NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E

	Excerpt or Section Number from NEI 20-07	Question or Feedback					
3							
Gene	ral or Overarching comment						
1.	The enclosure to SRM-SECY-22-0076 states: "The applicant must assess the defense in depth and diversity of the facility incorporating the proposed digital I&C system to demonstrate that vulnerabilities to digital CCFs have been adequately identified and addressed"	Defense in depth has always been part of NPP facilities. The assessment of the facilities defense in depth is not clear from the content of NEI 20-07 Rev. E. Therefore, NEI 20-07 Rev. E does not address the entire SRM.					
	NEI 20-07 Rev. E states: "This document provides a process for developing a new type of Diversity and Defense-in-Depth (D3) analysis. This document establishes a safety case using claims, arguments, and evidence to demonstrate that vulnerabilities to digital CCF have been adequately addressed. The safety case depends on outputs from EPRI engineering and diagnostic tools to provide evidence that supports claims and arguments described in this document."						
2.	NEI 20-07 generally credits EPRI DEG, HAZCADS, DRAM and says the outputs provide the evidence but does not specifically state what parts of these processes or what evidence is necessary and sufficient.	What are the acceptance criteria to determine whether the processes produces the desired result? What evidence or process outputs, <u>specifically</u> , should be examined?					
3.	Scoring of systematic control methods. Section 4.1.2 states "Once a set of systematic Control Methods has been identified for a given Loss Scenario, each Control Method is individually scored to provide an objective comparison of the relative effectiveness of the Control Methods. A scoring method is used as a tool to perform a qualitative assessment of the Control Method effectiveness. A scoring	 What is the technical basis for the validity of the scoring method? Provide an example that illustrates the scoring method. Generally, each different control method is good for addressing some specific source(s) of concern, but not others. The selection of control methods should, when taken together, broadly address all the sources of concerns. Please 					

Official	lleo	Only	Dro	nriotan	/ Inform	nation
Unional	050	Unity	- FIU	priotary	mon	nation.

	NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E			
-	Excerpt or Section Number from NEI 20-07	Question or Feedback		
	method removes potential bias in the qualitative assessment. Each Control Method is evaluated separately for its Control Method effectiveness and in combination when more than one Control Method is applied to an I&C element or relationship set of I&C elements."	explain how this concept is addressed by the process in NEI 20-07 Rev. E.		
4.	Various sections regarding guidance on non- light-water reactors (non-LWRs) and new LWRs	NEI 20-07, applicable to both operating and new LWRs and non-LWRs, is written with more details or focus on operating LWRs. Additional enhancements or clarifications regarding guidance for non-LWRs are needed as discussed in comments below. Examples include the use of risk metrics, acceptability of the probabilistic risk assessment used, and other guidance on non-LWRs under the licensing modernization project (LMP). The NRC staff suggest a comprehensive evaluation of NEI 20-07 on this topic. Most new LWRs have successfully employed the deterministic best-estimate coping analysis to address CCF concerns in accordance the Commission Policy in SRM-SECY-93-087 and may choose to follow the same deterministic approach for future LWR designs. NEI 20-07 should also address use of deterministic approach for addressing the digital I&C CCF		
5.	Applicability of NEI 20-07, Rev E to advanced reactors including microreactors	The NRC staff notes that some of the advanced reactors may not be vulnerable to potential DI&C CCFs of concern. For example, the inherent safety and/or passive features may demonstrate that the designs are safe for the CCF scenarios using the LMP process in RG 1.233. Another example may be that there may not be any HSSSR DI&C systems in some of these designs. The NRC staff suggests that NEI 20-07 consider this feedback and include any additional clarifications as necessary regarding the use of its guidance for this area.		

EXECUTIVE SUMMARY AND TABLE OF CONTENTS			
6.	"Historically, CCF has been addressed	Suggest deleting this sentence or modifying	
	through the implementation of independent	sentence to acknowledge other means of	

2 of 18

Official	lleo	Only	- Proprietan	Information
Unional	050	Unity	Fropriotar	, mormation.

-	NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E			
-	Excerpt or Section Number from NEI 20-07	Question or Feedback		
	and diverse Instrumentation and Control (I&C) systems."	addressing CCF or add "some" after "Historically".		
7.	"To prove that vulnerabilities to CCF have been adequately addressed, the D3 analysis must be able to demonstrate that: 1. Credible and likely sources of potential CCF have been identified and analyzed. 2. Each source of potential CCF has been reasonably prevented, mitigated, or adequately dispositioned."	The concept of "vulnerability to CCF" in SRM- SECY-22-0076 was understood to mean a situation where a CCF would produce unacceptable results. This quotation, and others in this document change the focus to sources of CCF, but this document does not describe or define what sorts of things sources of CCF are (e.g., people, equipment, procedures). There is an implication that not all sources of CCF can be identified, which is created by this document only addressing credible and likely sources of CCF.		
		mode (e.g., potential CCF) and failure mechanism (e.g., source of potential CCF)? Does each potential CCF have many sources?		
8.	"Credible and likely sources of potential CCF have been identified and analyzed."	The enclosure to SRM-SECY-22-0076 states: "In performing the defense-in-depth and diversity assessment, the applicant must analyze each postulated CCF using either best- estimate methods or a risk-informed approach or both."		
		How does addressing the credible and likely sources meet this aspect of the policy?		
9.	"This document provides the safety case which provides the details that demonstrates the output of the EPRI Digital Engineering Guideline (DEG), Hazards and Consequence Analysis in Digital Systems (HAZCADS), and Digital Reliability Analysis Methodology (DRAM) processes (References 13, 14, and 15) provide a D3 analysis addressing the SRM-SECY-22-0076 policy."	This statement is misleading and should be changed. This document provides a high-level overarching approach, but it does not provide details.		
10.	"The use of independent and diverse I&C systems may address some sources of CCF, but these systems do not sufficiently address other sources of CCF."	This needs further elaboration or a reference to where it is described which sources of CCF are addressed with independent and diverse systems, and which sources are not addressed		

-	NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E				
-	Excerpt or Section Number from NEI 20-07	Question or Feedback			
		using independence and diversity. Also, it is not clear from the balance of this document whether <i>all</i> sources of CCF are really being addressed with the proposed methodology. An explanation is provided that seems to address control logic sources of CCF but no explanation is provided regarding common cause sources of hardware- related failures.			
11.	"Independence and diversity are indeed useful design techniques; however, these design techniques should be used when supported by an engineering analysis."	It seems to imply that there may be other design techniques that do not need to be "supported by an engineering analysis." If engineering analysis is not being used for these other techniques, what proof is there to demonstrate that CCF has been sufficiently addressed?			
12.	"This document provides the safety case which provides the details that demonstrate the output of the EPRI Digital Engineering Guideline (DEG), Hazards and Consequence Analysis in Digital Systems (HAZCADS), and Digital Reliability Analysis Methodology (DRAM) processes (References 13, 14, and 15) provide a D3 analysis addressing the SRM-SECY-22-0076 policy."	In effect, this document does not provide an evidentiary safety case—rather at best, it attempts to describe a method one could use to develop a safety case.			
13.	"Tier 2 provides sub-claims and arguments that demonstrate the efficacy of the EPRI HAZCADS and DRAM processes to identify and establish the criteria for each applicant to demonstrate they adequately executed these processes."	It does not appear that such sub-claims and arguments are conclusive (see NRC staff comments on Section 5.3).			
SECT					
14.	"The use of independent and diverse I&C systems may address some sources of CCF, but these systems do not sufficiently address	What sources of CCF do independent and diverse I&C systems not address, that the proposed methodology of NEI 20-07 does			

 SECTION 1 INTRODUCTION

 14.
 "The use of independent and diverse I&C systems may address some sources of CCF, but these systems do not sufficiently address other sources of CCF."
 What sources of CCF do independent and diverse I&C systems not address, that the proposed methodology of NEI 20-07 does addresses?

 Explain or provide examples of which sources of CCF can be addressed via independence and diversity and which sources of CCF cannot but can be addressed by NEI 20-07.

-	NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E			
2	Excerpt or Section Number from NEI 20-07	Question or Feedback		
15.	"This process may be applied to operating reactor licensees or new plant applicants." "Applicants using this guidance for new plant applications using Regulatory Guide 1.233 can use this guidance to develop a D3 assessment to demonstrate the adequacy of special treatments applied to address CCF."	The NEI 20-07 process primarily focuses on risk metrics of CDF and LERF and corresponding thresholds for these risk metrics based on regulatory guidance for operating light-water reactors. Advanced light-water and non-light- water reactors do not use the same risk metrics, so it is not apparent that this process can be applied to all new plant applicants without changes, which will need a detailed review. There is a need to define the scope of NEI 20- 07.		
16.	"Independence and diversity are indeed useful design techniques; however, these design techniques should be used when supported by an engineering analysis."	It seems to imply that there may be other design techniques that do not need to be "supported by an engineering analysis." If engineering analysis is not being used for these other techniques, what proof is there to demonstrate that CCF has been sufficiently addressed?		

		01.
16.	"Independence and diversity are indeed useful design techniques; however, these design techniques should be used when supported by an engineering analysis."	It seems to imply that there may be other dest techniques that do not need to be "supported an engineering analysis." If engineering anal is not being used for these other techniques, what proof is there to demonstrate that CCF been sufficiently addressed?
SECT	ION 2 DEFINITIONS	
17.	NEI 20-07 defines a high-safety-significant safety-related as an SSC that has "one or more of the following3. Failure could lead directly to accident conditions that have	Does NEI 20-07 intend to make a distinction between the definition of high-safety-significa safety-related in NEI 20-07 and BTP 7-19?

	safety-related as an SSC that has "one or more of the following3. Failure could lead directly to accident conditions that have unacceptable consequences." This definition is broader than that in BTP 7-19, which uses the criterion, "their failure could lead directly to accident conditions that may have unacceptable consequences (e.g., exceeding siting dose guidelines for a DBE) if no other automatic systems are available to provide the safety function, or no preplanned manual operator actions have been validated to provide the safety function."	between the definition of high-safety-significant safety-related in NEI 20-07 and BTP 7-19?
18.	HSSSR SSC definition	For non-LWRs, the LMP in RG 1.233 classifies SSCs as safety-related, safety-related with special treatment, and non-safety-related with no special treatment based on a systematic risk- informed and performance-based approach. NEI

	NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E			
	Excerpt or Section Number from NEI 20-07	Question or Feedback		
		20-07's definition of HSSSR SSCs should include information on what SSCs are considered HSSSR under the LMP.		
19.	NEI 20-07 defines a risk reduction target as the "risk reduction to be achieved by the [] safety-related systems and/or other risk reduction measures in order to ensure that the tolerable risk is not exceeded."	Can SSCs that are not safety-related be credited to reduce risk to achieve the risk reduction target? How do the safety-related systems and/or other risk reduction measures <u>ensure</u> that the tolerable risk is not exceeded? [emphasis added]		
20.	NEI 20-07 uses the term "Stakeholder Losses" in several sections. It also uses the term "loss scenarios."	Please provide a definition of "Stakeholder Losses" and provide examples that help the reader to understand the Stakeholder Loss concept to differentiate such losses from those regulatory safety concerns (e.g., design basis events") that could result from digital I&C failures to perform required safety functions. Also, are both "Stakeholder Losses" and safety consequences of failures both considered "loss scenarios" described in Section 4.0? If not, please explain further.		
21.	Section 4.1 discusses controller "beliefs" and process model beliefs	Please define what is a controller "belief" and what is a process model "belief"?		

SECT	ION 3 REGULATORY BASIS	
22.	3.1 SRM-SECY-22-0076 "SRM-SECY-22-0076 provides NRC direction regarding an expanded policy on potential CCF <u>in HSSSR</u> DI&C systems." [emphasis added]	The insertion of "in HSSSR" is inconsistent with the policy. Therefore, this phrases is an incorrect characterization of the policy. The SRM states "The Commission has approved the staff's recommendation to expand the existing policy for digital instrumentation and control (I&C) common-cause failures"
		Please remove the words "in HSSSR".
23.	3.1.2 SRM-SECY-22-0076 Point 4 "BTP 7-19"	Since NEI 20-07 was written in July 2023; it does not accurately reflect what is in the new version of BTP 7-19. Therefore, such wording must be checked after the final version 9 of BTP 7-19 is issued. See Section B.1.2 for critical safety function.
24.	3.1.2 SRM-SECY-22-0076 Point 4 "For applicants using Regulatory Guide 1.233, special treatment considerations and human factors engineering processes should	Consider adding additional guidance for non LWR applicants on addressing Point 4 of SRM.

6 of 18

Official	lleo	Only	- Proprietary	Information
omorar	050	omy	roprotary	mormation.

).	NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E				
-	Excerpt or Section Number from NEI 20-07	Question or Feedback			
	indicate required monitoring parameters that support safety functions."	Not clear how guidance in NEI 20-07 is applied to non-LWR applicants, because a lot of the guidance appear to be LWR specific.			
		The NRC staff agrees with the statement but provides the following additional information. For non-LWRs, the LMP in RG 1.233, safety functions are defined and discussed mainly in terms of Required Safety Functions (RSFs) and Probabilistic Risk Assessment Safety Functions (PSFs). The NRC staff relies on RG 1.233 and the Design Review Guide (DRG), "Instrumentation and Controls for Non-LWRs Reviews" (ML21011A140) for non-LWR I&C reviews. The NRC staff will use pre-application engagement to discuss use of the expanded policy, including critical safety functions in Point 4 for non-LWRs with interested applicants to address any questions or concerns. A relevant discussion is in SECY-23-0092.			
25.	"the assumed definition of this term [critical safety functions] only includes functions pertinent to existing light water reactor designs."	The NEI 20-07 process primarily focus on risk metrics of CDF and LERF and corresponding thresholds for these risk metrics based on regulatory guidance for operating light-water reactors. Advanced light-water do not use the same risk metrics, so it is not apparent that this process can be applied to all new plant applicants without changes, which will need a detailed review. There is a need to define the scope of NEI 20-07. For non-LWRs, the LMP in RG 1.233 identifies technology-inclusive risk metrics for use, which is also discussed in ASME/ANS RA-S-1.4-2021,			
26.		endorsed in RG 1.247 with clarifications. The LMP also discusses the possibility of reactor- specific risk metrics by the designers as needed. Including some of the relevant information from the LMP on risk metrics in an appropriate section should enhance NEI 20-07 for clarity and being more technology-agnostic. Please clarify what "sensitivity analysis" is being referred to here.			

7 of 18

	NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E		
	Excerpt or Section Number from NEI 20-07	Question or Feedback	
]]	Please specify what would be the sensitivity analysis being referred to here. A postulated digital CCF either has an adverse risk impact or it doesn't. Alternatively, define the criteria that will be used to identify "significant risk impact" or "not a significant risk impact."	
SECT	ION 4 SYSTEM DIAGNOSTIC PROCESS	1	
27.	"Section 4.1.1 EPRI HAZCADS Overview For example, a realistic break opening time should be used to determine the necessary response time to a Large Break Loss of Coolant Accident in lieu of an assumed double-ended guillotine break)."	Provide a clarification or footnote indicating that this example would apply only to those piping subsystems for which the fracture mechanics have been analyzed and reviewed.	
28.		With the level of reliance placed on the execution of the EPRI HAZCADS and DRAM processes as part of the NEI 20-07 methodology, it appears that the NRC staff would need to review and endorse the HAZCADS and DRAM processes. Does NEI intend to provide these documents on the docket and seek their review and endorsement by NRC staff?	
29.	"As the system design matures in detail, new hazards may be uncovered and the list of hazardous system states can be revisited and revised, as needed."	What process is used to determine if the list of hazardous system states needs to be revisited and revised? How often is this process performed? Alternatively, describe how an iterative design process is used which continues to uncover new hazards as the design evolves.	
30.	NEI 20-07 states, "A control structure model does not typically capture purely physical relationships like physical proximity between components or fire propagation." Draft BTP 7-19, Revision 9, states, "the application should evaluate DI&C system interconnectivity and address DI&C system spatial separation that could significantly influence the risk due to fires, earthquakes, and other hazards."	How does the NEI 20-07 process address these spatial concerns?	

NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E		
-	Excerpt or Section Number from NEI 20-07	Question or Feedback
31.	"Realistic times should be considered in lieu of overly conservative estimates for improbable licensing basis events."	Does this statement apply to all CCFs or only those evaluated through the NEI 20-07 risk- informed process? How are "improbable" licensing basis events defined?
32.	"The RRT can be developed from one of five different pathways based upon the scope of the system under analysis, the stage of the design process, and whether the system(s) is modeled in the PRA."	It would be beneficial to include additional information on the five different pathways to develop the risk reduction targets.
33.	"The result may be a change in core damage frequency (CDF) and large early release frequency (LERF). Some reactor technologies may use different risk metrics specific to the reactor design. For those reactor technologies, the RRT thresholds should align with industry accepted guidance."	For advanced light-water and non-light-water reactors, what industry accepted guidance would be used to determine the risk reduction target? For advanced light-water and non-light-water reactors, what risk metrics and associated criteria that would be used to determine the risk reduction target? The NEI 20-07 process primarily focuses on risk metrics of CDF and LERF and corresponding thresholds for these risk metrics based on regulatory guidance for operating light-water
		reactors. Advanced light-water and non-light- water reactors do not use the same risk metrics, so it is not apparent that this process can be applied to all new plant applicants without changes, which will need a detailed review. There is a need to define the scope of NEI 20- 07.
34.	"For the purposes of this document, only loss scenarios associated with regulatory safety factors (e.g., core damage or radiological release) should be considered."	Are loss scenarios that do not result in core damage or radiological release but affect other regulatory programs such as MSPI and the maintenance rule considered?
35.	"A set of pre-scored systematic control methods are established to mitigate the loss scenarios of an inadequate control algorithm."	What process is used to provide scores to the control methods? Additional details and justification of the scoring process are necessary, including examples demonstrating how the scoring process is performed. The justification needs to explain how the scoring process is objective, structured, and consensus-based.

-	NRC Staff Questions and Feedba	ack on NEI 20-07, Draft Rev E
	Excerpt or Section Number from NEI 20-07	Question or Feedback
36.	"Refer to EPRI DRAM for details regarding the CME scoring methodology."	Does NEI intend for the NRC staff to review and endorse the DRAM process? Does NEI intend to provide this document on the
37.	4.2.1 EPRI HAZCADS Clarifications []	What requirements, if any, are provided for the technical acceptability of a PRA model? SECY-22-0076 provides guiding principles that the staff will follow, one of which is that "the underlying PRAs used for the bounding assessment as part of risk-informed approaches will be technically acceptable and will be supported by an effective PRA configuration control and feedback mechanism." To use the NEI 20-07 process for a risk-informed application, the applicants PRA models will need to be demonstrated to be technically acceptable.
38.	See the previous item.	For non-LWRs, RG 1.247 (For Trial Use), "Acceptability of Probabilistic Risk Assessment Results for Non-Light-Water Reactor Risk- informed Activities," provides guidance on the subject. Additional guidance on technical acceptability of a PRA such as a discussion on RG 1.247 related to non-LWRs should enhance NEI 20-07 for clarity.
39.	4.2.1 EPRI HAZCADS Clarifications	This description is solely applicable to LWRs with the "shall" language. It should be revised to be technology-agnostic or additional clarification should be made on guidance for non-LWRs.

10 of 18



SECT	SECTION 5 SAFETY CASE DEVELOPMENT				
41.	5 SAFETY CASE DEVELOPMENT "The safety case structure provided in this	NEI does not explain what was changed or what was adopted from the identified standards or			
	15026-2:2022	NEI understands to be a "safety case."			
42.	 5 SAFETY CASE DEVELOPMENT "The safety case is constructed by connecting key elements, which include: Claims which are assertions about a property of the system. Claims that are asserted as true without justification become assumptions and claims supporting the argument are called sub-claims. Arguments which link the evidence to the claim, which can be deterministic, probabilistic or qualitative. Evidence which supplies the basis for the justification of the claim. Some sources of evidence may include the design, the development process, testing, and inspections." 	In short, a safety case is built on three things: Claims, Reasoning, & Evidence. The reasoning explains how or why the evidence supports the claim. The reasoning part of the safety case in NEI 20- 07 is missing. In some cases, the "reasons" provided are just unsupported claims. For example			
		How is this argument/reason <u>not</u> just a restatement of the claim? The argument/reason does not explain why the evidence listed below it supports the claim. It is obvious that the EPRI processes identify some losses and hazards, but what makes us believe that they identify enough? Why? The same problem exists with the other Tier 2			

NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E Excerpt or Section Number from Question or Feedback NEI 20-07 5.1 Safety Case Structure 43. The last sentence appears to indicate that the Tier 3 evidence will not be in the licensing application but will be available for audit or inspection by the NRC staff. At a minimum, a summary of the Tier 3 evidence to support the corresponding argument should be included in the licensing application with the detailed documentation being available for a regulatory audit or inspection. 5.1.1 Safety Case Description 44. This is an unsupported claim. How do we know "The technical process described in EPRI it is true? HAZCADS and DRAM produces a diversity and defense-in-depth analysis that demonstrates vulnerabilities to digital CCF have been adequately identified and addressed." 5.1.1 Safety Case Description Item # 1 45 How does this compare with BTP 7-19 Section B.3.4? [How is the output of the 11]] used? This description is more applicable to LWRs 46 5.1.1 Safety Case Description regarding the use of Core Damage and Large Early Release. It should be revised to be technology-agnostic or additional clarification should be made on guidance for non-LWRs. The technology of establishing risk effectiveness 47. 5.1.2 Safety Case Uncertainty scores and applying them to individual control measure has not yet been demonstrated to be 11 adequate for regulatory purposes. Please describe how this adequacy will be demonstrated. 48. 5.1.2 Safety Case Uncertainty Traditionally (or historically), one did not try to determine the source of the CCF, but rather just [postulated the CCF and determined whether the results were acceptable or not. The US NRC (called AEC at the time) described the reasoning behind this traditional approach for example, in the Chapter 12 of the AEC HB on I&C Part 2 TID-25952-P2.

Official Use Only – Proprietary Information.



	NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E				
	Excerpt or Section Number from NEI 20-07	Question or Feedback			
		How is the implementation of the term []] risk informed?			
49.	5.2 Tier 1 Claim, Argument and Sub- Claims []	This text does not match the text in Figure []]]. There is a very big difference in the wording. Please ensure consistent wording throughout. Since there is an inconsistency in the description, it is not clear which one (or neither) you are proposing. Furthermore, there are other inconsistencies between the figure and the textual description of it. It would be helpful if the claims, arguments, & evidence in the figure were labeled the same as those in the text			
50.	5.3.1 EPRI HAZCADS and DRAM Efficacy "The graded approach is consistent with the acceptance guidelines for changes to Core Damage Frequency and Large Early Release Frequency described in RG 1.174 Section 2.4. Aspects of the proposed modification that result in changes to CDF or LERF that map to Region 1 in RG 1.174 Figures 4 and 5 apply the most rigorous approach; whereas those that map to Region 3apply the least rigor while maintaining the design basis commitments and consistency with the facility's defense-in-depth philosophy and safety margins."	This is another example where the description is more applicable to LWRs regarding the use of Core Damage and Large Early Release. Additional clarification should be made on guidance for non-LWRs.			
51.	5.3.1 EPRI HAZCADS and DRAM Efficacy "This process provides the system designers with greater insights to potential sources of failure and provides insights to the most risk- significant vulnerabilities that need to be addressed."	For "the most risk-significant vulnerabilities," should it be "risk-significant vulnerabilities"? It is not clear why it is focused on the most risk- significant item only.			
52.	5.3.1 EPRI HAZCADS and DRAM Efficacy "RG 1.233 provides the scope of functions under control and reliability targets for a safety-related DI&C system via the Licensing Basis Event selection and SSC classification (including defense-in-depth functions). These	RG 1.233 covers all <u>safety-significant</u> SSCs including both safety-related and non-safety- related with special treatment (NSRST) SSCs. The NRC staff suggest using "safety-significant" instead of "safety-related" in the statement.			

-

Official Use Only – Proprietary Information.

Г

NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E

	Excernt or Section Number from	
	NEI 20-07	Question or Feedback
-	criteria are inputs to the initial/conceptual design phase."	
53.	"EPRI HAZCADS and DRAM have been proven effective in identifying and addressing hazards and sources of failure in DI&C systems NRC has conducted its own research on the efficacy of hazards analysis and STPA. TLR-RES/DE-2022-006, "Hazard Analysis: An Outline of Technical Bases for the Evaluation of Criteria, Methodology, and Results," documents an evaluation of the need "to develop criteria for technical bases supporting the evaluation of the criteria and methodology for, and of the results from, [] hazards analysis."	The staff can recognize how the processes described can provide insights toward attaining a degree of reliability of operations as a complement to existing regulatory activities. However, it is not clear whether these processes alone, without the complementary regulatory activities are effective at identifying and eliminating all sources of CCF, which is the purpose of this document.
54.	Risk Informed Principles subsection: "2. Key assumptions and sources of uncertainty in the PRA models that can impact the assessment are addressed by assuming everything in the HSSSR system fails. By assuming the CCF occurs, uncertainty associated with the HSSSR DI&C system is a negligible factor since this process provides a bounding assessment of the failure of the HSSSR DI&C system. Because this process requires the use of a high-fidelity PRA model, other sources of uncertainty (e.g., parameter uncertainty) are unaffected by the sensitivity analysis performed by this process."	Do the high-fidelity aspects of the PRA model extend to the modeling of the operator's response—e.g., including recognizing that a complete or partial failure of the HSSR has occurred, and then taking appropriate manual actions to identify process trends and then correctly address the symptoms of the event? Doesn't the operator response portion of the PRA rely on rule of thumb assumptions regarding successful correct and timely operator actions? One reason it is assumed that failure of the HSSSR DI&C system results in a negligible change in risk factor, is that these operator action success assumptions are overly optimistic.
55.	5.4.1 Resolution of Tier 2, Sub-Claim 1	The use of the term "argument" here in the first sentence is understood to be used in the sense of "claims, reasoning & evidence" and not simply a statement to include the bolded text in the application. Consider using the term "claims, reasoning & evidence" instead of "argument" in all such cases. Otherwise, it is confusing with respect to the use of the term "argument" in the figures. This [[]] does not contain reasoning about why the evidence supports the claim. But rather is just an unsupported claim.



Official Use Only - Proprietary Information.

NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E Excerpt or Section Number from Question or Feedback NEI 20-07 technology-agnostic or additional clarification should be made on guidance for non-LWRs. 11 61. 5.4.2 Resolution of Tier 2 Sub-claim 2. Relevant PRA information including credited external design features (e.g., manual operator E actions, passive design features) need to be of a sufficient high fidelity to provide an accurate picture as to the likelihood of success of each operator response action in the event of a common cause failure of the HSSSS. It should not rely on handbook-based canned assumptions, without formal validation.]] 62. NEI 20-07 lists the following EPRI reports as Does NEI intend to provide these reports available evidence: EPRI 3002004995, EPRI document on the docket? 3002004997, and EPRI 3002000509. "A combined control method effectiveness 63. Is the phrase "geometrically weighted value" score provides a geometrically weighted intended to represent a weighted geometric value." mean or a different mathematical value? Additional details and justification of the geometrically weighted combined control method effectiveness score calculation are necessary, including examples demonstrating how the combined control method effectiveness score is calculated. The justification needs to explain how the calculation is objective. structured, and consensus based. 64 [11

Official Use Only - Proprietary Information.

16 of 18

7	NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E			
÷.	Excerpt or Section Number from NEI 20-07	Question or Feedback		
SECT	TION 6 CONCLUSION			
65.	"UCAs that are present in multiple redundancies of a DI&C system and impact core damage or large early releases are considered CCF."	This definition of CCF is not consistent with the NRC understanding of the term. Using different definitions for terms than the NRC uses only creates regulatory uncertainty.		
		Maybe it is better to say "considered <u>risk/safety</u> <u>significant</u> CCFs" and that control measures are applied to these CCFs.		
66.	"UCAs that are present in multiple redundancies of a DI&C system and impact core damage or large early releases are considered CCF."	This description is more applicable to LWRs regarding the use of core damage and large early release. It should be revised to be technology-agnostic or additional clarification should be made on guidance for non-LWRs.		
67.	"This process is effective at identifying the most likely and credible CCFs at a nuclear power plant."	The NRC is also concerned about CCFs that are unlikely. This conclusion should probably be a bit more precise in terms of Modes, causes, mechanisms, or sources of CCF.		

SECTION 7 REFERENCES	
68.	The document text often does not identify the specific reference. Please ensure all references are used refer to the reference number in the body of the document.

APPE	APPENDIX A. RELEVANT NRC REGULTORAY FRAMEWORK			
69.	Appendix A "This Appendix describes the relationship between the process described in this document and the NRC regulatory framework." "Note that the regulations listed below may not necessarily apply to all applicants and licensees. The applicability of the regulatory requirements is determined by the plant- specific licensing basis and any proposed changes to the licensing basis associated with the proposed DI&C system under evaluation."	Conceptually, NEI 20-07 is proposed to be used as an alternative way to meet the Commission policy on CCF; therefore, this appendix should explicitly include the NRC regulatory framework applicable to the Commission policy on CCF. It appears that this appendix is incomplete in that respect. For example, it does not include the SRM. The NRC regulatory framework includes more than just regulatory requirements.		
70.	Appendix A, Section A1 states "A.1. 10 CFR 50.54(jj), 10 CFR 50.55a(h) IEEE 603-1991 or IEEE 279 -1971 as incorporated by reference requires, in part, that components	10 CFR 50.55a(h) also incorporates by reference IEEE 279-1968; therefore, please add this regulatory requirement to this section.		

	NRC Staff Questions and Feedback on NEI 20-07, Draft Rev E				
i.	Excerpt or Section Number from NEI 20-07	Question or Feedback			
	and modules shall be designed, manufactured, inspected, installed, tested, operated, and maintained in accordance with a prescribed quality assurance program."	Please add GDC 1 and 10 CFR50.55(i) to this section. The requirement that an NPP is constructed and operated in accordance with a quality assurance program is not imposed by GDC1, 10 CFR 50.55a(h), 10 CFR 50.54.(jj), or 10 CFR 50.55(i) – as this section states. The QA program is imposed by other regulatory requirements. GDC 1, 10 CFR 50.54.(jj), and 10 CFR 50.55(i) impose the requirement to do things in			
		accordance with established standards. A 10 CFR Appendix B compliant QA program ensures the standards are achieved. The establishment of standards to ensure equipment meets its obligations in the FSAR is a technical matter and not a programmatic QA matter. The quotation seems to confuse these issues and thereby create regulatory uncertainty. For example, RG 1.75 establishes standards for separation, and if committed to, the QA program ensures the standards established are conformed to.			
71.	Appendix A, Section A.1	The term "quality standards" is used in several places in the regulations and guidance e.g., GDC 1, 10 CFR 50.54(jj), 10 CFR 50.55(i), AEC I&C Handbook Volumes 1 & 2, RG 1.26 Rev. 5, SECY-03-0117, GL 84-01. This appendix should use the term "quality standards" in a manner consistent with how the NRC uses the term.			
72.	Appendix A, Section A.2.1 "Pre-scored Systematic Control Methods are techniques and measures that may, synthesized from the industry standard IEC 61508 Part 3, normative Annex A which is a recognized safety standard in the petrochemical industry."	Doing a part of a standard is not the same as following the standard. These use of these methods in this document differs from how they are used in the standard.			
*The portion markings on this document was changed on March 26, 2024 to reflect the redactions in NEI 20-07, Draft Revision E that was attached to NEI's letter dated March 14, 2024 (<u>ML24074A459</u>). No other changes were made to this document.					