heat removal system (RNS) provide decay heat removal in accident sequences
- Functional containment provided by TRISO fuel layers, graphite fuel compact, and canister pressure boundary
- No operator interaction required in accident scenarios
 - Operator intervention possible as defense-in-depth function
 - PRA not crediting any human actions
- NTR design changes secondary side cooling and adds DOME as an additional layer of functional containment



NTR Plan Overview and Interface with INL/DOE





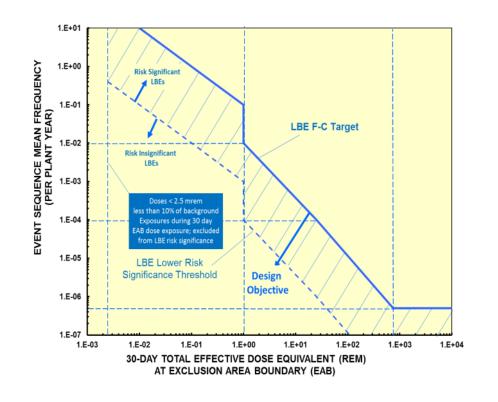
NTR PRA Model Development to Date

- As design continues to develop, many key PRA inputs are also in development:
 - Technical specifications
 - Operating procedures
 - System design documents or specifications
 - Operating experience for determining initiating event frequencies and component reliability data via frequentist approach
- Completed preliminary (tabletop) PRA
- Nearing completion of full-power, internal events quantification
- Starting simplified assessments of other hazards and operating modes



Cumulative Risk Targets (-)

- Guidance on how to evaluate against cumulative risk targets for integrated risk of entire facility would be helpful:
 - Total frequency of exceeding 100 mrem dose from all LBEs ≤ 1/yr
 - Early fatality risk ≤ 5.0E-07/yr
 - Latent cancer risk ≤ 2.0E-06/yr
- Looking to share best practices relating to:
 - Deriving risk importance measures relative to cumulative criteria
 - How and when to best characterize uncertainties
 - Summation method?
 - Visualization of meeting cumulative criteria
 - Event sequence family definition and grouping





Reliability Data for Advanced Reactor Components (-)

- No LWR experience base to draw on for data for some components specific to advanced reactor designs that are modeled in PRA
- Currently evaluating existing data sources and adjustment mechanisms
- When failure data is available, the applicability of the data needs to be assessed:
 - Is applicability data scale invariant? eVinci microreactor components much smaller than LWR
 - Is data applicable when using helium instead of water? A valve leakage rate, for example, may be significantly different when passing a gas through a valve vs a liquid.



Success criteria definition in FATE (-)

- FATE (Facility Flow, Aerosol, Thermal, and Explosion) is a new code for this application
 - Models heat and mass transfer and radionuclide release
- Challenges understanding how success criteria should be stated
 - Light water reactor (LWR) analog of core damage as top event no longer applicable
 - Assessing consequences from non-core sources, e.g., sodium from heat pipes
 - FATE model considers TRISO failure due to peak temperature as well as time-at-temperature
- First experience with several NLWR technical elements (MS, RC, RI)
 - Mechanistic source term and consequences developed by thermal hydraulic team
 - Communicating and understanding PRA concepts with analysis team and vice versa



Interviews with other functional areas (+)

- Required by ASME/ANS RA-S-1.4-2021 for systems analysis (SY-A6) and plant operating states definition (POS-A8)
 - Additional requirements in HR and hazard technical elements
- System engineer interviews crucial for understanding system function (as expected) but even more so due to in-progress nature of design
 - Example: understanding signal processing for primary I&C system
- Operations interviews for understanding operating modes
 - Names of operating modes match PWR experience (e.g., hot standby) but not always intuitive with this design – only nuclear heat-up
 - Mode transitions based on sodium phase change temperatures
 - Ops input critical for judging system (in)operability, needed for defining POS characterizations



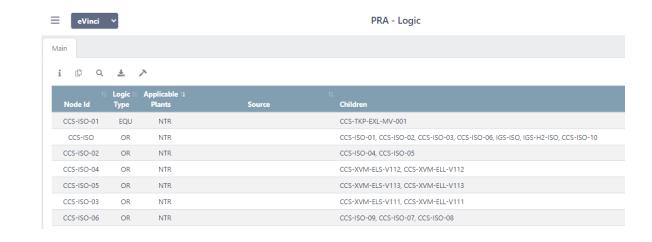
Interviews with other functional areas [cont.] (+)

- No explicit requirement for interviews in SC technical element
 - Interface with thermal fluid analyses is critical
 - Especially for meeting, e.g., SC-B3
- Example of unexpected result for NTR
 - Expected event tree path with failure of all top events to be most limiting dose consequence
 - Some cases where normal residual heat removal system (RNS) success results in worse consequences
 - Also helps inform our eVinci microreactor design
- Must understand and document what the thermal analysis code models for event tree success, failure, and bypass branches



Model Configuration Control Database (+)

- Optimal time to set up database to link model elements
 - Links assumptions to affected system models, type codes, event sequences, etc.
 - Pulls data from other technical areas such as cable mapping database
- PRA model logic added directly from logic model database file when system model is finalized
- Automates fault tree linking process
- Anticipate many design changes
 - Documentation of PRA impact statements
 - Plan implementation of design changes that impact PRA model and link to affected elements
 - Space for documenting resolution or justification for deferral
 - Allows for the iterative process of NEI 18-04 to be implemented as intended





Model configuration control database [cont.] (+)

- Some challenges with FMEAs because of overlap with other functional groups
 - Want system designers to "own" the FMEA documentation but for PRA to be able to add our input (IE contribution, assoc. type code for each line item)
 - Document FMEA review to link to assumptions so that PRA treatment can be modified if design change triggers an FMEA change
 - Ex.: oxidation of DHS components assumed to not significantly affect heat transfer



Conclusions and Westinghouse perspective on future actions

- NEI 18-04 process generally clear but some areas still challenging for first application
- Need for practical guidance or OE sharing
 - Event sequence family grouping, treatment of component reliability data, and treatment of cumulative risk metrics
 - Underscores need for implementation experience prior to issuing RG 1.247
- NEI 18-04 drives interactions with other technical areas which is beneficial









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