~~SECURITY-RELAED INFORMATION~~
~~Withhold Under 10 CFR 2.390~~

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HDI PNP 2023-002

10 CFR 50.71(e)
10 CFR 50.4(b)(6)

March 31, 2023

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Palisades Nuclear Plant
Docket No. 50-255
Renewed Facility Operating License No. DPR-20

Subject: Final Safety Analysis Report Update Revision 36

In accordance with Title 10 of the Code of Federal Regulations (10 CFR) Sections 50.71, *Maintenance of records, making of reports*, paragraph (e), 10 CFR 50.71(e), and 50.4, *Written communications*, paragraph (b)(6), 10 CFR 50.4(b)(6), Holtec Decommissioning International, LLC (HDI) on behalf of Holtec Palisades, LLC (Holtec Palisades) is providing the Palisades Nuclear Plant (PNP) Final Safety Analysis Report (FSAR) update, Revision 36. Revision 36 includes changing the FSAR title to Defueled Safety Analysis Report (DSAR) reflecting the transition of PNP to a permanently defueled facility.

Since incorporation of the DSAR changes resulted in a major rewrite of the FSAR, revision bars were not used to denote the changes. All changes, other than those involving typographical corrections, format changes, and removed obsolete information, were made under the provisions of 10 CFR 50 or in accordance with safety evaluations received from the Nuclear Regulatory Commission (NRC). The FSAR update incorporates changes made to the facility or the procedures described in the FSAR, and all other applicable information and analyses submitted to the NRC or prepared pursuant to NRC Requirements, up to October 14, 2022, which is 18 months since the last UFSAR submittal on April 14, 2021.

DSAR Revision 36 is provided, in its entirety, on the CD-ROMs in the enclosures.

Enclosure 1 contains the list of changes included in FSAR Revision 36. Attachment 1 to Enclosure 1 contains the list of changes specific to system classification changes as a result of transitioning to a defueled facility. Attachment 2 to Enclosure 1 contains the list of changes specific to the implementation of the permanently defueled technical specifications at PNP.

Enclosure 2 contains a public version of the updated FSAR (Revision 36), with certain information redacted in accordance with NRC Regulatory Issue Summary (RIS) 2015-17,

NOTICE: Enclosure 3 to this letter contains sensitive unclassified information. Upon separation of Enclosure 3 from this letter, this document becomes "DECONTROLLED."

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NRR

~~SECURITY-RELAED INFORMATION~~
~~Withhold Under 10 CFR 2.390~~

Review and Submission of Updates to Final Safety Analysis Reports, Emergency Preparedness Documents, and Fire Protection Documents.

Enclosure 3 contains a non-public version (non-redacted) of the updated FSAR (Revision 36). Enclosure 3 contains security-related information, and HDI requests that this enclosure be withheld from public disclosure under 10 CFR 2.390, *Public Inspections, exemptions, and requests for withholding*, paragraph (d)(1).

For this FSAR update, no information has been removed in accordance with Appendix A to Nuclear Energy Institute (NEI) 98-03, Revision 1, *Guidelines for Updating Final Safety Analysis Reports*, as endorsed by Regulatory Guide 1.181, *Content of the Updated Final Safety Analysis Report in Accordance with 10 CFR 50.71(e)*.

This letter contains no new regulatory commitments and no revised commitments.

Should you have any questions or require additional information, please contact Jim Miksa, Regulatory Assurance Engineer at (269) 764-2945.

I declare under penalty of perjury that the foregoing is true and correct. Executed on March 30, 2023.

Respectfully,

Jean A. Fleming Digitally signed by Jean A. Fleming
Date: 2023.03.31 10:43:00 -04'00'

Jean A. Fleming
Vice President, of Licensing, Regulatory Affairs & PSA
Holtec International

Enclosures
1: Palisades Nuclear Plant Final (Defueled) Safety Analysis Report (FSAR/DSAR) Revision 36 List of Changes

2: CD-ROM Containing Final (Defueled) Safety Analysis Report (FSAR/DSAR) Revision 36 (Public Version)

3: CD-ROM Containing Final (Defueled) Safety Analysis Report (FSAR/DSAR) Revision 36 (Non-Public Version) (Security-Related Information, Withhold Under 10 CFR 2.390)

cc: NRC Region III Regional Administrator
NRC Decommissioning Inspector – Palisades Nuclear Plant
NRC NMSS Project Manager – Palisades Nuclear Plant

NOTICE: Enclosure 3 to this letter contains sensitive unclassified information. Upon separation of Enclosure 3 from this letter, this document becomes "DECONTROLLED."

Enclosure 1 to

HDI PNP 2023-002

Palisades Nuclear Plant

Final (Defueled) Safety Analysis Report (FSAR/DSAR) Revision 36

List of Changes

Palisades Nuclear Plant
Final (Defueled) Safety Analysis Report (FSAR/DSAR) Revision 36
List of Changes

Log No.	Affected FSAR/DSAR Sections, Figures, and Tables	Description of Change
18-0032	Figure 9-1 Sh 1A and Figure 9-1 Sh 2	EC 78385 installs a new skid mounted Sodium Hypochlorite delivery method for the Service Water Pump Bay and the Cooling Tower/Condenser loops (A&B). FSAR Figure 9-1 Sh 1A and 9-1 Sh 2 are impacted by this change.
21-001	FSAR Figure 11-1 Sh 2, FSAR Chapter 4 References, FSAR Section 4.3.5, FSAR Section 4.7, FSAR Table 11-2, FSAR Table 11-5	EC 88999, "Primary Coolant Pump Uncontrolled Bleed-off Re Alignment to Containment Sump," reconfigured the Flow Path of the Primary Coolant Pump vapor seal leakage from the Primary System Drain Tank to the Containment Sump. The DSAR revisions to the FSAR superseded the changes to FSAR Chapter 4 References, FSAR Section 4.3.5, FSAR Section 4.7, and FSAR Table 11-2.
21-002	Figure 6-2 Sh 1B	Editorial change to FSAR Figure 6-2 Sh 1B. The figure has been revised to correct a reference to E-904 Sh 1.
21-004	Appendix 7A Section 7A.1.1.1	Surveillance Test Interval (STI) evaluation, STI-2021-001, changed the Tech Spec surveillance requirements (3.3.2.1 and 3.3.3.2) from 92 to 184 days in accordance with the Surveillance Frequency Control Program. FSAR sections were revised to reflect the changes in frequency. The DSAR revisions to the FSAR superseded the changes to FSAR Appendix 7A Section 7A.1.1.1.

Log No.	Affected FSAR/DSAR Sections, Figures, and Tables	Description of Change
21-005	Figure 9-9 Sh 1	EC 89044, "C-7 Back up Extractor." This EC adds a new manual valve to the Service Air System to throttle service air for C-917 "C-7 Back up Extractor".
21-006	Figure 9-18 Sh 1A	EC 90144 changed the position of MV-DMW773, "Spent Fuel Pool PMW Fill Isolation," from locked closed to closed.
21-007	Section 9.8, Section 9 References, and Table 9-1	EC 89606, "Revise Engineered Safeguards Room Heatup Calculations," replaces references to EA-D-PAL-93-242F-01, Revision 1, "Engineered Safeguards Room Heatup following LOCA in Conjunction with a LOOP," with EA-EC516-05, Revision 1, "Calculation of Engineered Safeguards Room Temperature Following a LOCA." The DSAR revisions to the FSAR superseded the changes to FSAR Section 9.8, Section 9 References, and Table 9-1.
21-008	Figure 8-3, Sh 2	Administrative change to reinstate Fig.8-3, Sh 2 that was removed in error during a previous FSAR update.
21-010	Figure 9-1 and Table 9-1	EC 90291 modifies the non-critical service water system by adding a new pipeline, heat exchanger and valves to support the addition of E-924, "Plant HTG Condensate Return Heat Exchanger."
21-011	Figure 10-1	Administrative change to Figure 10-1. To replace an electronic document copy that was illegible when magnified with a clearer electronic copy.

Log No.	Affected FSAR/DSAR Sections, Figures, and Tables	Description of Change
21-012	Section 2.6.3 and Table 2-20	Offsite Dose Calculation Manual (ODCM) Radiological Environmental Monitoring Program (REMP), Revision 32 change which enhanced the REMP thermoluminescent dosimeters (TLD) program by reducing the number of control TLDs from 3 to 1 and relocating 4 TLDs. FSAR revised to reflect the reduction in control TLDs.
22-001	See Attachment 2	Renewed Facility Operating License (RFOL) Amendment 266, Administrative Controls for a Permanently Defueled Condition, Amendment 272, Technical Specifications for Permanently Defueled Condition, and Exemption from 10 CFR 73.55 allowing CFH to suspend security measures implementation. Amendment 272 retitled the PNP UFSAR to a Defueled Safety Analysis Report (DSAR) which revised every UFSAR page. See HDI PNP 2023-002 Enclosure 1, Attachment 2 for specific change details.
22-002	Section 1.9.1.9, Chapter 1 References, Number 36	ADMIN 7.28, "Diesel Fuel Oil Program," procedure implementation to reflect Palisades Permanently Defueled Technical Specifications. FSAR revised to remove reference to a duplicate ASTM diesel fuel centrifuge method.
22-003	Appendix 7A Section 7A.1.1.1, Appendix 7A Section 7A.1.1.3	Surveillance Test Interval (STI) evaluation, STI-2022-001, changed the Tech Spec surveillance requirements (3.3.1.5 and 3.3.3.2) from 92 to 184 days in accordance with the Surveillance Frequency Control Program. FSAR sections were revised to reflect the changes in frequency. The DSAR revisions to the FSAR superseded the changes to FSAR Appendix 7A Section 7A.1.1.1 and Appendix 7A Section 7A.1.1.3.

Log No.	Affected FSAR/DSAR Sections, Figures, and Tables	Description of Change
22-005	Section 11.5.4	Offsite Dose Calculation Manual (ODCM), Revision 33, incorporated changes to the ODCM due to implementation of Renewed Facility Operating License Amendment 272, Permanently Defueled Technical Specifications. ODCM Revision 33 changes resulted in revision to FSAR section 11.5.4.
22-006	Section 5.1.3.8, Section 8.1.2, Section 8.2.2, 8.2.3, Section 8.3.1.1, 8.3.1.2, 8.3.1.3, Section 8.3.2.1, 8.3.2.2, 8.3.2.3, Section 8.4.1.2, Section 8.6.2, Figure 7.14 Sh 2, Figure 7.14 Sh 10, Figure 8.1 Sh 1, Figure 8.2, Figure 8.3 Sh 1	EC92138, Permanently Air Gap Motor Operated Switch 26H5 to Support Unenrollment from NERC Following Permanent Plant Shutdown, removed MOD-26H5 to disconnect the Main Transformer (EX-10) from the grid.

Log No.	Affected FSAR/DSAR Sections, Figures, and Tables	Description of Change
22-007	Section 1.1.2, Section 2.1, Section 2.5.2.3, Section 5.2.2.8, Section 12.1.1, Section 12.2.1.2, Section 12.3.1, Section 12.4, Section 15.1.1, 15.1.2 Figure 12-1 Table 1-1 Table 1-9 Table of Contents	Renewed Facility Operating License (RFOL) Amendment 273, Transfer of Palisades Facility ownership and operating license from Entergy Nuclear Palisades, LLC to Holtec Palisades, LLC and from Entergy Nuclear Operations, Inc. to Holtec Decommissioning International, respectively. Implementation of RFOL Amendment 273 revised the listed DSAR sections to reflect this change in ownership.
22-010	Section 8.2 Figure 7.14 Sh. 1 Figure 7.14 Sh. 2 Figure 7.14 Sh. 10	EC 92298, 25F7, *Generator Breaker*, and 25H9, *Generator Breaker*, relay modification to remove trip function that opens the main generator output breakers. The DSAR revisions to the FSAR superseded the changes to Figure 7.14 Sheet 1, Figure 7.14 Sh. 2, and Figure 7.14 Sh. 10. Section 8.2 changes were incorporated into the DSAR.

Log No.	Affected FSAR/DSAR Sections, Figures, and Tables	Description of Change
22-012	Section 3.3 Section 4.2, 4.3, 4.5, 4.8 Section 5.1, 5.2, 5.5, 5.7, 5.8 Section 6.1, 6.2, 6.3, 6.4, 6.10 Section 7.1, 7.2, 7.4, 7.6, 7.7 Section 8.1, 8.2, 8.3, 8.4, 8.5, 8.7 Section 9.1, 9.2, 9.3, 9.4, 9.5, 9.7, 9.8, 9.10, 9.11 Section 10.1 Table 4.2 Table 5.2-1, 5.2-2, 5.2-3, 5.2-4, 5.2-5 Table 5.4-1 Table 9.1 Table 9.9	EC92577, "Decommissioning Equipment Classification and FSAR Markups," revised structures, systems, and component (SSC) safety classifications to reflect a permanently defueled reactor. The FSAR revisions were made to reflect these changes. See HDI PNP 2023-002 Enclosure 1, Attachment 1 for specific change details.
22-013	Figure 7.14 Sh. 10	EC92493- "Administrative Engineering Change to correct drawing E0017-0017 (Rev. 3) to show the proper logic per CR-PLP-2022-00392" EC92493. The DSAR revisions to the FSAR superseded the changes to Figure 7.14 Sheet 10.
22-014	Section 8.4 Section 8.5 Section 8.9 Section 9.5 Section 9.6 Section 9.10 Table 9-21 Table of Contents	Renewed Facility Operating License Amendment 272, Defueled Technical Specifications revised the Palisades fire protection program requirements. These changes to the FSAR were made to reflect implementation of the defueled fire protection program requirements at Palisades.

Log No.	Affected FSAR/DSAR Sections, Figures, and Tables	Description of Change
22-017	Section 1.8 Section 12.1 Section 15.1 Table 1-5 Table of Contents	Renewed Facility Operating License Amendment 273, the transfer of Palisades RFOL and General license for Dry Fuel Storage from Entergy Nuclear Operations, Inc. (Entergy) to Holtec Decommissioning International, LLC (HDI) implementation included the adoption for the DDI Decommissioning Quality Assurance Program (DQAP) at Palisades. The changes to the FSAR were made to reflect implementation of the DQAP at Palisades.

Attachments:
1. LBDCR Log Number 22-012, Decommissioning Equipment Classification, Description of Changes - Details
2. LBDCR Log Number 22-001, Permanently Defueled Technical Specifications, Description of Changes – Details

Attachment 1 to Enclosure 1

HDI PNP 2023-002

Palisades Nuclear Plant

LBDCR Log Number 22-012

Decommissioning Equipment Classification

Description of Changes - Details

Palisades Nuclear Plant LBDCR Log Number 22-012
Description of Changes – Details

Section	Title	Change	Description of Change
3.3	Reactor Design	Modify	Add statement that the reactor design information in Section 3.3 will be historical/information only and not impose design requirements following defuel. Mark design requirements (ASME Classes, Seismic Classes, etc. as historical). The reactor will no longer perform a design function.
4.2	Design Basis (Primary Coolant System Chapter)	Modify	Add statement that the PCS design information will be historical/information only and not impose design requirements following defuel.
4.3	System Design and Operation	Modify	Add statement that the PCS design information will be historical/information only and not impose design requirements following defuel.
4.5	Tests and Inspection	Modify	Add statement that the PCS design information will be historical/information only and not impose design requirements following defuel.
4.8	Primary Coolant Gas Vent System	Modify	Add statement that the PCS design information will be historical/information only and not impose design requirements following defuel.
5.1.7.2	General Design Criteria – Criterion 61 – Fuel Storage and Handling and Radioactivity Control	Modify	Revise statement that Shutdown Cooling may be utilized to augment SFP cooling with Service Water System instead. This change is not adverse – procedures exist to provide Service Water to the Spent Fuel Pool, the system capacity of the Service Water System is sufficient to provide SWS to SFP, and the SWS is a Seismic Category I system.

Section	Title	Change	Description of Change
5.2	Classification of Structures, Systems, and Components	Modify	5.2.1.1 - Add discussion that SSCs were subjected to reclassification upon decommissioning of the facility. 5.2.2.4 – Note that QAPM procedures were still utilized to reclassify SSCs for decommissioning. 5.2.2.7 – Note that EQ and 1E requirements are no longer applicable in accordance with 10 CFR 50.49(a). 5.2.2.8.2 – Clarify definition of Important to Safety and Augmented Quality.
5.5	Missile Protection	Modify	Remove "safety related" in 5.5.1.3.2 and 5.5.4 as the components listed in the sections are no longer SR.
5.7	Seismic Design	Modify	Add reference to DSAR Tables 5.2-2, 5.2-3, 5.2-4, and 5.2-5 for seismic classification. The design method of seismic evaluation is not altered.
5.8	Containment Structure	Modify	Note the design information for containment is historic, but that the containment must retain its structural integrity during decommissioning.
6.1	Safety Injection System	Modify	Throughout the section, note that the design requirements are historical as the systems are no longer SR or AQ.
6.2	Containment Spray System	Modify	Throughout the section, note that the design requirements are historical as the system is no longer SR or AQ.
6.3	Containment Air Coolers	Modify	Throughout the section, note that the design requirements are historical as the system is no longer SR or AQ.
6.4	Containment Sump pH Control	Modify	Throughout the section, note that the design requirements are historical as the system is no longer SR or AQ.

Section	Title	Change	Description of Change
6.10	Control Room Habitability	Modify	Throughout the section, note that the design requirements are historical as the system is no longer SR or AQ.
7.1	Instrumentation and Controls Introduction	Modify	Note that Class 1E is no longer applicable per 10 CFR 50.49(a).
7.2	Reactor Protective System	Modify	Note that RPS "was" Class 1E, instead of "is" Class 1E.
7.4	Other Safety Related Protection, Control, and Display Systems	Modify	Note that the systems listed in the section were reevaluated following permanent defuel.
7.6	Plant Process Computer and Miscellaneous Instrumentation	Modify	Note that Seismic II/I am no longer a concern for the PPC as it is longer in the proximity of the remaining augmented quality electrical cabinets.
7.7	Operating Control Stations	Modify	Remove "safety related" as the cabinets are no longer safety related.
8.1	Electrical Systems Introduction	Modify	Note that Class 1E is no longer applicable per 10 CFR 50.49(a) and that information within is historic.

Section	Title	Change	Description of Change
8.2.4	Transmission System Ownership	Modify	Remove historic / obsolete information regarding switchyard ownership (former companies listed).
8.3	Station Distribution	Modify	Note that Class 1E is no longer applicable per 10 CFR 50.49(a) Note that Seismic Categories are per Table 5.2-4 for electrical equipment and that other references to seismic design are historical.
8.4	Emergency Power Sources	Modify	Note that Class 1E is no longer applicable per 10 CFR 50.49(a) Note that Seismic Categories are per Table 5.2-4 for electrical equipment and that other references to seismic design are historical.
8.5	Raceway and Cabling Systems	Modify	Add "historically" to "safety-related" as the cables described in the section are not specific and some are and are not safety-related post-operation.
8.7	Physical Separation and Support Systems	Modify	Add note that the information in this section is historically related to the "two train" separation original design theory.
9.1	Service Water System	Modify	Note that the critical system is provided to support Spent Fuel Pool Cooling via Component Cooling Water.
9.2	Reactor Primary Shield Cooling System	Modify	Throughout the section, note that the design requirements are historical as the system is no longer SR or AQ.
9.3	Component Cooling System	Modify	Note that sometimes in the defueled condition, zero CCW pumps may be running (as the section currently states one or two pumps will be running). This is to maintain the temperature of the SFP within its analyzed bands.

Section	Title	Change	Description of Change
9.4	Spent Fuel Pool Cooling System	Modify	Revise SFP makeup sources from Safety Injection Refueling Water Tank and Recycled Boric Acid Storage Tank to Service Water System and Fire Protection System. These sources provide equivalent design levels of makeup (that is, SWS is Seismic Category I as the SIRWT was (although, later determined not to be as described in other sections of the FSAR) and FPS is not, similar to the Recycled BAST). As such, the same level of inventory makeup is available. The FSAR and DSAR allow for unborated makeup sources. Remove SDC connections consistent with 5.1.
9.5	Compressed Air Systems	Modify	Throughout the section, note that the design requirements are historical as the system is no longer SR or AQ.
9.7	Auxiliary Feedwater System	Modify	Throughout the section, note that the design requirements are historical as the system is no longer SR or AQ.
9.8	Heating, Ventilation, and Air-Conditioning System	Modify	Throughout the section, note that the design requirements are historical as the system is no longer SR or AQ.
9.10	Chemical and Volume Control System	Modify	Throughout the section, note that the design requirements are historical as the system is no longer SR or AQ.
9.11	Fuel Handling and Storage Systems	Modify	Revise SFP makeup sources consistent with 9.4.

Section	Title	Change	Description of Change
10.1	Steam and Power Conversion System Design Basis	Modify	Throughout the section, note that the design requirements are historical as the system is no longer SR or AQ.

Attachment 2 to Enclosure 1

HDI PNP 2023-002

Palisades Nuclear Plant

LBDCR Log Number 22-001

Permanently Defueled Technical Specifications

Description of Changes - Details

Palisades Nuclear Plant LBDCR Log Number 22-001
Description of Changes - Details

Section	Title	Change	Description of Change
All	Use of "plant," "station," and "unit"	Modify	The UFSAR is modified to replace the words "plant," "station" and "unit" with the word "facility." This is an administrative change to reflect that PNP will be permanently shut down and defueled. As a result, power operations and electrical generation will no longer occur. The principal activities will be the safe storage of spent nuclear fuel and the management of radioactive wastes. Given that status, PNP is better described as a facility versus a plant. This justification applies to all uses of "plant," station" and "unit" throughout the UFSAR which are modified to "facility." "FSAR" is replaced with "DSAR" as this document is referred to as Defueled Safety Analysis Report" since Amendment 272 was implemented.
1.1	Introduction	Modify	This section is modified to replace the word "plant" with the word "facility." The discussion of licensing activity during operation has been replaced with a discussion of licensing related to Amendment 272 (cessation of operation) and a description of the DSAR.
1.1.1	General	Modify	This section is revised to add recent licensing history related to Amendment 272 and the defueled condition
1.1.2	Licensing History	Modify	This section is modified to replace the word "plant" with the word "facility," add discussion of the licensing actions related to permanent shutdown and defueling, and delete reference to plant operating limits. After implementation of Amendment 272, plant operation can no longer occur and plant operating limits are no longer applicable.

Section	Title	Change	Description of Change
1.2	General Plant Description	Modify	The title of this section is modified to replace the word "Plant" with the word "Facility."
1.2.1	Plant Site	Modify	The title and text of this section is modified to replace the word "Plant" with the word "Facility."
1.2.2	Plant Arrangement	Modify	The title and text of this section is modified to replace the word "Plant" with the word "Facility." This section is further modified by eliminating the description of fuel transfer, which is not credited or required in the defueled condition and removing discussion regarding resin leakage. After implementation of Amendment 272, fuel can no longer be placed in the reactor vessel and fuel transfer cannot occur.
1.2.3	Containment	Modify	This section is modified by eliminating discussion of the design functions and capability of the containment and associated SSCs to withstand internal pressure associated with a loss of coolant accident. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, this FSAR section is modified to remove design and safety functional requirements.

Section	Title	Change	Description of Change
1.2.4	Nuclear Steam Supply System (NSSS)	Modify	This section is modified by eliminating discussions of design functions of the NSSS and support systems. Pressure Control System, Reactor Control, CVCS, Chemical Treatment, Nuclear Control and Instrumentation, and Shutdown Cooling are deleted in their entirety. Physical descriptions of SSCs are retained. Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, the design functions of NSSS and support system are no longer relevant.
1.2.5	Turbine Generator	Modify	This section is modified by eliminating discussions of design functions of the turbine generator. Physical descriptions of SSCs are retained. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, the design functions of the turbine generator are no longer relevant.

Section	Title	Change	Description of Change
1.4.1	Station Design	Modify	The title and text of this section is modified to replace the word "Station" with the word "Facility." This section is modified by removing discussion of operating parameters and transients and limiting discussion to design codes and standards. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible.
1.4.2	Reactor	Delete	This section is proposed to be deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore there are no pertinent reactor design criteria or design functions in the permanently defueled condition.

Section	Title	Change	Description of Change
1.4.3	Primary Coolant and Auxiliary Systems	Delete	This section is proposed to be deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore there are no pertinent design criteria or design functions for the primary coolant and auxiliary Systems in the permanently defueled condition.
1.4.4	Containment System	Modify	This section is modified by eliminating the discussions of the capability of the containment to withstand internal pressure associated with a loss of coolant accident. In addition, the section is updated to reflect that the containment must maintain its structural integrity in the permanently shutdown and defueled condition to ensure that it does not impact the safe storage of spent fuel. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
1.4.5	Engineered Safeguards	Delete	This section is proposed to be deleted in its entirety. Amendment 272 removed Tech Spec Section 3.5, ECCS, and the facility license no longer contains requirements for the ECCS and associated systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no pertinent engineered safeguards design criteria or design functions in the permanently defueled condition.
1.4.6	Instrumentation and Control	Delete	This section is proposed to be deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit any active systems. Therefore, there are no pertinent instrumentation or control design criteria or design functions in the permanently defueled condition.

Section	Title	Change	Description of Change
1.4.7	Electrical Systems	Modify	This section is modified to replace the word "plant" with the word "facility." Amendment 272 removed Tech Spec Section 3.8, Electrical, and the facility license no longer contains requirements for electrical systems. Therefore, discussion is revised to remove references to safe shutdown. Electrical systems support is limited to those systems supporting SFC and auxiliaries required during any normal facility conditions.
1.4.8	Radioactive Wastes and Radiation Protection	Modify	This section is modified to replace the word "plant" with the word "facility."
1.4.10	Fire Protection	Modify	The text of this section is modified to replace the word "Plant" with the word "Facility." This section is modified to reflect that the licensing basis for fire protection changed to 10 CFR 50.48(f) after the certifications required by 10 CFR 50.82(a)(1) were docketed in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed License Condition 2.C.(3) of Facility License DPR-20 for PNP regarding the Fire Protection Program. This license condition is deleted to reflect the permanently defueled condition of the facility. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, the fire protection program will be revised to take into account the decommissioning facility conditions and activities. PNP will continue to utilize the defense-in-depth concept, placing special emphasis on detection and suppression in order to minimize the potential for radiological releases to the environment.

Section	Title	Change	Description of Change
1.4.11	Circulating Water System	Delete	This section is proposed to be deleted in its entirety. The circulating water system provided the condensers with a continuous supply of cooling water, for removing the heat rejected by the turbine generator, and the ability to inject sodium hypochlorite. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, nuclear steam cannot be produced, electrical power cannot be generated, and there are no pertinent circulating water design criteria or design functions.
1.4.12	Security	Modify	This section is modified to replace the word "plant" with the word "facility."
1.4.13	Emergency Planning	Modify	This section is modified to replace the word "plant" with the word "facility."
1.4.14	Plant Operation	Modify	This section is modified to replace the word "plant" with the word "facility." This section is modified to reflect the revised TS.
1.4.15	Structures	Modify	This section is modified to replace the word "plant" with the word "facility."

Section	Title	Change	Description of Change
1.5	Major Plant Modifications (Design/ Construction)	Modify	This section is modified to replace the word "plant" with the word "facility" and delete certain descriptions with no current safety significance: Condenser Retubing, Condenser Replacement, Steam Generator Tube Plugging – Old Generators, Operational information for Replacement Steam Generators, Auxiliary Feedwater modification – Bechtel/Bechtel, and High Head Auxiliary Feedwater Pump Installation and cross connection of Storage Tanks. This section is further modified by eliminating some discussion of operating parameters. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Historical information is retained.
1.7	Research and Development Requirements	Delete	This section (and subsections) is proposed to be deleted in its entirety. Amendment 272 removed the SSCs discussed in this section, and the TSs no longer contain requirements for these SSCs. Therefore, test and analysis of these SSCs can be removed from the UFSAR.
1.7.1	Flow Mixing and Flow Distribution	Delete	See 1.7 above
1.7.2	Control Rod Tests	Delete	See 1.7 above
1.7.3	Control Rod Drive Mechanisms	Delete	See 1.7 above
1.7.4	Fuel Bundle Design	Delete	See 1.7 above

Section	Title	Change	Description of Change
1.7.5	Reactor Vessel Flow Tests	Delete	See 1.7 above
1.8.1.3	Continuing Applicability and Interpretation of SEP Information	Modify	This section is modified to replace the word "plant" with the word "facility."
1.8.2	TMI Action Items (NUREG-0737)	Delete	Section is deleted in its entirety NUREG 737 requirements only apply to operating reactors. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2).
1.8.4	Seismic Analysis for As-Built Safety-Related Piping Systems (ie Bulletin 79-14)	Modify	This section is modified to replace the word "plant" with the word "facility."

Section	Title	Change	Description of Change
1.8.5	Unresolved Safety Issues (NUREG-0410)	Modify	This section is modified to replace the word "plant" with the word "facility" and delete discussion of the SIRWT. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids which do not rely on the SIRWT for mitigation.
1.8.6	Environmental Qualification of "Safety-Related" Electrical Equipment (EEQ)	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and(ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids which do not create any of the conditions related to environmental qualification.

Section	Title	Change	Description of Change
1.8.7	Control Room Habitability (NUREG-0696)	Delete	Amendment 272 removed License Condition 2.C.(7), CONTROL ROOM ENVELOPE HABITABILITY, and Tech Spec Sections 3.7.10 and 3.7.11, control room ventilation filtration and cooling, and the facility license no longer contains requirements for the control room ventilation systems. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit control room cooling or filtration.
1.8.8	Effects of Pipe Rupture (SEP Topics iii.5.a and b)	Delete	This Section is deleted in its entirety. Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, pipe rupture, and its associated dynamic effects, is no longer a credible scenario.
1.8.10	Safe Shutdown	Delete	This Section is deleted in its entirety. This section is deleted to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. Therefore, discussion of safe shutdown is no longer relevant.

Section	Title	Change	Description of Change
1.9.1	Summary Descriptions of Aging Management Programs	Modify	The text of this section is modified to replace the word "Plant" with the word "Facility."
1.9.1.2	ASME Section XI, Subsections IWB, IWC, IWD, IWF Inservice Inspection Program	Modify	The text of this section is modified to replace the word "Plant" with the word "Facility."
1.9.1.8	Containment Leakage Testing Program	Delete	This section is proposed to be deleted in its entirety. Amendment 272 removed License Condition 2.D regarding Appendix J testing, and the facility license no longer requires a leakage testing program for containment. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur. Consequently, the Containment is no longer required to perform a function in the permanently shut down and defueled state. Thus, the Containment Leakage Testing Program no longer applies to a plant system, structure, or component that is within the 10 CFR 54.4 Scope for License Renewal and may be eliminated.

Section	Title	Change	Description of Change
1.9.1.16	Reactor Vessel Integrity Surveillance Program	Delete	This section is deleted in its entirety. The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, requirements for the surveillance program are no longer applicable.
1.9.1.17	Reactor Vessel Internals Inspection Program	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, the reactor vessel internal inspection program is no longer relevant.

Section	Title	Change	Description of Change
1.9.1.18	Steam Generator Tube Integrity Program	Delete	This section is proposed to be deleted in its entirety. Amendment 272 removed TS 5.5.8, "Steam Generator (SG) Program," which ensures that SG tube integrity is maintained, and the license will no longer contain requirements for tube integrity. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit SG tube integrity. Consequently, the Steam Generator Tube Integrity Program no longer applies to a plant system, structure, or component that is within the 10 CFR 54.4 Scope for License Renewal and may be eliminated.
1.9.1.19	Structural Monitoring Program	Modify	The text of this section is modified to replace the word "plant" with the word "facility."
1.9.1.23	Compressed air monitoring program	Modify	The text of this section is modified to replace the word "plant" with the word "facility."
1.9.2.1	Reactor Vessel Neutron Embrittlement	Delete	This section is deleted in its entirety. The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, vessel embrittlement and requirements for the surveillance program are no longer applicable.

Section	Title	Change	Description of Change
1.9.2.2	Metal Fatigue	Modify	This section is modified to remove discussion of specific metal fatigue analyses. Discussion of the Fatigue Monitoring Program is retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and components will not incur any additional fatigue cycles. After implementation of Amendment 272, calculation of usage factors for metal SSCs is no longer relevant.
1.9.2.3	Environmental Qualification of Electrical Equipment	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not create any of the conditions related to environmental qualification.

Section	Title	Change	Description of Change
1.9.2.4	Containment Liner Plate, Metal Containments and Penetrations Fatigue Analysis	Modify	This section is modified by eliminating discussion of the design functions and capability of the containment and associated SSCs to withstand internal pressure associated with a loss of coolant accident. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. In addition, power operations and electrical generation can no longer occur, and components will not incur any additional fatigue cycles. Therefore, containment tendons and containment liner plate and penetration fatigue are no longer relevant.
1.9.2.5	Other Plant-Specific Time-Limited Aging Analyses	Modify	This section is modified to remove discussion of Alloy 600, primary coolant pump flywheel, and reactor vessel underclad. Discussion of crane TLAA is retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. TLAA of Alloy 600, primary coolant pump flywheel, and or reactor vessel underclad have no bearing on the remaining accidents.

Section	Title	Change	Description of Change
1.0	References	Modify	Deleted Reference 17 EQ and 10 CFR 50.49 only apply to operating reactors
Table 1-1	Chronological Licensing Events	Modify	Additions are proposed to the licensing events to add Amendment 272 Amendment and defueled certification to the NRC.
Table 1-2	Palisades Design Characteristics (Nominal Values)	Modify	The information in this table regarding operational parameters for the reactor and steam cycles is proposed for deletion. Equipment descriptions and operational parameters for equipment required to support SFP cooling and the defueled condition is retained. This table is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Operational parameters on equipment required for power operations, electrical generation, and core related design basis accidents is deleted.
Table 1-3	Systematic Evaluation Program (SEP) and Integrated Assessment Program (IAP), Palisades Plant - NUREG-0820	Modify	Table is modified to remove deleted references from Section 5.1. Numerous GDC no longer apply rendering the associated SEP item irrelevant.

Section	Title	Change	Description of Change
Table 1-4	Post-TMI Requirements for Consumers Power Company's, Palisades Plant - NUREG-0737	Delete	Table is deleted in its entirety. NUREG-0737 only applies to operating power reactors.
Table 1-9	Applicable to Palisades 1-9 Long-Term Commitments for License Renewal of Palisades Nuclear Plant	Modify	The following commitments are deleted 1 - Actions required by this commitment were completed and no longer require retention to support a permanently defueled facility 2, 3, 29, 30, 31, 32 and 33 - The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, requirements for the surveillance program are no longer applicable. 4 &5 - Plant system operation that results in concerns over charging line and pressurizer temperature elements fatigue will no longer occur since the RFOL does not authorize reactor operation 8 - Actions required by this commitment were completed and no longer require retention to support a permanently defueled facility

Section	Title	Change	Description of Change
2.5.5.1	X/Q Determination for TID Analyses	Modify	This section is revised to delete X/Q values greater than the 0-2 time period. This section is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. The analyses for these events assume an immediate release and only utilize the 0-2 hour X/Q values.
2.5.5.2	X/Q Determination for AST Analyses	Modify	This section is revised to delete X/Q values greater than the 0-2 time period. This section is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. The analyses for these events assume an immediate release and only utilize the 0-2 hour X/Q values.

Section	Title	Change	Description of Change
2.5.5.2.1	Offsite X/Q Determination	Modify	This section is revised to delete X/Q values greater than the 0-2 time period. This section is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. The analyses for these events assume an immediate release and only utilize the 0-2 hour X/Q values.
2.5.5.2.2	Control Room X/Q Determination	Modify	This section is revised to change a Table reference in Chapter 14.
Table 2-17	Relative Dispersion (X/Q) Values (s/m^3) Versus Averaging Time, Turbine Building Vent - Ground Level Release Exclusion Area Boundary	Modify	This table is revised to delete X/Q values greater than the 0-2 time period. This table is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. The analyses for these events assume an immediate release and only utilize the 0-2 hour X/Q values.

Section	Title	Change	Description of Change
Table 2-18	Relative Dispersion (X/Q) Values (s/m^3) Versus Averaging Time, Turbine Building Vent - Ground Level Release Low Population Zone	Modify	This table is revised to delete X/Q values greater than the 0-2 time period. This table is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. The analyses for these events assume an immediate release and only utilize the 0-2 hour X/Q values.
Table 2-21	X/Q Values (s/m^3) – Ground Level Release – Exclusion Area Boundary	Modify	This table is revised to delete X/Q values greater than the 0-2 time period. This table is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. The analyses for these events assume an immediate release and only utilize the 0-2 hour X/Q values.

Section	Title	Change	Description of Change
Table 2-22	X/Q Values (s/m^3) – Ground Level Release – Low Population Zone	Modify	This table is revised to delete X/Q values greater than the 0-2 time period. This table is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. The analyses for these events assume an immediate release and only utilize the 0-2 hour X/Q values.
3.1	Introduction	Modify	This section is modified by eliminating discussions of reactor design, operation and reactivity control. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, this section is modified by eliminating the discussion of the reactor operation and control. The reactor vessel will never be loaded with fuel again.

Section	Title	Change	Description of Change
3.2.1	Performance Objectives	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, there are no performance objectives for the reactor.
3.2.2	Design Objective	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, there are no design objectives for the reactor.

Section	Title	Change	Description of Change
3.2.3	Design Limits	Modify	This section is modified by eliminating discussions of reactor vessel design limits. Title is revised to "FUEL AND CONTROL ROD DESIGN LIMITS." Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, this section is modified by eliminating the discussion of the reactor vessel and the control rod drive mechanism design limits. The reactor vessel will never be loaded with fuel again. In addition, the control rod drive mechanisms perform no function in the defueled state. The information regarding the fuel bundles will be retained, because they will continue to be stored in the Spent Fuel Pool (SFP) or the Independent Spent Fuel Storage Installation (ISFSI) until permanent removal from the site.

Section	Title	Change	Description of Change
3.3.1	General Summary	Modify	This section is modified by eliminating discussions of reactor power distribution and control mechanisms. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, this section is modified by eliminating the discussion of the reactor operation. The information regarding the fuel assemblies and control rods will be retained, because they will continue to be stored in the Spent Fuel Pool (SFP) or the Independent Spent Fuel Storage Installation (ISFSI) until permanent removal from the site.

Section	Title	Change	Description of Change
3.3.2	Nuclear Design and Evaluation	Delete	This section is deleted in its entirety along with the associated subsections Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). This section discusses reactor reactivity, reactivity control, reactor stability and neutron fluence. The reactor vessel will never be loaded with fuel again and this information is obsolete. Additional information on ASME stress limits and seismic analysis is also deleted.
3.3.2.1	Reactivity and Control Requirement	Delete	See 3.3.2 above
3.3.2.2	Reactivity Coefficients	Delete	See 3.3.2 above
3.3.2.3	Control Blade Worths	Delete	See 3.3.2 above
3.3.2.4	Reactivity Insertion Rates	Delete	See 3.3.2 above
3.3.2.5	Power Distribution and Power Escalation Rates	Delete	See 3.3.2 above

Section	Title	Change	Description of Change
3.3.2.6	Neutron Fluence on Pressure Vessel	Delete	See 3.3.2 above
3.3.2.7	Nuclear Evaluation	Delete	See 3.3.2 above
3.3.2.8	Reactor Stability	Delete	See 3.3.2 above
3.3.3	Thermal-Hydraulic Design and Evaluation	Delete	This section is deleted in its entirety along with the associated subsections. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). This section discusses reactor reactivity, reactivity control, reactor stability and neutron fluence. The reactor vessel will never be loaded with fuel again and this information is obsolete. Additional information on ASME stress limits and seismic analysis is also deleted.

Section	Title	Change	Description of Change
3.3.3.1	Thermal-Hydraulic Design Criteria	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Since the reactor vessel will never be loaded with fuel again, thermal-hydraulic criteria are obsolete.
3.3.3.2	Plant Parameter Variations	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Since the reactor vessel will never be loaded with fuel again, parameter variations are not relevant.

Section	Title	Change	Description of Change
3.3.3.3	Core Flow Distribution	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The reactor vessel will never be loaded with fuel again and therefore core flow information is obsolete.
3.3.3.4	Trip Set Points	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.3, Instrumentation, including all required instrumentation setpoints, rendering discussion regarding trip setpoints obsolete. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2).

Section	Title	Change	Description of Change
3.3.4	Mechanical Design and Evaluation	Modify	This section of the IP2 UFSAR provides information regarding the mechanical design limits for the reactor internals and core components. It will be modified to eliminate the discussions regarding the reactor internals and reactor operations. The title of the section is changed from "Mechanical Design and Evaluation" to "Mechanical Design." Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. As a result, the discussions regarding the reactor internals (with the exception of the fuel rods, fuel assemblies, and rod cluster control assemblies) and reactor operations are obsolete. The discussions regarding the fuel pellets, fuel rods, fuel assemblies, and rod cluster control assemblies will be retained, but are modified to eliminate the information regarding nuclear fuel operation or emplacement in the reactor vessel and retain the information regarding fuel design that is applicable to storage in the SFP or the ISFSI.
3.3.4.1	Reactor Internals	Modify	This section is modified to delete design and operational aspects of reactor internals. Physical descriptions of internal components is retained. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. As a result, the discussions regarding the reactor internals design requirements and reactor operations are obsolete.

Section	Title	Change	Description of Change
3.3.4.2	Control Rod Drive Mechanism	Modify	This section is modified to delete design and operational aspects of control rod drive mechanisms (CRDM). Physical descriptions of the CRDMs is retained. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The reactor vessel will never be loaded with fuel again and therefore CRDM design information and operational aspects are obsolete.
3.3.4.3	Core Mechanical Design	Modify	This section is modified to delete core design information and operational aspects of fuel bundles, control rods and sources. Design information and physical descriptions for fuel bundles, fuel rods, control rods and neutron sources is retained. Amendment 272 deleted TS Sections 2.0, 3.1, 3.2 and 4.2, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The reactor vessel will never be loaded with fuel again and therefore core design requirements and operational aspects are obsolete. The information regarding the fuel rods, fuel assemblies, control rods and neutron sources will be retained, because they will continue to be stored in the Spent Fuel Pool (SFP) or the Independent Spent Fuel Storage Installation (ISFSI) until permanent removal from the site.
3.0	References	Delete	This section is deleted in its entirety. There are no remaining references in Section 3.0

Section	Title	Change	Description of Change
Table 3-1	Primary Stress Limits for Critical Reactor Vessel Internal Structures	Delete	This table is deleted in its entirety. Discussion of reactor vessel stresses is deleted from 3.2.2.
Figure 3-2	Position of Fuel Assemblies and Control Rod Groups in the Palisades Core	Delete	This figure is deleted in its entirety. Amendment 272 reflects the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The reactor vessel will never be loaded with fuel again and therefore position information for fuel assemblies and control rods is obsolete.
Figure 3-3	Control Rod Insertion Limits	Delete	This figure is deleted in its entirety. Amendment 272 reflects the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The reactor vessel will never be loaded with fuel again and therefore insertion limits are rendered obsolete.

Section	Title	Change	Description of Change
Figure 3-4	Reactivity Difference Between Fundamental and Excited States of a Bare Cylindrical Reactor Cylindrical Reactor	Delete	This figure is deleted in its entirety. Amendment 272 reflects the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The reactor vessel will never be loaded with fuel again and therefore information on reactivity, reactor control and flux distribution is rendered obsolete.
Figure 3-5	Thermal Neutron Flux at the Center of the Core vs Time	Delete	This figure is deleted in its entirety. Amendment 272 reflects the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The reactor vessel will never be loaded with fuel again and therefore information on reactivity, reactor control and flux distribution is rendered obsolete.

Section	Title	Change	Description of Change
Figure 3-6	Damping Coefficient vs Reactivity Difference Between Fundamental and Excited State	Delete	This figure is deleted in its entirety. Amendment 272 reflects the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The reactor vessel will never be loaded with fuel again and therefore information on reactivity, reactor control and flux distribution is rendered obsolete.
Figure 3-7	End of Life Axial Oscillations With Doppler Feedback, Full Power (2-Hour Time Steps)	Delete	This figure is deleted in its entirety. Amendment 272 reflects the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The reactor vessel will never be loaded with fuel again and therefore information on reactivity, reactor control and flux distribution is rendered obsolete.

Section	Title	Change	Description of Change
Figure 3-8	Split Detector Response to Axial Power Profiles in the Core	Delete	This figure is deleted in its entirety. Amendment 272 reflects the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The reactor vessel will never be loaded with fuel again and therefore information on reactivity, reactor control and flux distribution is rendered obsolete.
4.1	Introduction	Modify	This section is modified by eliminating discussions of design and safety functions of the primary coolant system. Physical descriptions of SSCs are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, this section is modified by eliminating the discussion of Primary Coolant System operation and design requirements.
4.2	Design Basis	Modify	Reference to Tables 4-1 and 4-2 are relocated to this section.

Section	Title	Change	Description of Change
4.2.1	Performance Objectives and Parameters for Normal Conditions	Modify	This section is modified to remove performance objectives and operating parameters. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, the performance objectives and parameters for normal operation of the primary coolant system are no longer relevant. Table 4-1 is retained in Section 4.2 to provide initial design parameters.
4.2.2	Design Cyclic Loads	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, the design cyclic loads of the primary coolant system are no longer relevant. Table 4-2 is retained in Section 4.2 to provide Code requirements.

Section	Title	Change	Description of Change
4.2.3	Design Service Life Considerations	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, the design service life considerations of the primary coolant system are no longer relevant.
4.2.5	Safety Considerations of Design Parameters	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, the performance objectives and parameters for normal operation of the primary coolant system are no longer relevant.

Section	Title	Change	Description of Change
4.2.6	Primary Coolant System Asymmetric Loads	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, the performance objectives and parameters for normal operation of the primary coolant system are no longer relevant.
4.3	System Design and Operation	Modify	Section header is revised to System Design
4.3.1	General Description	Modify	This section is modified by eliminating discussions of primary coolant system operation and safety and design functions. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, safety and design functions, along with operational aspects, of the primary coolant system are no longer relevant.

Section	Title	Change	Description of Change
4.3.2	Interfaces with Other Systems	Modify	This section is modified by eliminating functional descriptions of interfacing systems. Physical descriptions of SSCs are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, design functions of systems connected to the primary coolant system are no longer relevant.
4.3.3	Reactor Vessel	Modify	This section is modified by eliminating discussions of reactor vessel design requirements. Physical descriptions of vessel components are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, design functions of the reactor vessel are no longer relevant.

Section	Title	Change	Description of Change
4.3.4	Steam Generator	Modify	This section is modified by eliminating discussions of SG operational and design requirements. Physical descriptions of SG components are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, and TS 5.5.8, Steam Generator Program, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, the SG will no longer perform a function in the permanently shut down and defueled facility and, therefore, design requirements of the SGs are no longer relevant.
4.3.4.1	Steam Generator Tube Degradation	Delete	This section deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, and TS 5.5.8, Steam Generator Program, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, the SG will no longer perform a function in the permanently shut down and defueled facility. Therefore, SG tube degradation is no longer relevant.

Section	Title	Change	Description of Change
4.3.5	Primary Coolant Pumps	Modify	This section is modified by eliminating discussions of pump operational and design requirements. Physical descriptions of RCP components are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, the primary coolant pumps will no longer be required to operate, and pump design functions are no longer relevant.
4.3.6	Primary Coolant Piping	Modify	This section is modified by eliminating discussions of primary coolant piping design requirements and operational issues. Physical descriptions of piping are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, the primary coolant piping will no longer be utilized, and piping design requirements and operational issues are no longer relevant.

Section	Title	Change	Description of Change
4.3.7	Pressurizer	Modify	This section is modified by eliminating discussions of pressurizer safety functions, design requirements and operational issues. Physical descriptions of SSCs are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.9, Pressurizer, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, pressurizer safety functions and design requirements are no longer relevant.
4.3.8	Quench Tank	Modify	This section is modified by eliminating discussions of pressurizer quench tank operation and design requirements. Physical descriptions of SSCs are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.9, Pressurizer, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, pressurizer quench tank operation and design requirements are no longer relevant.

Section	Title	Change	Description of Change
4.3.9.1	General Criteria	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, primary coolant valves no longer have safety or design functions.
4.3.9.2	Pressurizer Throttling (spray) Control Valves	Delete	This section deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.9, Pressurizer, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, pressurizer throttling control valve functional and design requirements are no longer relevant.

Section	Title	Change	Description of Change
4.3.9.3	Power-Operated Relief Valves (PORV) and Block Valves	Modify	This section is modified by eliminating discussions of PORV functional and design requirements. Physical descriptions of SSCs are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.9, Pressurizer, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, PORV functional and design requirements are no longer relevant.
4.3.9.4	Spring-Actuated Primary Safety Valves	Modify	This section is modified by eliminating discussions of Pressurizer Safety Valves functional and design requirements. Physical descriptions of SSCs are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.10, Pressurizer Safety Valves, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, Pressurizer Safety Valves functional and design requirements are no longer relevant.

Section	Title	Change	Description of Change
4.3.10	Environmental protection	Modify	This section is modified by eliminating discussions of missile protection and accident loading. Seismic characteristics of the NSSS are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, missile protection and accident loading in a seismic event are no longer relevant.
4.3.11	Materials exposed to coolant	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, SSC exposure to coolant is no longer relevant.

Section	Title	Change	Description of Change
4.3.12	Insulation	Modify	This section is modified by eliminating discussions of insulation performance. Physical characteristics of the insulation are retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, functional requirements of insulation is no longer relevant.
4.4	System design evaluation	Delete	This section header is deleted as it has no content.
4.4.1	Design margin	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. As a result, the design margin of the primary coolant system is longer relevant.

Section	Title	Change	Description of Change
4.4.2	Prevention of brittle fracture	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. As a result, primary coolant system pressure temperature limits and brittle fracture prevention is longer relevant.
4.5	Tests and Inspections	modify	Section Header is revised to "INSPECTIONS." This is an editorial change to reflect revised section content.
4.5.2	Nil ductility transition temperature determination	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Consequently, nil ductility of the primary coolant system is longer relevant.

Section	Title	Change	Description of Change
4.5.3	Surveillance program	Delete	This section is deleted in its entirety. The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, requirements for the surveillance program are no longer applicable.
4.5.6	Inservice inspection	Modify	This section is modified to remove reference to the SG tube inspection program and the vessel surveillance program which were removed in Amendment 272. See UFSAR Sections 4.5.3 and 4.3.4.1.
4.5.7	NDTT of other primary system components	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Consequently, nil ductility of primary coolant system SSCs is no longer relevant.

Section	Title	Change	Description of Change
4.6	Operating limitation	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, primary coolant system operating limits are no longer relevant.
4.7	Primary coolant pressure boundary leakage detection	Delete	This header is deleted in its entirety as there are no remaining subsections. Amendment 272 deleted TS Sections 3.4.14 PCS Operational LEAKAGE, 3.4.14 PCS Pressure Isolation Valve (PIV) Leakage, and 3.4.15 PCS Leakage Detection Instrumentation. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, primary coolant system operating limits are no longer relevant.
4.7.1	Leak detection	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Sections 3.4.14 PCS Operational LEAKAGE, 3.4.14 PCS Pressure Isolation Valve (PIV) Leakage, and 3.4.15 PCS Leakage Detection Instrumentation. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, primary coolant system leakage is no longer relevant.

Section	Title	Change	Description of Change
4.7.2	Operator action following leak detection	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Sections 3.4.14 PCS Operational LEAKAGE, 3.4.14 PCS Pressure Isolation Valve (PIV) Leakage, and 3.4.15 PCS Leakage Detection Instrumentation. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, operator actions upon detection of primary coolant system leakage is no longer relevant.
4.8	Primary coolant gas vent system	Modify	This section is modified to remove operational aspects of the primary coolant gas vent system. Physical description and design information of SSCs is retained. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, primary coolant gas vent system operation is no longer relevant.

Section	Title	Change	Description of Change
Table 4-1	Primary coolant system parameters	Delete	This table is deleted to remove operating parameters. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, primary coolant system operating parameters are no longer relevant.
Table 4-2	Primary coolant system code requirements	Modify	Title is revised to Primary Coolant System Codes
Table 4-4	Steam generator parameters	Modify	This table is modified to remove operating parameters. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, SG operating parameters are no longer relevant.

Section	Title	Change	Description of Change
Table 4-5	Secondary Safety Valve Parameters	Modify	This table is modified to remove operating parameters. Amendment 272 deleted TS Section 3.7.1, Main Steam Safety Valves, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, safety valve operating parameters are no longer relevant.
Table 4-6	Primary Coolant Pump Parameters	Modify	This table is modified to remove operating parameters. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, pump operating parameters are no longer relevant.

Section	Title	Change	Description of Change
Table 4-8	Pressurizer Parameters	Modify	This table is modified to remove operating parameters. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.9, Pressurizer, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, pressurizer operating parameters are no longer relevant.
Table 4-9	Pressurizer Normal Level Control Program/Pressurizer Backup Level Control Program	Delete	This table is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.9, Pressurizer, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, pressurizer operating parameters are no longer relevant.

Section	Title	Change	Description of Change
Table 4-10	Pressurizer Safety Valve Parameters	Delete	This table is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.10, Pressurizer Safety Valves, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, Pressurizer Safety Valves functional parameters are no longer relevant.
Table 4-11	Quench Tank Parameters	Modify	This table is modified to remove operating parameters. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.9, Pressurizer, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, quench tank operating parameters are no longer relevant.

Section	Title	Change	Description of Change
Table 4-12	Actuator-Operated Throttling Valve Parameters	Delete	This table is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.9, Pressurizer, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, pressurizer throttling control valve operating parameters are no longer relevant.
Table 4-13	Actuator-Operated Block Valve Parameters	Delete	This table is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.9, Pressurizer, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, block valve operating parameters are no longer relevant.

Section	Title	Change	Description of Change
Table 4-14	Pressurizer Power-Operated Relief Valve Parameters	Modify	This table is modified to remove operating parameters. Amendment 272 deleted TS Section 3.4, Primary Coolant System, including TS 3.4.11 Pressurizer Power Operated Relief Valves (PORVs), reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, PORV operating parameters are no longer relevant.
Table 4-16	Primary Coolant Chemistry	Delete	This table is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, primary coolant chemistry parameters are no longer relevant.
Table 4-17	Summary of Specimens Provided for Each Exposure Location / Summary of Specimens Provided for Each Supplemental Surveillance Capsule	Delete	This table is deleted in its entirety. The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, requirements for the surveillance program are no longer applicable.

Section	Title	Change	Description of Change
Table 4-20	Reactor Vessel Surveillance Coupon Removal Schedule	Delete	This table is deleted in its entirety. The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, requirements for the surveillance program are no longer applicable.
Figure 4-5	Deleted	Delete	Previously Deleted
Figure 4-7	Primary Coolant Pump Characteristics Curve	Delete	This figure is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Consequently, operational parameters of the primary coolant system are longer relevant.
Figure 4-9	Temperature Control Program	Delete	This figure is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Consequently, operational parameters of the primary coolant system are longer relevant.

Section	Title	Change	Description of Change
Figure 4-10	Pressurizer Level Set Point vs TAVG	Delete	This figure is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Consequently, operational parameters of the primary coolant system are longer relevant.
Figure 4-11	Location of Surveillance Capsule Assemblies	Delete	This figure is deleted in its entirety. The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, requirements for the surveillance program are no longer applicable.
Figure 4-12	Typical Surveillance Capsule Assembly	Delete	This figure is deleted in its entirety. The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, requirements for the surveillance program are no longer applicable.
Figure 4-13	Typical Charpy Impact Compartment Assembly	Delete	This figure is deleted in its entirety. The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, requirements for the surveillance program are no longer applicable.

Section	Title	Change	Description of Change
Figure 4-14	Typical Tensile - Monitor Compartment Assembly	Delete	This figure is deleted in its entirety. The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, requirements for the surveillance program are no longer applicable.
Figure 4-15	Variable LTOP Setpoints	Delete	This figure is deleted in its entirety. Amendment 272 deleted TS Section 3.4, Primary Coolant System, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Consequently, initial inspection and qualification of components of the primary coolant system is longer relevant.
4.0	References	Delete	References deleted from text and tables were removed from Section. This resulted in deletion of entire reference section.

Section	Title	Change	Description of Change
5.1.1	Introduction	Modify	This section is modified to refer to the permanently defueled condition, replace the word “Plant” with the word “Facility,” change "FSAR" to "DSAR." Use of "Plant" is retained when used in historical reference. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. In the permanently defueled condition, certain aspects of the GDC are no longer applicable. After implementation of Amendment 272, the document will be referred to as Defueled Safety Analysis Report.
5.1.2.3	Criterion 3 - Fire Protection	Modify	This section is modified to replace the word “Plant” with the word “Facility,” and "safe reactor shutdown" is replaced by "safe and stable conditions." Use of "Plant" is retained when used in historical reference. This section is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. Therefore, discussion of safe shutdown is replaced with safe and stable conditions related to spent fuel storage.

Section	Title	Change	Description of Change
5.1.2.4	Criterion 4 - Environmental and Missile Design Bases	Modify	This section is modified to remove discussion of accident conditions and scenarios. SEP topics on pipe breaks and internal missiles are deleted. This section is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, discussion of accident conditions and pipe whip scenarios inside containment is deleted.
5.1.2.5	Criterion 5 - Sharing of Structures, Systems and Components	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.1.2.6	Conclusions	Modify	This section is modified to replace the word "Plant" with the word "Facility." The text is revised to reference "applicable" criteria as some aspects of Group 1 criteria are no longer applicable (See 5.1.2.3 and 5.1.2.4).

Section	Title	Change	Description of Change
5.1.3.1	Criterion 10 - Reactor Design	Modify	This section is modified to remove discussion of reactor core operations and transients. Reference to TS Section 2.1 is deleted, and information on fuel design is retained. Amendment 272 deleted TS Sections 2.0, Safety Limits, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. This section is revised to reflect that the certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore, discussion of operating parameters is deleted.
5.1.3.2	Criterion 11 - Reactor Inherent Protection	Modify	This section is modified to remove discussion of reactor core operations and note that Criterion 11 is not applicable in the defueled condition. This section is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore, discussion of reactor core nuclear coefficients is deleted.

Section	Title	Change	Description of Change
5.1.3.3	Criterion 12 - Suppression of Reactor Power Oscillations	Modify	This section is modified to remove discussion of reactor core operations and note that Criterion 12 is not applicable in the defueled condition. Amendment 272 deleted TS Sections 3.1, REACTIVITY CONTROL SYSTEMS, and 3.2, POWER DISTRIBUTION LIMITS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). As a result, reactor power oscillations are not possible and Criterion 13 no longer applies.
5.1.3.4	Criterion 13 - Instrumentation and Control	Modify	This section is modified to remove discussion of reactor core operations and note that Criterion 12 is not applicable in the defueled condition. Amendment 272 deleted TS Section 3.3, INSTRUMENTATION, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any instrumentation or operator action and, consequently, Criterion 13 no longer applies.

Section	Title	Change	Description of Change
5.1.3.6	Criterion 15 - Reactor Coolant System Design	Modify	This section is modified to remove discussion of accident conditions and scenarios. This section is revised to reflect that The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, discussion of normal reactor operation and transients is deleted along with reference to the Tech Specs.
5.1.3.7	Criterion 16 - Containment Design	Modify	This section is modified to remove discussion of containment leakage and Technical Specifications. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, discussion of containment leakage is deleted along with reference to the Tech Specs.

Section	Title	Change	Description of Change
5.1.3.8	Criterion 17 - Electrical Power Systems	Delete	This section is deleted in its entirety. Amendment 272 deleted TS Section 3.8, ELECTRICAL POWER SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any electrical power sources and, consequently, Criterion 18 no longer applies.
5.1.3.9	Criterion 18 - Inspection and Testing of Electrical Power Systems	Modify	This section is modified to remove discussion of facility electrical systems and note that Criterion 18 is not applicable in the defueled condition. Amendment 272 deleted TS Section 3.8, ELECTRICAL POWER SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any electrical power sources and, consequently, Criterion 18 no longer applies.

Section	Title	Change	Description of Change
5.1.3.10	Criterion 19 - Control Room	Modify	This section modified to remove discussion of control room habitability systems, hot shut down and cold shutdown, and is modified to replace the word "Plant" with the word "Facility." Amendment 272 deleted TS 3.7.10, Control Room Ventilation (CRV) Filtration, and TS 3.7.11, Control Room Ventilation (CRV) Cooling, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accident analyses do not credit any control room habitability systems while still maintaining CR doses within licensing basis requirements. Hot and cold shutdown are not applicable in the defueled condition and are deleted.
5.1.3.11	Conclusions	Modify	This section is modified to replace the word "Plant" with the word "Facility." The text is revised to reference "applicable" criteria as some aspects of Group II criteria are no longer applicable (See 5.1.3.1, 5.3.1.2, 5.3.1.6 and 5.3.1.7, and 5.3.1.10).

Section	Title	Change	Description of Change
5.1.4.1	Criterion 20 - Protection System Functions	Modify	This section is modified to remove discussion of the RPS and note that Criterion 20 is not applicable in the defueled condition. Amendment 272 deleted TS Section 3.3, INSTRUMENTATION, including 3.3.1 Reactor Protective System (RPS) Instrumentation, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any protective systems and, consequently, RPS is no longer required.
5.1.4.2	Criterion 21 - Protection System Reliability and Testability	Modify	This section is modified to remove discussion of protection systems and note that Criterion 21 is not applicable in the defueled condition. Amendment 272 deleted TS Section 3.3, INSTRUMENTATION, including 3.3.1 Reactor Protective System (RPS) Instrumentation, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any protection systems and, consequently, reliability and testability is no longer applicable.

Section	Title	Change	Description of Change
5.1.4.3	Criterion 22 - Protection System Independence	Modify	This section is modified to remove discussion of protection systems and note that Criterion 22 is not applicable in the defueled condition. Amendment 272 deleted TS Section 3.3, INSTRUMENTATION, including 3.3.1 Reactor Protective System (RPS) Instrumentation, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any protection systems and, consequently, independence is no longer applicable.
5.1.4.4	Criterion 23 - Protection System Failure Modes	Modify	This section is modified to remove discussion of protection system failure modes and note that Criterion 23 is not applicable in the defueled condition. Amendment 272 deleted TS Section 3.3, INSTRUMENTATION, including 3.3.1 Reactor Protective System (RPS) Instrumentation, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any protection systems and, consequently, independence is no longer applicable.

Section	Title	Change	Description of Change
5.1.4.5	Criterion 24 - Separation of Protection and Control System	Modify	This section is modified to remove discussion of protection systems separation and note that Criterion 24 is not applicable in the defueled condition. Amendment 272 deleted TS Section 3.3, INSTRUMENTATION, including 3.3.1 Reactor Protective System (RPS) Instrumentation, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any protection systems and, consequently, separation is no longer applicable.
5.1.4.6	Criterion 25 - Protection System Requirements for Reactivity Control Malfunctions	Modify	This section is modified to remove discussion of reactivity control system malfunctions and note that Criterion 25 is not applicable in the defueled condition. Amendment 272 deleted TS SECTION 3.1, REACTIVITY CONTROL SYSTEMS, and TS Section 3.3, INSTRUMENTATION, including 3.3.1 Reactor Protective System (RPS) Instrumentation, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any protection systems and, consequently, system malfunctions are no longer applicable.

Section	Title	Change	Description of Change
5.1.4.7	Criterion 26 - Reactivity Control System Redundancy and Capability	Modify	This section is modified to remove discussion of reactivity control systems and note that Criterion 26 is not applicable in the defueled condition. Amendment 272 deleted TS SECTION 3.1, REACTIVITY CONTROL SYSTEMS, and TS Section 3.3, INSTRUMENTATION, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, reactivity control is no longer applicable
5.1.4.8	Criterion 27 - Combined Reactivity Control Systems Capability	Modify	This section is modified to remove discussion of protection systems and note that Criterion 27 is not applicable in the defueled condition. Amendment 272 deleted TS SECTION 3.1, REACTIVITY CONTROL SYSTEMS, and TS Section 3.3, INSTRUMENTATION, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, reactivity control is no longer applicable

Section	Title	Change	Description of Change
5.1.4.9	Criterion 28 - Reactivity Limits	Modify	This section is modified to remove discussion of reactivity control systems and note that Criterion 28 is not applicable in the defueled condition. Amendment 272 deleted TS SECTION 3.1, REACTIVITY CONTROL SYSTEMS, and TS Section 3.3, INSTRUMENTATION, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, reactivity control and reactivity limits are no longer applicable
5.1.4.10	Criterion 29 - Protection Against Anticipated Operational Occurrences	Modify	This section is modified to remove discussion of anticipated operational occurrences and note that Criterion 29 is not applicable in the defueled condition. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, there are no anticipated operational occurrences related to protection and reactivity control systems.
5.1.4.11	Conclusions	Modify	This section is modified to replace the word "Plant" with the word "Facility." The text is revised to reference "applicable" criteria as some aspects of Group III criteria are no longer applicable.

Section	Title	Change	Description of Change
5.1.5.1	Criterion 30 - Quality of Reactor Coolant Pressure Boundary	Modify	This section is modified to remove discussion of RCS leakage and associated references. Amendment 272 deleted 3.4.13, PCS Operational LEAKAGE, and TS , 3.4.15 PCS Leakage Detection Instrumentation, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). RCS leakage limits are no longer applicable
5.1.5.2	Criterion 31 - Fracture Prevention of Reactor Coolant Pressure Boundary	Modify	This section is modified to remove discussion of fracture prevention and note that Criterion 31 is not applicable in the defueled condition. The requirements in 10 CFR 50 Appendix H are only applicable to nuclear plants that are performing power operations in the reactor critical operating mode. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). In addition, Amendment 272 deleted License Condition 2.J on the surveillance capsule program. Consequently, requirements for fracture prevention are no longer applicable.

Section	Title	Change	Description of Change
5.1.5.3	Criterion 32 - Inspection of Reactor Coolant Pressure Boundary	Modify	This section is modified to delete reference to unit shutdown and the RPV material surveillance program (see 5.1.5.2), remove reference to TS Section 3.4 and SEP V-5, and is modified to replace the word "Plant" with the word "Facility." Amendment 272 deleted TS SECTION 3.4, PRIMARY COOLANT SYSTEM, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). SEP V-5 concerns PCS leakage detection and is also deleted (See 5.1.5.1).
5.1.5.4	Criterion 33 - Reactor Coolant Makeup	Modify	This section is modified to remove discussion of reactor coolant makeup and note that Criterion 33 is not applicable in the defueled condition. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, there are no anticipated operational occurrences requiring reactor coolant makeup.
5.1.5.5	Criterion 34 - Residual Heat Removal	Modify	This section is modified to remove reference to shutdown cooling and note that Criterion 34 is not applicable in the defueled condition. Amendment 272 deleted TS SECTION 3.4, PRIMARY COOLANT SYSTEM, and 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, shutdown cooling is no longer applicable.

Section	Title	Change	Description of Change
5.1.5.6	Criterion 35 - Emergency Core Cooling	Modify	This section is modified to remove reference to ECCS and note that Criterion 35 is not applicable in the defueled condition. Amendment 272 deleted TS Section 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on ECCS for mitigation. Therefore, ECCS is no longer applicable.
5.1.5.7	Criterion 36 - Inspection of Emergency Core Cooling System	Modify	This section is modified to remove reference to ECCS and note that Criterion 36 is not applicable in the defueled condition. Amendment 272 deleted TS Section 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on ECCS for mitigation. Therefore, ECCS inspection is no longer applicable.

Section	Title	Change	Description of Change
5.1.5.8	Criterion 37 - Testing of Emergency Core Cooling System	Modify	This section is modified to remove reference to ECCS testing and note that Criterion 37 is not applicable in the defueled condition. Amendment 272 deleted TS Section 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on ECCS for mitigation. Therefore, ECCS requirements are no longer applicable.
5.1.5.9	Criterion 38 - containment heat removal	Modify	This section is modified to remove reference to containment cooling and note that Criterion 38 is not applicable in the defueled condition. Amendment 272 deleted TS 3.6.6, Containment Cooling Systems, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on containment cooling for mitigation. Therefore, containment cooling requirements are no longer applicable.

Section	Title	Change	Description of Change
5.1.5.10	Criterion 39 - Inspection of Containment Heat Removal System	Modify	This section is modified to remove reference to containment cooling inspection and note that Criterion 39 is not applicable in the defueled condition. Amendment 272 deleted TS 3.6.6, Containment Cooling Systems, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on containment cooling for mitigation. Therefore, containment cooling requirements are no longer applicable.
5.1.5.11	Criterion 40 - Testing of Containment Heat Removal System	Modify	This section is modified to remove reference to testing of containment cooling SSCs and note that Criterion 40 is not applicable in the defueled condition. Amendment 272 deleted TS 3.6.6, Containment Cooling Systems, and associated testing requirements, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on containment cooling for mitigation. Therefore, containment cooling testing requirements are no longer applicable.

Section	Title	Change	Description of Change
5.1.5.12	Criterion 41 - Containment Atmosphere Cleanup	Modify	This section is modified to remove reference to testing of containment atmosphere cleanup and note that Criterion 41 is not applicable in the defueled condition. Amendment 272 deleted TS 3.5.5, Containment Sump Buffering Agent and Weight Requirements, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on containment cleanup for mitigation. Therefore, containment atmosphere cleanup requirements are no longer applicable.
5.1.5.13	Criterion 42 - Inspection of Containment Atmosphere Cleanup Systems	Modify	This section is modified to remove reference to inspection of sodium tetraborate baskets and note that Criterion 42 is not applicable in the defueled condition. Amendment 272 deleted TS 3.5.5, Containment Sump Buffering Agent and Weight Requirements, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on containment cleanup for mitigation. Therefore, inspection of the sodium tetraborate baskets is no longer applicable.

Section	Title	Change	Description of Change
5.1.5.14	Criterion 43 - Testing of Containment Atmosphere Cleanup Systems	Modify	This section is modified to remove discussion of testing of sodium tetraborate baskets and note that Criterion 43 is not applicable in the defueled condition. Amendment 272 deleted TS 3.5.5, Containment Sump Buffering Agent and Weight Requirements, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on containment cleanup for mitigation. Therefore, discussion of testing of the sodium tetraborate baskets is no longer applicable.
5.1.5.15	Criterion 44 - Cooling Water	Modify	This section is modified to remove reference to cooling water systems and note that Criterion 44 is not applicable in the defueled condition. Amendment 272 deleted TS 3.7.7, Component Cooling Water (CCW) System, TS 3.7.8, Service Water System (SWS), and 3.7.9, Ultimate Heat Sink (UHS), reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on cooling water systems for mitigation. Therefore, cooling water systems are no longer relevant.

Section	Title	Change	Description of Change
5.1.5.16	Criterion 45 - Inspection of Cooling Water System and Criterion 46 - Testing of Cooling Water System	Modify	This section is modified to remove reference to inspection and testing of cooling water systems and note that Criterion 45 and 46 are not applicable in the defueled condition. Amendment 272 deleted TS 3.7.7, Component Cooling Water (CCW) System, TS 3.7.8, Service Water System (SWS), and 3.7.9, Ultimate Heat Sink (UHS), reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on cooling water systems for mitigation. Therefore, cooling water systems are no longer relevant.
5.1.5.17	Conclusions	Modify	This section is modified to replace the word "Plant" with the word "Facility." The text is revised to reference "applicable" criteria as some aspects of Group IV criteria are no longer applicable.
5.1.6.1	Criterion 50 - Containment Design Basis	Modify	This section is modified to delete containment design basis information and note that Criterion 50 is not applicable in the defueled condition. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, containment design basis is no longer pertinent.

Section	Title	Change	Description of Change
5.1.6.2	Criterion 51 - Fracture Prevention of Containment Pressure Boundary	Modify	This section is modified to delete fracture prevention information and note that Criterion 51 is not applicable in the defueled condition. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, containment fracture prevention is no longer pertinent.
5.1.6.3	Criterion 52 - Capability for Containment Leakage Rate Testing	Modify	This section is modified to delete containment testing information and note that Criterion 52 is not applicable in the defueled condition. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, containment testing is no longer pertinent.
5.1.6.4	Criterion 53 - Provisions for Containment Testing and Inspection	Modify	This section is modified to delete containment inspection and testing information and note that Criterion 53 is not applicable in the defueled condition. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, containment inspection and testing is no longer pertinent.

Section	Title	Change	Description of Change
5.1.6.5	Criterion 54 - Piping Systems Penetrating Containment	Modify	This section is modified to delete containment penetration design and leakage testing information and note that Criterion 54 is not applicable in the defueled condition. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, containment penetration design and leakage testing is no longer pertinent.
5.1.6.6	Criterion 55 - Primary Coolant Pressure Boundary Penetrating Containment	Modify	This section is modified to delete PCS pressure boundary design information and note that Criterion 55 is not applicable in the defueled condition. Amendment 272 removed Tech Spec 3.4.14 PCS Pressure Isolation Valve (PIV) Leakage," and the facility license no longer contains requirements for the PCS PIV leakage. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, PCS pressure boundary information is no longer pertinent.
5.1.6.7	Criterion 56 - Primary Containment Isolation	Modify	This section is modified to delete containment penetration isolation information and note that Criterion 56 is not applicable in the defueled condition. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, containment penetration isolation is no longer pertinent.

Section	Title	Change	Description of Change
5.1.6.8	Criterion 57 - Closed System Isolation Valves	Modify	This section is modified to delete containment penetration isolation information and note that Criterion 57 is not applicable in the defueled condition. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, containment penetration isolation is no longer pertinent.
5.1.6.9	Conclusions	Modify	Section is modified to note that Group V criteria are not applicable in the permanently defueled condition (See 5.1.6.1 through 5.1.6.8). Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents.
5.1.7.1	Criterion 60 - Control of Releases of Radioactive Materials to the Environment	Modify	This section is modified to remove reference to defective fuel and to replace the word “Plant” with the word “Facility.”
5.1.7.2	Criterion 61 - Fuel Storage and Handling and Radioactivity Control	Modify	This section is modified to replace the word “Plant” with the word “Facility.”

Section	Title	Change	Description of Change
5.1.7.4	Criterion 63 - Monitoring Fuel and Waste Storage	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.1.7.5	Criterion 64 - Monitoring Radioactivity Releases	Modify	This section is revised to remove reference to monitors no longer credited in safety analyses, and is modified to replace the word "Plant" with the word "Facility." Amendment 272 removed Tech Spec 3.3.3, "Engineered Safety Features (ESF) Instrumentation," 3.3.6, "Refueling Containment High Radiation (CHR) Instrumentation, and 3.3.10, "Engineered Safeguards Room Ventilation (ESRV) Instrumentation," and the facility license no longer contains requirements for the radiation monitors. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not rely on radiation monitors for mitigation.
5.1.7.6	Conclusions	Modify	This section is modified to replace the word "Plant" with the word "Facility." The text is revised to reference "applicable" criteria as some aspects of Group IV criteria are no longer applicable (See 5.1.7.5).
5.1.8	Overall Conclusion	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.2.1	Background Information	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.2.1.2	Original Palisades Design Review	Modify	This section is modified to replace the word "Plant" with the word "Facility."

Section	Title	Change	Description of Change
5.2.2.3	Design - Class 3	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.2.2.7	Service - Electrical and Instrumentation and Controls Equipment Classification	Modify	This section is modified to revise reference to FSAR to the DSAR. After implementation of Amendment 272, the document will be referred to as Defueled Safety Analysis Report.
5.2.2.8.2	Important to Safety	Modify	This section is modified to revise reference to FSAR to the DSAR. After implementation of Amendment 272, the document will be referred to as Defueled Safety Analysis Report.
5.4.1.1	Description of Events	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.4.1.2	Effects on Consumers Design Class 1 Structures and Safety-Related Equipment	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.4.2	Flooding and Wetting from Plant Sources	Modify	This section is modified to replace the word "Plant" with the word "Facility."
Table of Contents 5.5.1.1.2	Auxiliary Building TSC/EER/HVAC Addition	Modify	Error in TOC. Section title in TOC is revised from "Intake Structure" to "Auxiliary Building TSC/EER/HVAC Addition." No change to DSAR text.

Section	Title	Change	Description of Change
Table of Contents 5.5.1.1.3	Diesel Fuel Oil Storage Tank Housing	Modify	Error in TOC. Section title in TOC is revised from "Auxiliary Building Radwaste Addition" to "Diesel Fuel Oil Storage Tank Housing." No change to DSAR text.
5.5.1.1.4 And Table of Contents	Condensate Storage Tank	Modify	Error in TOC. Section title in TOC is revised from "Auxiliary Building TSC/EER/HVAC Addition" to "Condensate Storage Tank" and is modified to replace the word "Plant" with the word "Facility." No change to DSAR text.
Table of Contents 5.5.1.1.5	Equipment Hatch	Modify	Error in TOC. Section title in TOC is revised from "Diesel Fuel Oil Storage Tank Housing" to "Equipment Hatch."
Table of Contents 5.5.1.1.6	Condensate Storage Tank	Delete	Error in TOC. Reference to 5.5.1.1.6 is removed from TOC. There are only 5 subsections.
5.5.1.2	Structural Considerations	Modify	This section is modified remove reference to missile impact to operation of safety related systems (which are no longer required to operate) and to replace the word "Plant" with the word "Facility."
5.5.1.3	Facility Reevaluation	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.5.1.3.2	Summary	Modify	This section is modified to replace the word "Plant" with the word "Facility."

Section	Title	Change	Description of Change
5.5.2	Turbine Missiles	Delete	This section is deleted in its entirety. This section is revised to reflect that the certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore, turbine missiles are no longer a credible scenario.
5.5.3	Internally Generated Missiles	Delete	This section is deleted in its entirety. This section is revised to reflect that the certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore, internally generated missiles are no longer a credible scenario.
5.5.4	Site Proximity Missiles	Modify	This section is modified to revise safe shutdown to maintain fuel in a safe condition, delete reference to backups for safety related equipment, and replace the word "Plant" with the word "Facility." Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and safe shutdown is no longer applicable. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs rendering "backup systems" irrelevant.

Section	Title	Change	Description of Change
5.5	References	Modify	References removed from text are deleted.
5.6	Dynamic Effects of Pipe Rupture	Modify	The sub-sections in 5.6 are deleted with the exception of 5.6.7.3. Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, pipe rupture, and its associated dynamic effects, is no longer a credible scenario. Pipe break in the two moderate energy systems has no safety impact (see 5.6.7.3).
5.6.1	Definitions	Delete	See Section 5.6
5.6.2	Design Bases	Delete	See Section 5.6
5.6.3	Criteria Used to Define Breaks	Delete	See Section 5.6
5.6.4	Protective Measures	Delete	See Section 5.6
5.6.5	Jet Impingement	Delete	See Section 5.6

Section	Title	Change	Description of Change
5.6.6	Plant Modification Line-Break Analysis	Delete	See Section 5.6
5.6.7	History of Palisades High-Energy Line-Break Analysis	Delete	See Section 5.6
5.6.7.1	High-Energy Line Breaks Outside Containment	Delete	See Section 5.6
5.6.7.2	High-Energy Line Breaks Inside Containment	Delete	See Section 5.6
5.6.7.3	Moderate-Energy System Pipe-Break Evaluation	Modify	The sections is revised to note no safety impact from pipe break in CCW or SWS. Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, pipe rupture, and its associated dynamic effects, is no longer a credible scenario. Pipe break in the two moderate energy systems has no safety impact (see 5.6.7.3).
5.6	References	Delete	See Section 5.6

Section	Title	Change	Description of Change
5.7	Seismic Design	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.7.1.1	Design Bases	Modify	Section is revised to remove definition of Operating Basis Earthquake (OBE) and revise SSE to safe storage of spent nuclear fuel. This section is modified to revise reference to FSAR to the DSAR. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Reference to plant operation or safe shutdown is no longer relevant, and safe shutdown has been replaced with safe storage of spent fuel. After implementation of Amendment 272, the document will be referred to as Defueled Safety Analysis Report.
5.7.4	Seismic Analysis of CP CO Design Class 1 Piping	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.7.8	Seismic Analysis of Buried Structures and Components	Modify	This section is modified to replace the word "Plant" with the word "Facility."
5.7.9	Seismic Instrumentation	Modify	This section is modified to replace the word "Plant" with the word "Facility."

Section	Title	Change	Description of Change
5.8.1	Design Basis	Modify	This section is modified to remove discussion of accident conditions and scenarios. This section is revised to reflect that the certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, discussion of accident conditions and scenarios inside containment is deleted.
5.8.2	General Description	Modify	This section is revised to remove reference to section 5.8.8.3, which has been deleted, and is modified to replace the word "Plant" with the word "Facility." .
5.8.3.1.2	Working Stress Condition	Modify	This section is modified to remove loadings associated with operation and accident conditions. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, operational and accident conditions are removed from the stress requirements.

Section	Title	Change	Description of Change
5.8.3.1.3	Yield Strength Condition	Modify	This section is modified to remove loadings associated with operation of accident conditions. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, operational and accident conditions are removed from the yield strength requirements.
5.8.3.2.1	Liner Plate	Modify	This section is modified to remove loadings associated with operational and accident conditions. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, operational and accident conditions are removed from the structure load requirements.

Section	Title	Change	Description of Change
5.8.3.2.2	Liner Plate Anchors	Modify	This section is modified to remove loadings associated with operational and accident conditions. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, operational and accident conditions are removed from the fatigue load requirements.
5.8.4.1.1	General	Modify	This section is modified to remove loadings associated with operational and accident conditions. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, operational and accident conditions are removed from the load requirements.

Section	Title	Change	Description of Change
5.8.5.1	Design Basis	Modify	This section is modified to remove reference to accident conditions and leak tightness and is modified to replace the word "Plant" with the word "Facility." Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, reference to operational and accident conditions are removed.
5.8.5.2.1	General Criteria	Modify	This section is modified to remove reference to accident conditions and leak tightness. Reference to Section 5.8.8.2, which has been deleted, is also removed. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, reference to operational and accident conditions are removed.

Section	Title	Change	Description of Change
5.8.5.4.1	General	Modify	This section is modified to remove reference to accident conditions and leak tightness. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, reference to operational and accident conditions are removed.
5.8.5.4.2	Liner Plate	Modify	This section is modified to remove reference to accident conditions. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, reference to operational and accident conditions are removed.

Section	Title	Change	Description of Change
5.8.6.1	Design Basis	Modify	This section is modified to remove reference to accident conditions. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, reference to operational and accident conditions are removed.
5.8.6.2	General Description	Modify	This section is modified to remove reference to leak tightness. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, reference to leak tightness is removed.

Section	Title	Change	Description of Change
5.8.6.2.1	Personnel and Equipment Openings	Modify	This section is modified to remove references to leak testing. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and License Condition 2.D regarding Appendix J testing, and the facility license will no longer require a leakage testing program for containment. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, reference to leak testing is removed.
5.8.6.2.2	Other Openings	Modify	This section is modified to remove references to leak testing. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and License Condition 2.D regarding Appendix J testing, and the facility license will no longer require a leakage testing program for containment. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, reference to leak testing is removed.

Section	Title	Change	Description of Change
5.8.6.4.1	Small Penetrations	Modify	This section is modified to remove reference to accident conditions. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. However, it will continue to be required to maintain its structural integrity to ensure that it does not have any impact on the safe storage of spent fuel in the SFP. Therefore, reference to operational and accident conditions are removed.
5.8.8	Containment Structure Testing	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and License Condition 2.D regarding Appendix J testing, and the facility license will no longer require a leakage testing program for containment. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Once PNP is permanently shut down and defueled, the containment is no longer credited as part of the initial conditions of the remaining applicable accident analyses or as part of the primary success path for mitigation of these events.
5.8.9.2	Containment Reevaluation	Modify	This Section is modified to replace the word “Plant” with the word “Facility.”

Section	Title	Change	Description of Change
5.8.10	Containment Structural Integrity Surveillance Program	Delete	This section is deleted in its entirety Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and Tech Spec Section 5.5.5, "Containment Structural Integrity Surveillance Program," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. The surveillance program was established to satisfy the requirements of the facility Technical Specifications therefore, this program is no longer required.
5.8	References	Modify	References removed from text are deleted
5.9.1.4	Loads Common to All Structures	Modify	This Section is modified to replace the word "Plant" with the word "Facility."

Section	Title	Change	Description of Change
5.9.2.1	General Description	Modify	This section is revised to remove reference to functional requirements related to internally generated missiles and pipe whip and is modified to revise reference to FSAR to the DSAR. Physical descriptions of SSCs are retained. Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents and associated structural loadings are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, and internal missiles and pipe whip are no longer applicable. After implementation of Amendment 272, the document will be referred to as Defueled Safety Analysis Report.
5.9.2.2	Loads	Modify	This section is modified to delete reference to Subsection 5.5.3, which has been deleted.
5.9.3.1	General Description	Modify	This section is revised to remove reference to pipe whip and associated jet forces. See 5.6.3 which was deleted. Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents and associated pipe whip are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
5.9.5.1	General Description	Modify	This section is modified to change FSAR to DSAR. After implementation of Amendment 272, the document will be referred to as Defueled Safety Analysis Report.
5.9.6.1	General Description	Modify	Section is modified by deleting reference to NUREG 0737. NUREG 0737 only applies to operating power reactors.
5.10.3.2.2	Identification	Modify	This Section is modified to replace the word "Plant" with the word "Facility."
Table 5.5-2	Burst Probability for Each LP Rotor and Total Unit at 120% Rated Speed	Delete	Reference to Table is deleted in text 5.5.2.3
Table 5.6-1	High-Energy Pipe Failures Outside Containment - Summary of Operating Stresses - Main Steam	Delete	Section 5.6 is deleted
Table 5.6-2	High-Energy Pipe Failures Outside Containment - Summary of Operating Stresses - Feedwater	Delete	Section 5.6 is deleted

Section	Title	Change	Description of Change
Table 5.6-3	High-Energy Pipe Failures Outside Containment - Summary of Operating Stresses - Main Steam Dump	Delete	Section 5.6 is deleted
Table 5.6-4	Summary of Eliminated Hardware Resulting from Application of Generic Letter 87-11, "Relaxation of Arbitrary Pipe Rupture Requirements"	Delete	Section 5.6 is deleted
Table 5.8-4	Containment Penetrations and Appendix J Test Requirements	Delete	Reference to Table is deleted in text 5.8.8.2.3
Figure 5.8-10	Containment Structure, DBA Thermal Gradients Across Containment Wall, No Insulation	Delete	Reference to Figure is deleted in text 5.8.4.1.2
Figure 5.8-18	Containment Structure, Thermal Gradient at Main Steam Penetration	Delete	Reference to Figure is deleted in text 5.8.6.4.1

Section	Title	Change	Description of Change
Figure 5.8-24	Containment Structure, Location and Identification of Eleven Surveillance Tendons for One- and Three-Year Surveillances	Delete	Reference to Figure is deleted in text 5.8.8.4.4
Figure 5.8-25	Sh 1 Containment Structure, Structural Integrity Test Inside Hoop Strain Profiles, Typical Section	Delete	Reference to Figure is deleted in text 5.8.8.4.4
Figure 5.8-25	Sh 2 containment structure, structural integrity test outside meridional strain profiles, typical section	Delete	Reference to Figure is deleted in text 5.8.8.4.4
Figure 5.8-26	Containment Structure, Pressure Test Displacement Profiles	Delete	Reference to Figure is deleted in text 5.8.8.4.4
Figure 5.8-27	Sh 1 Containment Structure, Pressure Test Tendon Load Change	Delete	Reference to Figure is deleted in text 5.8.8.4.4

Section	Title	Change	Description of Change
Figure 5.8-27	Sh 2 Containment Structure, Pressure Test Tendon Load Change	Delete	Reference to Figure is deleted in text 5.8.8.4.4
Figure 5.8-28	Containment Structure, End Anchorage Surveillance Program, Crack Surveillance Locations	Delete	Reference to Figure is deleted in text 5.8.8.6.3
6.1.1	Design Bases	Modify	Section is revised to delete information on safety functions and accident parameters. SSC classifications were retained along with information on the EQ program. Amendment 272 removed Tech Spec Section 3.5, ECCS, and the facility license no longer contains requirements for the ECCS and associated systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no pertinent engineered safeguards design criteria or design functions in the permanently defueled condition. SSC Classification and EQ program descriptions are retained as these are not addressed in Amendment 272.

Section	Title	Change	Description of Change
6.1.2.1	General Description	Modify	Section is revised to delete information on safety functions and accident parameters. SSC physical descriptions were retained. Amendment 272 removed Tech Spec Section 3.5, ECCS, and the facility license no longer contains requirements for the ECCS and associated systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no pertinent engineered safeguards design criteria or design functions in the permanently defueled condition. SSC physical descriptions are retained in accordance with RG 1.184.
6.1.2.2	Component Design	Modify	Section is revised to delete information on safety functions and accident and operational parameters of ECCS components. SSC physical descriptions were retained. Amendment 272 removed Tech Spec Section 3.5, ECCS, and the facility license no longer contains requirements for the ECCS and associated systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no pertinent engineered safeguards design criteria or design functions in the permanently defueled condition. SSC physical descriptions are retained in accordance with RG 1.184.

Section	Title	Change	Description of Change
6.1.2.3	Operation	Delete	Section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.5, ECCS, and the facility license no longer contains requirements for the ECCS and associated systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no pertinent engineered safeguards operational modes in the permanently defueled condition.
6.1.3	Testing	Delete Subsections	All subsections are deleted.
6.1.3.1	Operational Testing	Delete	Section is deleted in its entirety. Amendment 272 removed TS SECTION 3.5, EMERGENCY CORE COOLING SYSTEMS (ECCS), as well as Tech Spec 3.3.3, Engineered Safety Features (ESF) Instrumentation, 3.3.4, Engineered Safety Features Logic and Manual Initiation, and 3.3.5, Diesel Generator (DG) – Undervoltage Start (UV Start) Instrumentation. Therefore, there are no testing requirements for logic circuits, pumps, or power-actuated valves in the Safety Injection System. Amendment 272 removed the TS definition of inservice testing and associated references in Section 3.0. Inservice testing in accordance with 10 CFR 50.55a is not required in the permanently defueled condition.
6.1.3.2	Environmental Testing	Delete	Section is deleted in its entirety. 10 CFR 50.49 only applies to operating power reactors.

Section	Title	Change	Description of Change
6.1.4	Design Analysis	Modify	Section is revised to delete information on safety functional requirements, design requirements and accident parameters. SSC classifications were retained. Amendment 272 removed Tech Spec Section 3.5, ECCS, and the facility license no longer contains requirements for the ECCS and associated systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no pertinent engineered safeguards safety functional requirements in the permanently defueled condition. SSC Classification descriptions are retained as these are not addressed in Amendment 272.
6.2.1	Design Basis	Modify	Section is revised to delete information on safety functional requirements, design requirements and accident parameters. SSC classifications were retained. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no pertinent safety functional requirements for containment spray in the permanently defueled condition. SSC Classification descriptions are retained as these are not addressed in Amendment 272.

Section	Title	Change	Description of Change
6.2.2.1	General Description	Modify	Section is revised to delete information on safety functional requirements, operation descriptions, design requirements and accident parameters. SSC classifications and physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no pertinent safety functional requirements for containment spray in the permanently defueled condition. SSC Classification and physical descriptions are retained as these are not addressed in Amendment 272.
6.2.2.3	System operation	Delete	Section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no operational or functional requirements for containment spray in the permanently defueled condition.

Section	Title	Change	Description of Change
6.2.3	Design Analysis	Delete	Section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no margin of safety requirements for containment spray in the permanently defueled condition.
6.3.1	Design Bases	Modify	Section is revised to delete information on safety and design functional requirements and accident parameters. SSC classifications and physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no safety and design functional requirements for containment air coolers in the permanently defueled condition.

Section	Title	Change	Description of Change
6.3.2.1	General Description	Modify	Section is revised to delete information on functional requirements and system operation. SSC classifications and physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no functional requirements for containment air coolers in the permanently defueled condition.
6.3.2.2	System Operation	Delete	Section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no functional requirements for containment air coolers in the permanently defueled condition.

Section	Title	Change	Description of Change
6.3.3	Design Analysis	Delete	Section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no safety or design functional requirements for containment air coolers in the permanently defueled condition.
6.4.1	Design Basis	Modify	Section is revised to delete information on qualification and functional requirements of paint inside containment. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no qualification and functional requirements related to paint in the permanently defueled condition.

Section	Title	Change	Description of Change
6.4.2.1	General Description	Modify	Section is revised to delete information on functional requirements. SSC physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no functional requirements related to sump pH control in the permanently defueled condition.
6.4.2.2	Operation	Delete	Section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no operational requirements related to sump pH control in the permanently defueled condition.

Section	Title	Change	Description of Change
6.4.2.3	Materials	Modify	Section is revised to delete information on safety and design functional requirements. SSC physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no safety or design functional requirements related to sump pH control in the permanently defueled condition.
6.4.2.4	Paint	Modify	Section is revised to delete information on qualification and functional requirements of paint inside containment. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.6, Containment Cooling Systems, and the facility license no longer contains requirements for the containment systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no qualification and functional requirements related to paint in the permanently defueled condition.

Section	Title	Change	Description of Change
6.5.1	General	Modify	Section is revised to delete information on iodide generation post DBA. The physical description of the containment charcoal filter is retained. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, iodide production during a DBA is not possible.
6.7	Containment Isolation System	Delete	Section and subsections are deleted in their entirety. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.3, Containment Isolation Valves, and the facility license no longer contains requirements for the containment isolation. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no requirements for containment isolation in the permanently defueled condition.
6.8	Reactor Cavity Flooding System	Modify	This section is modified to remove discussion of post DBA operation. Physical descriptions are retained.
6.8.1	System Operation	Modify	This section is modified to remove discussion of post DBA operation. Physical descriptions are retained.

Section	Title	Change	Description of Change
6.9.2	Pump and Valve Testing Program	Delete	Section is deleted in its entirety. Amendment 272 removed the TS definition of inservice testing and associated references in Section 3.0. Inservice testing in accordance with 10 CFR 50.55a is not required in the permanently defueled condition.
6.9.2.1	Pump Testing Program	Delete	Section is deleted in its entirety. Amendment 272 removed the TS definition of inservice testing and associated references in Section 3.0. Inservice testing in accordance with 10 CFR 50.55a is not required in the permanently defueled condition.
6.9.2.2	Valve Testing Program	Delete	Section is deleted in its entirety. Amendment 272 removed the TS definition of inservice testing and associated references in Section 3.0. Inservice testing in accordance with 10 CFR 50.55a is not required in the permanently defueled condition.
6.10.1	Design Basis	Delete	This section is deleted in its entirety. Amendment 272 deleted TS 3.7.10, Control Room Ventilation (CRV) Filtration, and TS 3.7.11, Control Room Ventilation (CRV) Cooling, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. After certifications are submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible, and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accident analyses do not credit any control room habitability systems while still maintaining CR doses within licensing basis requirements, and therefore there is no safety design basis for control room habitability.

Section	Title	Change	Description of Change
6.10.2	System Design	Modify	This section modified to remove discussion of safety functions and accident conditions. Amendment 272 deleted TS 3.7.10, Control Room Ventilation (CRV) Filtration, and TS 3.7.11, Control Room Ventilation (CRV) Cooling, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. After certifications are submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible, and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accident analyses do not credit any control room habitability systems while still maintaining CR doses within licensing basis requirements, and therefore reference to safety functions and accident conditions are deleted.
6.10.3	Design Analysis	Delete	This section is deleted in its entirety. Amendment 272 deleted TS 3.7.10, Control Room Ventilation (CRV) Filtration, and TS 3.7.11, Control Room Ventilation (CRV) Cooling, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. After certifications are submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible, and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accident analyses do not credit any control room habitability systems while still maintaining CR doses within licensing basis requirements, and therefore there is no safety design basis for control room habitability.

Section	Title	Change	Description of Change
6.0	References	Modify	This section is modified to delete references which have been removed from the text. In addition, Reference 23, which is the license amendment that deleted hydrogen recombiners and is not used in the text, is deleted.
Table 6-1	Safety Injection and Refueling Water Tank Design Parameters	Modify	The information in this table regarding operational parameters for the tank is deleted. Equipment descriptions are retained. Amendment 272 deleted Tech Spec Section 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Operational parameters on equipment required for core related design basis accidents is deleted.

Section	Title	Change	Description of Change
Table 6-2	Low Pressure Safety Injection Pump Data Summary	Modify	The information in this table regarding operational and DBA parameters for the LPSI pump is deleted. Equipment descriptions are retained. Amendment 272 deleted Tech Spec Section 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Operational and DBA parameters on equipment required for core related design basis accidents is deleted.
Table 6-3	High Pressure Safety Injection Pump Data Summary	Modify	The information in this table regarding operational and DBA parameters for the HPSI pump is deleted. Equipment descriptions are retained. Amendment 272 deleted Tech Spec Section 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Operational and DBA parameters on equipment required for core related design basis accidents is deleted.

Section	Title	Change	Description of Change
Table 6-4	Shutdown Cooling Heat Exchanger Data Summary	Modify	The information in this table regarding operational and DBA parameters for the heat exchanger is deleted. Equipment descriptions are retained. This table is revised to reflect that the certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Operational and DBA parameters on equipment required for core related design basis accidents is deleted.
Table 6-5	Safety Injection Tank Design Parameters	Modify	The information in this table regarding operational and DBA parameters for the SI tank is deleted. Equipment descriptions are retained. Amendment 272 deleted Tech Spec Section 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Operational and DBA parameters on equipment required for core related design basis accidents is deleted.

Section	Title	Change	Description of Change
Table 6-6	Containment Spray System Component Description	Modify	The information in this table regarding operational and DBA parameters for containment spray is deleted. Equipment descriptions are retained. Amendment 272 deleted Tech Spec Section 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Operational and DBA parameters on equipment required for core related design basis accidents is deleted.
Table 6-7	Containment Air Cooler Component Description	Modify	The information in this table regarding operational and DBA parameters for air co0olers is deleted. Equipment descriptions are retained. This table is revised to reflect that the certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Operational and DBA parameters on equipment required for core related design basis accidents is deleted.

Section	Title	Change	Description of Change
Table 6-8	Containment Air Coolers Performance Data for Normal Operation	Delete	This table is deleted in its entirety. Amendment 272 deleted Tech Spec Section 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Operational parameters on equipment required for core related design basis accidents is deleted.
Table 6-9	Containment Air Coolers Performance Data for Post-DBA Conditions	Delete	This table is deleted in its entirety. Amendment 272 deleted Tech Spec Section 3.5, EMERGENCY CORE COOLING SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. DBA parameters on equipment required for core related design basis accidents is deleted.
Table 6-13	Inservice Pump Test Program Summary	Delete	Table is deleted in its entirety. Amendment 272 removed the TS definition of inservice testing and associated references in Section 3.0. Inservice testing in accordance with 10 CFR 50.55a is not required in the permanently defueled condition.

Section	Title	Change	Description of Change
Table 6-14	Technical Specification 3.6.3, "Containment Isolation Valves," Applicability	Delete	Table is deleted in their entirety. Amendment 272 removed Tech Spec Section 3.5, CONTAINMENT SYSTEMS, including TS 3.6.3, Containment Isolation Valves, and the facility license no longer contains requirements for the containment isolation. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no requirements for containment isolation in the permanently defueled condition.
Figure 6-3	Containment Air Coolers Relief Panel & Condensate Drain	Delete	This figure is deleted in its entirety. Amendment 272 deleted Tech Spec Section 3.5, EMERGENCY CORE COOLING SYSTEMS, and 3.6, CONTAINMENT SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. This figure provides operational and DBA features for the air coolers and is deleted.

Section	Title	Change	Description of Change
Figure 6-7	Capacity Curve 1713a Nozzle	Delete	This figure provides operational information for core spray is deleted in its entirety. Amendment 272 deleted Tech Spec Section 3.5, EMERGENCY CORE COOLING SYSTEMS, and 3.6, CONTAINMENT SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Operational parameters on equipment required for core related design basis accidents is deleted.
7.1	Introduction	Modify	This section is revised to delete discussion of control plant operations, instrumentation safety functions, and EQ. Also, "Plant" is replace with "Facility." Amendment 272 removed Tech Spec Section 3.3, Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent instrumentation or control functional requirements in the permanently defueled condition. In addition, EQ and 10 CFR 50.49 only apply to operating power reactors.

Section	Title	Change	Description of Change
7.2.1	General	Modify	This section is revised to delete functional and design requirements of the RPS. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent RPS design or functional requirements in the permanently defueled condition.
7.2.2	Design Bases	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, the design basis of the RPS instrumentation is no longer pertinent.

Section	Title	Change	Description of Change
7.2.3	Reactor Protective System Actions	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent RPS functional requirements in the permanently defueled condition.
7.2.4	Signal Generation	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent RPS functional requirements in the permanently defueled condition.

Section	Title	Change	Description of Change
7.2.5	Logic Operation	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent RPS functional requirements in the permanently defueled condition.
7.2.6	Testing	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. Therefore, there are no RPS testing requirements.

Section	Title	Change	Description of Change
7.2.7	Effects of Failures	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent RPS failure effects in the permanently defueled condition.
7.2.8	Power sources	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no power requirements for the RPS in the permanently defueled condition.

Section	Title	Change	Description of Change
7.2.9	Physical Separation and Electrical Isolation	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no separation requirements of the RPS in the permanently defueled condition.
7.2.10	Reactor Trip and Pretrip Set Points	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent RPS functional requirements in the permanently defueled condition.

Section	Title	Change	Description of Change
7.3	Engineered Safeguards Controls	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.3, Engineered Safety Features (ESF) Instrumentation, and TS 3.3.4, Engineered Safety Features (ESF) Logic and Manual Initiation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent design, functional or testing requirements for ECCS instrumentation in the permanently defueled condition.
7.4	Other Safety Related Protection, Control and Display Systems	Modify	This section is modified to limit discussion to mitigation of uncontrolled release of radioactive effluents. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, discussion of RPS and ECCS protective functions is deleted.

Section	Title	Change	Description of Change
7.4.1	Reactor Shutdown Controls	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, reactor shutdown is no longer pertinent.
7.4.2	Primary Coolant Boundaries Protection	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, reactor coolant leakage and boundary protection is not pertinent.
7.4.3	Auxiliary Feedwater Controls	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, and TS 3.7.5, Auxiliary Feedwater (AFW) System, the facility license no longer contains requirements for AFW controls. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, AFW controls are not pertinent.

Section	Title	Change	Description of Change
7.4.4	Containment Hydrogen Controls	Delete	Section is deleted. Hydrogen recombiners were abandoned previously and are considered obsolete.
7.4.4.1	Design Basis	Delete	Section is deleted. Hydrogen recombiners were abandoned previously and are considered obsolete.
7.4.5.1	Control Room	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec 3.7.10, Control Room Ventilation (CRV) Filtration, and 3.7.11, Control Room Ventilation (CRV) Cooling, and the facility license no longer contains requirements for control room ventilation. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, control room ventilation instrumentation has no functional requirements.
7.4.5.2	Engineered Safeguards Pump Rooms	Delete	This section is deleted in its entirety. Amendment 272 removed TS 3.3.10, Engineered Safeguards Room Ventilation (ESRV) Instrumentation, and the facility license no longer contains requirements for pump room ventilation and associated instrumentation. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, pump room ventilation instrumentation has no functional requirements.

Section	Title	Change	Description of Change
7.4.6	Other Safety Related Display System	Delete	This section is deleted in its entirety. Amendment 272 removed TS 3.3.7, Post Accident Monitoring (PAM) Instrumentation, the certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and the 10 CFR Part 50 license will longer contain requirements for the RG 1.97 instrumentation in Section 7.4.6. Therefore, design and functional requirements for this instrumentation are deleted.
7.5.1	Design Bases	Delete	This section is deleted in its entirety. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, there are no functional requirements for primary or secondary regulating controls, and the design bases can be deleted.
7.5.2.1	Reactor Regulating	Modify	This section is modified to delete design requirements for reactor regulating controls. Physical descriptions are retained. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, there are no functional requirements for primary regulating controls, and the design bases can be deleted.
7.5.2.2	Primary Pressure Regulating	Delete	This section is deleted to delete design requirements for primary pressure regulating controls. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, there are no functional requirements for primary regulating controls, and the design bases can be deleted.

Section	Title	Change	Description of Change
7.5.2.3	Feedwater Regulating	Delete	This section is deleted to delete design requirements for feedwater regulating controls. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, there are no functional requirements for feedwater regulating controls, and the design bases can be deleted.
7.5.2.4	Pressurizer Level Regulating	Delete	This section is deleted to delete design requirements for pressurizer level regulating controls. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, there are no functional requirements for pressurizer level regulating controls, and the design bases can be deleted.
7.5.2.5	Steam Dump and Bypass	Modify	This section is deleted to delete design requirements for steam dump and bypass controls. Physical descriptions are retained. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, there are no functional requirements for steam dump and bypass controls, and the design bases can be deleted.

Section	Title	Change	Description of Change
7.5.2.6	Turbine Generator Controls	Delete	This section is deleted to delete design requirements for Turbine generator controls. Physical descriptions are retained. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, there are no functional requirements for turbine generator controls, and the design bases can be deleted.
7.5.3	System Evaluation	Delete	This section is deleted in its entirety. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, there are no functional requirements for primary or secondary regulating controls, and associated system evaluations can be deleted.
7.6	Nuclear Steam Supply System Instrumentation	Modify Title to Plant Process Computer and Miscellaneous Instrumentation	Editorial changes are made to this section to remove reference to deleted FSAR sections.
7.6.1.1	Process Instrumentation	Delete	This section is deleted in its entirety. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur and there are no functional requirements for primary or secondary process instrumentation.

Section	Title	Change	Description of Change
7.6.1.2	Nuclear Instrumentation	Delete	This section is deleted in its entirety. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur and there are no functional requirements for nuclear instrumentation.
7.6.1.3	Control Rod Position Instrumentation	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.1, REACTIVITY CONTROL SYSTEMS, and the facility license no longer contains requirements for control rod position instrumentation. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur and there are no functional requirements for control rod position indication instrumentation.
7.6.1.4	Incore Instrumentation	Delete	This section is revised to delete functional and design requirements of incore instrumentation. Amendment 272 removed TS 3.2.1 Linear Heat Rate (LHR), and the facility license no longer contains requirements for incore instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and there are no functional requirements for incore instrumentation.

Section	Title	Change	Description of Change
7.6.1.5	Palisades Plant Computer (PPC)	Modify	This section is revised to delete reference to instrumentation associated with reactor and primary systems operation, and to change plant to facility. Amendment 272 removed Tech Spec Sections 3.1, REACTIVITY CONTROL SYSTEMS, 3.2, POWER DISTRIBUTION LIMITS, and 3.3, INSTRUMENTATION, and the facility license no longer contains requirements for these instrumentation systems: 1. Reactor Protective System 2. Engineered Safeguards Controls 3. Reactor Shutdown Controls 4. Fluid Systems Protection 5. Regulating Controls 6. Primary Plant Process Instruments The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, there are no functional requirements for this instrumentation.
7.6.2.1	Process Instrumentation	Delete	This section is revised to delete functional descriptions of the process instrumentation. Amendment 272 removed Tech Spec Sections 3.1, REACTIVITY CONTROL SYSTEMS, 3.2, POWER DISTRIBUTION LIMITS, and 3.3, INSTRUMENTATION, and the facility license no longer contains requirements for process instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, there are no functional requirements for process instrumentation.

Section	Title	Change	Description of Change
7.6.2.2	Nuclear Instrumentation	Modify	This section is revised to delete functional descriptions of the nuclear instrumentation. Physical descriptions of the RPS are retained. Amendment 272 removed Tech Spec Sections 3.1, REACTIVITY CONTROL SYSTEMS, 3.2, POWER DISTRIBUTION LIMITS, and 3.3, INSTRUMENTATION, and the facility license no longer contains requirements for process instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, there are no functional requirements for nuclear instrumentation.
7.6.2.3	Control rod position Instrumentation	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.1, REACTIVITY CONTROL SYSTEMS, and the facility license no longer contains requirements for control rod position instrumentation. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, all control rods have been removed from the core and there are no functional requirements for control rod position indication instrumentation.

Section	Title	Change	Description of Change
7.6.2.4	Incore Instrumentation	Delete	This section is revised to delete functional and physical descriptions of incore instrumentation. Amendment 272 removed TS 3.2.1 Linear Heat Rate (LHR), and the facility license no longer contains requirements for incore instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, incore instrumentation has no function and will no longer be installed in the core.
7.6.2.5	Palisades Plant Computer	Modify	Section is revised to change plant to facility
7.7.1	General Layout	Modify	This section is revised to delete reference to normal and emergency plant operations and change Plant to Facility. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, the operations referred to are no longer applicable.

Section	Title	Change	Description of Change
7.7.2	Control Room	Modify	This section is revised to delete reference to DBA conditions, safe shutdown, and temperature restrictions on RPS, Engineered Safeguard and Thermal Margin Monitor instrumentation. Design features and physical descriptions of the control room are retained. Amendment 272 removed Tech Spec Sections 3.1, REACTIVITY CONTROL SYSTEMS, 3.2, POWER DISTRIBUTION LIMITS, and 3.3, INSTRUMENTATION, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, DBA and safe shutdown are not relevant and there are no functional requirements for the instrumentation.
7.7.3	Engineered Safeguards Auxiliary Panel (C-33)	Modify	This section is revised to remove functional description and fire protection features of the safeguards auxiliary panel. A physical description of the panel is retained. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.3, Engineered Safety Features (ESF) Instrumentation, and TS 3.3.4, Engineered Safety Features (ESF) Logic and Manual Initiation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent design or functional requirements for engineered safeguards instrumentation in the permanently defueled condition.

Section	Title	Change	Description of Change
7.7.4	Auxiliary Hot Shutdown Control Panels (C-150/C-150A)	Modify	This section is revised to remove functional description and fire protection features of the safeguards auxiliary panel. A physical description of the panel is retained. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.8, Alternate Shutdown System, and the facility license no longer contains requirements for alternate shutdown instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent design or functional requirements for alternate shutdown features.
7.8	Quality Control	Delete	Section 7.8 is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems, and no pertinent RPS design or functional requirements in the permanently defueled condition. Therefore, quality control aspects of initial procurement and construction are no longer pertinent.

Section	Title	Change	Description of Change
7.0	References	Modify	Revised to reflect references deleted in the text
Appendix 7A	"Engineered Safeguard Testing	Delete	This Appendix is removed in its entirety. Amendment 272 removed Tech Spec 3.3.3, Engineered Safety Features (ESF) Instrumentation, TS 3.3.4, Engineered Safety Features (ESF) Logic and Manual Initiation, Tech Spec Section 3.5, EMERGENCY CORE COOLING SYSTEMS (ECCS), Tech Spec Section 3.6, CONTAINMENT SYSTEMS, and requirements for Inservice Testing. The facility license no longer contains requirements for Engineered Safeguard Testing. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent design or functional requirements for engineered safeguards in the permanently defueled condition.

Section	Title	Change	Description of Change
Appendix 7C	"Regulatory Guide 1.97 Rev 3, Parameter Summary Table"	Modify	This Appendix is modified by eliminating instrumentation required for core related accident scenarios. Radiation monitoring and meteorological instrumentation required for the remaining accidents is retained. Amendment 272 removed accident scenarios related to the NSSS and deleted Tech Spec 3.3.7, Post Accident Monitoring (PAM) Instrumentation. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, instrumentation required for core related accident scenarios are no longer relevant.
Table 7-1	Reactor Protective System Relays	Delete	This table is deleted in its entirety. Section 7.2.5 has been deleted and this table is no longer utilized.
Table 7-2	Regulating Rods Withdrawal Interlocks	Delete	This table is deleted in its entirety. Reference to Table 7-2 has been removed from 7.5.2.1. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, there are no functional requirements for primary regulating controls, and the design bases can be deleted.

Section	Title	Change	Description of Change
Table 7-3	Source/Wide-Range Nuclear Instrument Channel Trip Unit Actions	Delete	This table is deleted in its entirety. Reference to Table 7-3 has been removed from 7.6.2.2. Amendment 272 removed Tech Spec Sections 3.1, REACTIVITY CONTROL SYSTEMS, 3.2, POWER DISTRIBUTION LIMITS, and 3.3, INSTRUMENTATION, and the facility license no longer contains requirements for process instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, there are no functional requirements for nuclear instrumentation.
Table 7-4	Power-Range Safety Channel Trip Unit Actions	Delete	This table is deleted in its entirety. Reference to Table 7-4 has been removed from 7.6.2.2. Amendment 272 removed Tech Spec Sections 3.1, REACTIVITY CONTROL SYSTEMS, 3.2, POWER DISTRIBUTION LIMITS, and 3.3, INSTRUMENTATION, and the facility license no longer contains requirements for process instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, there are no functional requirements for nuclear instrumentation.

Section	Title	Change	Description of Change
Table 7-5	Control Rod Position Light Matrix	Delete	This table is deleted in its entirety. Reference to Table 7-5 has been removed from 7.6.2.3. Amendment 272 removed Tech Spec Section 3.1, REACTIVITY CONTROL SYSTEMS, and the facility license no longer contains requirements for control rod position instrumentation. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, and there are no functional requirements for control rod position indication instrumentation.
Figure 7-1	Reactor Protective System Block Diagram	Delete	This figure is deleted in its entirety. Reference to Figure 7-1 thru 7-5, 7-7 through 7-13, have been deleted from Sections 7.2. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.1, Reactor Protective System (RPS) Instrumentation, and the facility license no longer contains requirements for RPS instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, the design basis, functional requirements and schematics of the RPS instrumentation is no longer pertinent.
Figure 7-2	Reactor Protective System Functional Diagram	Delete	This figure is deleted. See Figure 7-1 basis.

Section	Title	Change	Description of Change
Figure 7-3	Typical Measurement Channel Functional Diagram	Delete	This figure is deleted. See Figure 7-1 basis.
Figure 7-4	Schematic Diagram, Pressurizer Pressure ATWS	Delete	This figure is deleted. See Figure 7-1 basis.
Figure 7-5	Schematic Diagram, Pressurizer Pressure ATWS	Delete	This figure is deleted. See Figure 7-1 basis.
Figure 7-6	Low Flow Protective System Functional Diagram	Delete	This figure is deleted in its entirety. Figure 7-6 is not referenced in the UFSAR. Amendment 272 removed Tech Spec Sections 3.1, REACTIVITY CONTROL SYSTEMS, 3.2, POWER DISTRIBUTION LIMITS, and 3.3, INSTRUMENTATION, and the facility license no longer contains requirements for process instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, there are no functional requirements for process instrumentation.
Figure 7-7	Logic for Thermal Margin Monitor	Delete	This figure is deleted. See Figure 7-1 basis.

Section	Title	Change	Description of Change
Figure 7-8	Neutron Flux Monitoring System Start- Up and Logarithmic Range Channels	Delete	This figure is deleted. See Figure 7-1 basis.
Figure 7-9	Neutron Flux Monitoring System Power Range Channels	Delete	This figure is deleted. See Figure 7-1 basis.
Figure 7-10	Power Rate-of-Change Trip and Pretrip Interface with RPS	Delete	This figure is deleted. See Figure 7-1 basis.
Figure 7-11	Zero Power Mode Bypass	Delete	This figure is deleted. See Figure 7-1 basis.
Figure 7-12	Reactor Protective System Interfaces	Delete	This figure is deleted. See Figure 7-1 basis.

Section	Title	Change	Description of Change
Figure 7-13	Logic Diagram, Safety Injection Initiation	Delete	This figure is deleted in its entirety. Reference to Figures 7-13, 7-14 Sh 2, 7-15 thru 7-26, and 7-30 thru 32 have been deleted from Sections 7.3. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.3, Engineered Safety Features (ESF) Instrumentation, and 3.3.4, Engineered Safety Features (ESF) Logic and Manual Initiation, and the facility license no longer contains requirements for ESF instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, the design basis, functional requirements and schematics of the EFS instrumentation is no longer pertinent.
Figure 7-14	Sh. 1 Logic Diagram, Main Generator Protection	Delete	Figure 7-14 Sh 1 and all other 7-14 sheets are deleted. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.3, Engineered Safety Features (ESF) Instrumentation, and 3.3.4, Engineered Safety Features (ESF) Logic and Manual Initiation, and the facility license no longer contains requirements for ESF instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no instrumentation functional requirements and instrumentation are no longer pertinent.

Section	Title	Change	Description of Change
Figure 7-14	Sh 2 Logic Diagram, Turbine-Generator Trips and Fast Transfer	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-14	Sh 3 Logic Diagram, 2400 Volt Load Shed	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.
Figure 7-14	Sh 4 Logic Diagram, Diesel Start (CPCO)	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.
Figure 7-14	Sh 5 Logic Diagram, Diesel Engine Control, Trips and Alarms	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.
Figure 7-14	Sh 6 Logic Diagram, Diesel Generator Breakers	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.
Figure 7-14	Sh 7 Logic Diagram, Bus 1C and 1D Incoming Breakers	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.
Figure 7-14	Sh 8 Logic Diagram, Power to Bus 1E, 1F and 1G Incoming Breakers	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.

Section	Title	Change	Description of Change
Figure 7-14	Sh 9 Logic Diagram, Bus 1A & 1B Incoming Breakers	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.
Figure 7-14	Sh 10 Logic Diagram, Generator Breaker and M.O.D. Control	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.
Figure 7-14	Sh 11 Logic Diagram, Start-up Xfmr. Protection	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.
Figure 7-14	Sh 12 Logic Diagram, Safeguard Transformer 1-1	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.
Figure 7-14	Sh 13 Logic Diagram, Circuit Breaker Operation	Delete	This figure is deleted. See Figure 7-14 Sh 1 basis.
Figure 7-15	Schematic Diagram, Safety Injection Signal Auxiliary Circuits	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-16	Schematic Diagram, Safety Injection Signal Auxiliary Circuits	Delete	This figure is deleted. See Figure 7-13 basis.

Section	Title	Change	Description of Change
Figure 7-17	Schematic Diagram, Safety Injection and Sequence Loading Circuits	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-18	Schematic Diagram, Safety Injection and Sequence Loading Circuits	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-19	Schematic Diagram, Safety Injection and Sequence Loading Circuits	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-20	Schematic Diagram, Safety Injection and Sequence Loading Circuits	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-21	Schematic Diagram, Safety Injection and Sequence Loading Test Circuit	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-22	Schematic Diagram, Safety Injection and Sequence Loading Test Circuit	Delete	This figure is deleted. See Figure 7-13 basis.

Section	Title	Change	Description of Change
Figure 7-23	Schematic Diagram, Safety Injection and Sequence Loading Test Circuit	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-24	Logic Diagram, Containment High Pressure Signal	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-25	Logic Diagram, Containment High Radiation	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-26	Schematic Diagram, Containment High Pressure, High Radiation and SIRW Tank Low Level	Delete	This figure is deleted. See Figure 7-13 basis.

Section	Title	Change	Description of Change
Figure 7-27	Schematic Diagram, Containment High Pressure, High Radiation and SIRW Tank Low Level	Delete	This figure is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.3, Engineered Safety Features (ESF) Instrumentation, and TS 3.3.4, Engineered Safety Features (ESF) Logic and Manual Initiation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent design, functional or testing requirements for containment sump recirculation initiation in the permanently defueled condition.
Figure 7-28	Schematic Diagram, Containment Isolation on High Pressure or High Radiation	Delete	This figure is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.3, Engineered Safety Features (ESF) Instrumentation, and TS 3.3.4, Engineered Safety Features (ESF) Logic and Manual Initiation, and the facility license no longer contains requirements for instrumentation systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These do not credit operator action or any active systems. Therefore, there are no pertinent design, functional or testing requirements for containment isolation instrumentation in the permanently defueled condition.

Section	Title	Change	Description of Change
Figure 7-29	Schematic Diagram, Containment Isolation on High Pressure or High Radiation	Delete	This figure is deleted. See Figure 7-28 Basis.
Figure 7-30	Logic Diagram, SIS Test and RAS	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-31	Schematic Diagram, SIRW Tank and Containment Sump Valves	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-32	Schematic Diagram, SIRW Tank and Containment Sump Valves	Delete	This figure is deleted. See Figure 7-13 basis.
Figure 7-33	Logic Diagram, Legend and Notes	Delete	This Figure is deleted. All logic diagrams have been deleted from Chapter 7 and there is no need for diagram legends and notes.
Figure 7-34	Logic Diagram, Legend and Notes	Delete	This Figure is deleted. All logic diagrams have been deleted from Chapter 7 and there is no need for diagram legends and notes.
Figure 7-35	Logic Diagram, Legend and Notes	Delete	This Figure is deleted. All logic diagrams have been deleted from Chapter 7 and there is no need for diagram legends and notes.

Section	Title	Change	Description of Change
Figure 7-36	Schematic Diagram, Steam Generator Level Instrumentation	Delete	This figure is deleted in its entirety. Reference to Figures 7-36 thru 7-52 have been deleted from Sections 7.4. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, AFW controls are not pertinent.
Figure 7-37	Logic Diagram, Auxiliary Feedwater Actuation System	Delete	This figure is deleted. See Figure 7-36.
Figure 7-38	Logic Diagram, Motor Driven Auxiliary Feedwater Pump P-8A	Delete	This figure is deleted. See Figure 7-36.
Figure 7-39	Logic Diagram, Motor Driven Auxiliary Feedwater Pump P-8C	Delete	This figure is deleted. See Figure 7-36.
Figure 7-40	Sh 1 Logic Diagram, Turbine Driven Auxiliary Feedwater Pump P-8B	Delete	This figure is deleted. See Figure 7-36.

Section	Title	Change	Description of Change
Figure 7-40	Sh 2 Logic Diagram, Turbine Driven Auxiliary Feedwater Pump P-8B	Delete	This figure is deleted. See Figure 7-36.
Figure 7-41	Interface Loop Diagram, Pump Low Suction Pressure Trip	Delete	This figure is deleted. See Figure 7-36.
Figure 7-41	Sh 1 Logic Diagram Turbine Driven Auxiliary Feedwater Pump P-8B	Delete	This figure is deleted. See Figure 7-36.
Figure 7-41	Sh 2 Logic Diagram Turbine Driven Auxiliary Feedwater Pump P-8B	Delete	This figure is deleted. See Figure 7-36.
Figure 7-42	Logic Diagram, Auxiliary Feedwater Pumps Low Suction Pressure Trip	Delete	This figure is deleted. See Figure 7-36.
Figure 7-43	Interface Loop Diagram, Flow Control	Delete	This figure is deleted. See Figure 7-36.
Figure 7-44	Interface Loop Diagram, Flow Control	Delete	This figure is deleted. See Figure 7-36.

Section	Title	Change	Description of Change
Figure 7-45	Interface Loop Diagram, Flow and Pressure Indication	Delete	This figure is deleted. See Figure 7-36.
Figure 7-46	Logic Diagram, Aux Feedwater Flow Control and Pump Test Logic	Delete	This figure is deleted. See Figure 7-36.
Figure 7-47	Logic Diagram, Auxiliary Feedwater Flow Control and Pump Test Logic	Delete	This figure is deleted. See Figure 7-36.
Figure 7-48	Logic Diagram, Auxiliary Feedwater Actuation System	Delete	This figure is deleted. See Figure 7-36.
Figure 7-49	Logic Diagram, Auxiliary Feedwater - Steam Generator Isolation Valves	Delete	This figure is deleted. See Figure 7-36.
Figure 7-50	Logic Diagram, Auxiliary Feedwater - Steam Generator Isolation Valves	Delete	This figure is deleted. See Figure 7-36.

Section	Title	Change	Description of Change
Figure 7-51	Logic Diagram, AFAS-FOGG Remote Display and Annunciator Assignment	Delete	This figure is deleted. See Figure 7-36.
Figure 7-52	Logic Diagram, AFAS-FOGG Remote Display and Annunciator Assignment	Delete	This figure is deleted. See Figure 7-36.
Figure 7-53	Reactor Shutdown Controls	Delete	This figure is deleted in its entirety. Reference to Figure 7-53 thru 7-59 have been deleted from Sections 7.5. Amendment 272 has been implemented and the certifications required by 10 CFR 50.82(a)(1) have been docketed, therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, there are no functional requirements for primary or secondary regulating controls, and the figures can be deleted.
Figure 7-54	Reactor Regulating System Block Diagram	Delete	This figure is deleted. See Figure 7-53.
Figure 7-55	Rod Drive Control System Schematic Diagram	Delete	This figure is deleted. See Figure 7-53.
Figure 7-56	Rod Position Set Points	Delete	This figure is deleted. See Figure 7-53.

Section	Title	Change	Description of Change
Figure 7-57	Pressure Control Program	Delete	This figure is deleted. See Figure 7-53.
Figure 7-58	Block Diagram, Steam Dump and Bypass System	Delete	This figure is deleted. See Figure 7-53.
Figure 7-59	Pressurizer Level Control System Failures Study - Mode "A" Failure	Delete	This figure is deleted. See Figure 7-53.
8.1.1	Design Basis	Modify	This section is modified by deleting the requirements of GDC 17 (see 5.1.3.8), deleting reference to accident conditions and engineered safeguards functional requirements, and deleting reference to turbine generator. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for the electrical systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no applicable accident conditions.

Section	Title	Change	Description of Change
8.1.2	Description and Operation	Modify	This section is modified by deleting the requirements of GDC 17 (see 5.1.3.8), reference to operational conditions, and turbine generator power. An editorial change is made to the description of offsite power to match 8.2.2 and 8.2.3. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for the electrical systems. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, there are no applicable accident or abnormal operational conditions.
8.1.3	Environmental Qualification of Electrical Equipment	Delete	This section provided the licensing history relate to EQ and is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. With no postulated loss of coolant accidents, harsh environments no longer exist.

Section	Title	Change	Description of Change
8.1.5	Station Blackout	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. 10 CFR 50.63, Loss of all alternating current power, limits itself to plants "licensed to operate," as stated below: 50.63(a) Requirements. (1) Each light-water-cooled nuclear power plant licensed to operate under this part.... Therefore, station blackout is no longer applicable.
8.2.1	Design Basis	Modify	The section is modified by deleting reference to plant operation and unit trip. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for the offsite power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore, switchyard functional requirements are no longer applicable.
8.2.2	Description and Operation	Modify	The section is modified by deleting reference to plant operation and unit trip. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for the offsite power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur.

Section	Title	Change	Description of Change
8.2.3	Design Analysis	Modify	The section is modified by deleting the functional requirements of GDC 17 (see 5.1.3.8) and deleting reference to plant operation and unit trip. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for the offsite power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible.
8.3.1.1	Design Basis	Modify	The section is modified by deleting the reference to DBA conditions and operational requirements. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no operational requirements for 4160V systems.

Section	Title	Change	Description of Change
8.3.1.2	Description and Operation	Modify	The section is modified by deleting the reference to main generator power, 4160V functional requirements, 4160V testing, and plant operation and unit trip. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional requirements for 4160V systems.
8.3.1.3	Design Analysis	Modify	The section is modified to remove reference to turbine generator power, generator coastdown and shutdown. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional requirements for 4160V systems.

Section	Title	Change	Description of Change
8.3.2.1	Design Basis	Modify	This Section is modified by deleting the reference to functional requirements, testing and operation of the 2400V system. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for 2400V systems.
8.3.2.2	Description and Operation	Modify	This Section is modified by deleting the reference to functional requirements, testing and operation of the 2400V system. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for 2400V systems.

Section	Title	Change	Description of Change
8.3.2.3	Design Analysis	Modify	This Section is modified by deleting the reference to functional requirements, testing and operation of the 2400V system. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for 2400V systems.
8.3.3.1	Design Basis	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the 480V system. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for 480V systems.

Section	Title	Change	Description of Change
8.3.3.2	Description and Operation	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the 480V system. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for 480V systems.
8.3.4	Control Rod Drive Power	Delete	Section 8.3.4 is deleted in its entirety. Amendment 272 removed Tech Spec Sections 3.1, REACTIVITY CONTROL SYSTEMS, 3.2, POWER DISTRIBUTION LIMITS, and 3.8 "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for control rod power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and no functional or operational requirements for control rod power.

Section	Title	Change	Description of Change
8.3.5.1	Design Basis	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the DC and Preferred AC systems. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for DC and Preferred AC systems.
8.3.5.2	Description and Operation	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the DC and Preferred AC systems. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for DC and Preferred AC systems.

Section	Title	Change	Description of Change
8.3.5.3	Design Analysis	Delete	This section is deleted. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for DC and Preferred AC systems.
8.3.6.2	Description and Operation	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the Instrument AC system. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Instrument AC systems.

Section	Title	Change	Description of Change
8.3.6.3	Design Analysis	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the Instrument AC system. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Instrument AC systems.
8.4	Emergency Power Sources	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the Emergency Power systems. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.

Section	Title	Change	Description of Change
8.4.1.1	Design Basis	Delete	This section is deleted. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.
8.4.1.2	Description and Operation	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the Emergency Power systems. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.

Section	Title	Change	Description of Change
8.4.1.3	Design Analysis	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the Emergency Power systems. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.
8.4.2.1	Design Basis	Delete	This section is deleted. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.

Section	Title	Change	Description of Change
8.4.2.2	Description and Operation	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the Emergency Power systems. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.
8.4.2.3	Design Analysis	Delete	This section is deleted. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.

Section	Title	Change	Description of Change
8.4.2.4	Dedicated Battery Supply to Address Fire Scenarios	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the Emergency Power systems. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.
8.4.3	Turbine Generator Coastdown	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.

Section	Title	Change	Description of Change
8.4.4	Emergency Power Supply for Pressurizer Heaters	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.
8.4.5.1	Design Basis	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the Emergency Power systems. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.

Section	Title	Change	Description of Change
8.4.5.2	Description and Operation	Modify	This section is modified by deleting the reference to functional requirements, testing and operation of the Emergency Power systems. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.
8.5.1.2	Electrical Penetrations of Reactor Containment	Delete	This section is deleted in its entirety. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, this FSAR section is modified to remove design and safety functional requirements.
8.5.2	Design Description	Modify	The section and subsections are modified by deleting the reference to functional requirements and separation criteria. Physical descriptions are retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and there are no functional or operational requirements for Emergency Power systems.

Section	Title	Change	Description of Change
8.5.3	Design Evaluation	Modify	This section is modified to delete reference to containment penetrations. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, this section is modified to remove containment penetration design and safety functional requirements.
8.5.3.1	Compliance with Regulatory Guide 1.75	Modify	The text of this section is modified to replace the word "Plant" with the word "Facility."
8.5.3.2	Raceway and Cabling Separation Criteria	Modify	The section is modified to delete reference to cable separation design criteria in its entirety. Physical descriptions and historical design information is retained. Amendment 272 deleted TS Sections 3.3, INSTRUMENTATION, and 3.8, ELECTRICAL POWER SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible, and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any electrical or control systems and GDC 24, Separation of Protection and Control Systems, is no longer applicable to PNP (See 5.1.4.5). Consequently, cable separation is no longer applicable.

Section	Title	Change	Description of Change
8.5.3.3	Raceway and Cabling Fire Barriers	Modify	The section is modified to delete reference to safe shutdown. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and safe shutdown is no longer applicable to PNP.
8.5.3.5	Cable Penetration Rooms Protection Design	Modify	The section is modified to delete reference to safe shutdown. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and safe shutdown is no longer applicable to PNP.
8.5.3.7	Safety-Related Cabling Routing Via Nonsafety-Related Areas	Modify	The section is modified by clarifying routing cables through non-safety areas does not affect safe storage of spent fuel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accident scenarios do not rely on electrical power, therefore a fire in the switchgear room will not impact safe storage of spent fuel.

Section	Title	Change	Description of Change
8.5.3.8	Containment Building Routing Protection	Modify	The section is modified to delete reference to safe shutdown. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and safe shutdown is no longer applicable to PNP.
8.5.3.9	Other Areas Routing Protection	Modify	The section is modified to delete reference to safe shutdown. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and safe shutdown is no longer applicable to PNP.
8.6	Automatic Transfer, Voltage Protection and Load Shedding Controls	Delete	The section is deleted in its entirety. Amendment 272 deleted TS Sections 3.3, INSTRUMENTATION, and 3.8, ELECTRICAL POWER SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible, and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any active protection systems and therefore transfer, voltage protection and load shedding are no longer applicable to PNP.

Section	Title	Change	Description of Change
8.7	Physical Separation, Electrical Isolation and Support Systems	Modify	This section title is revised, and the subsections are modified by removing reference to electrical separation. See 8.7.1
8.7.1	Electrical Isolation (see Figure 8- 1)	Delete	The section is deleted in its entirety. Amendment 272 deleted TS Sections 3.3, INSTRUMENTATION, and 3.8, ELECTRICAL POWER SYSTEMS, reflecting the permanent cessation of operations at PNP and permanent removal of fuel from the PNP reactor vessel. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, core related design basis accidents are no longer possible, and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accidents do not credit any protection systems and therefore GDC 24, Separation of Protection and Control Systems, is no longer applicable to PNP (See 5.1.4.5). Consequently, isolation is no longer applicable.
8.7.2.1	General	Modify	The section is modified by deleting discussion of alternate shutdown. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Since power operations can no longer occur alternate shutdown is no longer applicable.

Section	Title	Change	Description of Change
8.7.2.5	Switchgear Rooms Protection	Modify	The section is modified by clarifying an unmitigated fire in the switchgear rooms would not affect safe storage of spent fuel. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, and the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. These accident scenarios do not rely on electrical power, therefore a fire in the switchgear room will not impact safe storage of spent fuel.
8.7.2.6	Emergency Generators Rooms Protection	Modify	The section is modified by removing reference to transfer of diesel fuel and loss of one diesel generator due to fire. Discussion of fire protection features is retained. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, and core related design basis accidents are no longer possible. There are no functional requirements for diesel generators and therefore no functional requirements for fuel transfer.

Section	Title	Change	Description of Change
8.7.3.1	Ventilation	Modify	The section is modified by removing reference to plant operation, loss of offsite power, and diesel generator HVAC operations. Amendment 272 removed Tech Spec Section 3.8, "ELECTRICAL POWER SYSTEMS," and the facility license no longer contains requirements for electrical power. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur, core related design basis accidents are no longer possible, and loss of offsite power is not applicable. There are no functional requirements for diesel generators and therefore no functional or operational requirements for DG Room HVAC.
8.10	Quality Control	Modify	Reference to Section 7.8.7 is deleted as 7.8.7 has been deleted.
8.0	References	Modify	This section is modified to delete references which have been removed from the text.
Table 8-6	Diesel 1-1 Sequence Start	Delete	See 8.6
Table 8-7	Diesel 1-2 Sequence Start	Delete	See 8.6

Section	Title	Change	Description of Change
9.1.1	Design Basis	Modify	This section is modified by eliminating discussion of SWS safety functional requirements, shutdown and DBA operations, and testing. Operations supporting spent fuel cooling, physical descriptions and other non-safety aspects are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed Tech Spec 3.7.8, Service Water System (SWS), and the facility license no longer contains requirements for the service water system. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit SWS or any active SSCs. Therefore, the safety functional requirements of SWS are no longer relevant.
9.1.2.1	System Description	Modify	This section is modified by eliminating discussion of SWS safety functional requirements, shutdown and DBA operations, and testing. Operations supporting spent fuel cooling, physical descriptions and other non-safety aspects are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed Tech Spec 3.7.8, Service Water System (SWS), and the facility license no longer contains requirements for the service water system. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit SWS or any active SSCs. Therefore, the safety functional requirements of SWS are no longer relevant.

Section	Title	Change	Description of Change
9.1.2.3	System Operation	Modify	This section is modified by eliminating discussion of SWS safety functional requirements, shutdown and DBA operations, and testing. Operations supporting spent fuel cooling, physical descriptions and other non-safety aspects are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed Tech Spec 3.7.8, Service Water System (SWS), and the facility license no longer contains requirements for the service water system. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit SWS or any active SSCs. Therefore, the safety functional requirements of SWS are no longer relevant.
9.1.3	Design Analysis	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed Tech Spec 3.7.8, Service Water System (SWS), and the facility license no longer contains requirements for the service water system. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit SWS or any active SSCs. Therefore, the safety functional requirements of SWS are no longer relevant.

Section	Title	Change	Description of Change
9.2.1	Design Basis	Modify	This section is modified by eliminating discussion of functional requirements, operations, and safety margins. Physical descriptions are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur and there are no temperature gradients or thermal stresses on the biological shield. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. Therefore, the functional requirements and operation of the shield cooling system are no longer relevant.
9.2.2.1	System Description	Modify	This section is modified by eliminating discussion of functional requirements, operations, and safety margins. Physical descriptions are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur and there are no temperature gradients or thermal stresses on the biological shield. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. Therefore, the functional requirements and operation of the shield cooling system are no longer relevant.

Section	Title	Change	Description of Change
9.2.2.2	Component Description	Modify	This section is modified by eliminating discussion of functional requirements, operations, and safety margins. Physical descriptions are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur and there are no temperature gradients or thermal stresses on the biological shield. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. Therefore, the functional requirements and operation of the shield cooling system are no longer relevant.
9.2.2.3	System Operation	Delete	This section is modified by eliminating discussion of functional requirements, operations, and safety margins. Physical descriptions are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur and there are no temperature gradients or thermal stresses on the biological shield. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. Therefore, the functional requirements and operation of the shield cooling system are no longer relevant.

Section	Title	Change	Description of Change
9.2.3	Design analysis	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur and there are no temperature gradients or thermal stresses on the biological shield. Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. Therefore, the functional requirements and operation of the shield cooling system are no longer relevant.
9.3.1	Design Basis	Modify	This section is modified by eliminating discussion of CCW safety functional requirements, shutdown and DBA operations, and testing. Operation requirements supporting spent fuel cooling, physical descriptions and other non-safety aspects are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed Tech Spec 3.7.7, Component Cooling Water (CCW) System, and the facility license no longer contains requirements for the service water system. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit CCW or any active SSCs. Therefore, the safety functional requirements of CCW are no longer relevant.

Section	Title	Change	Description of Change
9.3.2.1	System Description	Modify	This section is modified by eliminating discussion of CCW safety functional requirements, shutdown and DBA operations, and testing. Operation requirements supporting spent fuel cooling, physical descriptions and other non-safety aspects are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed Tech Spec 3.7.7, Component Cooling Water (CCW) System, and the facility license no longer contains requirements for the service water system. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit CCW or any active SSCs. Therefore, the safety functional requirements of CCW are no longer relevant.
9.3.2.2	Component Description	Modify	This section is modified by eliminating discussion of CCW safety functional requirements, shutdown and DBA operations, and testing. Operation requirements supporting spent fuel cooling, physical descriptions and other non-safety aspects are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed Tech Spec 3.7.7, Component Cooling Water (CCW) System, and the facility license no longer contains requirements for the service water system. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit CCW or any active SSCs. Therefore, the safety functional requirements of CCW are no longer relevant.

Section	Title	Change	Description of Change
9.3.2.3	System Operation	Modify	This section is modified by eliminating discussion of CCW safety functional requirements, shutdown and DBA operations, and testing. Operation requirements supporting spent fuel cooling, physical descriptions and other non-safety aspects are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed Tech Spec 3.7.7, Component Cooling Water (CCW) System, and the facility license no longer contains requirements for the service water system. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit CCW or any active SSCs. Therefore, the safety functional requirements of CCW are no longer relevant.
9.3.3.1	Margins of Safety	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed Tech Spec 3.7.7, Component Cooling Water (CCW) System, and the facility license no longer contains requirements for the service water system. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit CCW or any active SSCs. Therefore, the safety functional requirements of CCW are no longer relevant.

Section	Title	Change	Description of Change
9.3.3.2	Provisions for Testing and Inspection	Modify	This section is modified by eliminating discussion of CCW safety functional requirements, shutdown and DBA operations, and testing. Operation requirements supporting spent fuel cooling, physical descriptions and other non-safety aspects are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 removed Tech Spec 3.7.7, Component Cooling Water (CCW) System, and the facility license no longer contains requirements for the service water system. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit CCW or any active SSCs. Therefore, the safety functional requirements of CCW are no longer relevant.
9.4.2.3	System Operation	Modify	This section is modified by eliminating discussion of Shutdown and post accident operations. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, shutdown and post accident operation are no longer relevant.
9.4.3.2	Provisions for Testing.	Delete	Section is deleted in its entirety. Amendment 272 removed the TS definition of inservice testing and associated references in Section 3.0. Pump testing in accordance with 10 CFR 50.55a is not required in the permanently defueled condition.

Section	Title	Change	Description of Change
9.5.1.3	Component Description	Modify	This section is modified by eliminating discussion of containment isolation Amendment 272 removed Tech Spec Section 3.6, "Containment Systems," and the facility license no longer contains requirements for the containment building and associated systems. The reactor containment no longer performs an isolation function in the permanently shut down and defueled condition, nor is it required to perform an active function following any of the remaining accidents. Therefore, this FSAR section is modified to remove safety functional requirements for containment isolation valves.
9.5.1.4	System Operation	Modify	This section is modified by eliminating discussion of shutdown and post accident operations. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, shutdown and post accident operation of instrument air are no longer relevant.

Section	Title	Change	Description of Change
9.5.1.5	Design analysis	Modify	This section is modified by eliminating discussion of failure analysis. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, failure analyses of instrument air are no longer relevant.
9.5.2.1	Design Basis	Modify	This section is modified by eliminating accident conditions, safety functional requirements and margins, and discussion of shutdown and post DBA operation. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, there are no safety functional requirements or margins, and discussion of shutdown and post DBA operation is no longer relevant.

Section	Title	Change	Description of Change
9.5.2.2	System Description	Modify	This section is modified by eliminating accident conditions, safety functional requirements and margins, and discussion of shutdown and post DBA operation. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, there are no safety functional requirements or margins, and discussion of shutdown and post DBA operation is no longer relevant.
9.5.2.4	System Operation	Modify	This section is modified by eliminating accident conditions, safety functional requirements and margins, and discussion of shutdown and post DBA operation. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, there are no safety functional requirements or margins, and discussion of shutdown and post DBA operation is no longer relevant.

Section	Title	Change	Description of Change
9.5.2.5	Design Analysis	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, there are no safety functional requirements or margins, and discussion of shutdown and post DBA operation is no longer relevant.
9.5.3.1	Design Basis	Modify	This section is modified by eliminating discussion of accident conditions, beyond- design-basis events, station blackout and functional requirements. Physical descriptions are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, there are no functional requirements, no credit taken, for backup nitrogen.

Section	Title	Change	Description of Change
9.5.3.4	System Operation	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, there are no functional requirements, no credit taken, for backup nitrogen.
9.5.3.5	Design Analysis	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, there are no functional requirements, no credit taken, for backup nitrogen.
9.5.4.4	System Operation	Modify	This section is modified to remove reference to emergency diesels. Amendment 272 removed Tech Spec Section 3.8, Electrical, and the facility license no longer contains requirements for electrical systems. Therefore, discussion is revised to remove references to diesel generators.

Section	Title	Change	Description of Change
9.6.1	Design Basis	Modify	This section is revised by changing reference to "achieving safe and stable conditions" to "maintaining safe and stable conditions." Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. With no DBA accidents, the facility will already be in a safe and stable condition should a fire be detected.
9.6.3	Design Analysis	Modify	This section is revised by changing reference to "achieving safe and stable conditions" to "maintaining safe and stable conditions." Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. With no DBA accidents, the facility will already be in a safe and stable condition should a fire be detected.

Section	Title	Change	Description of Change
9.7.1	Design Basis	Modify	This section is modified by eliminating functional requirements and operational aspects of auxiliary feedwater systems. Physical descriptions are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and Amendment 272 removed Tech Spec 3.7.5, Auxiliary Feedwater (AFW) System. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, functional requirements and operational aspects of auxiliary feedwater systems are no longer relevant.
9.7.2.1	System Description	Modify	This section is modified by eliminating functional requirements and operational aspects of auxiliary feedwater systems. Physical descriptions are retained. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and Amendment 272 removed Tech Spec 3.7.5, Auxiliary Feedwater (AFW) System. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, functional requirements and operational aspects of auxiliary feedwater systems are no longer relevant.

Section	Title	Change	Description of Change
9.7.2.3	System Operation	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and Amendment 272 removed Tech Spec 3.7.5, Auxiliary Feedwater (AFW) System. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, functional requirements and operational aspects of auxiliary feedwater systems are no longer relevant.
9.7.3	Design Analysis	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and Amendment 272 removed Tech Spec 3.7.5, Auxiliary Feedwater (AFW) System. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, functional requirements and operational aspects of auxiliary feedwater systems are no longer relevant.

Section	Title	Change	Description of Change
9.7.4	System Reliability	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and Amendment 272 removed Tech Spec 3.7.5, Auxiliary Feedwater (AFW) System. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, functional requirements and operational aspects of auxiliary feedwater systems are no longer relevant.
9.7.5	Tests and Inspection	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and Amendment 272 removed Tech Spec 3.7.5, Auxiliary Feedwater (AFW) System. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, functional requirements and operational aspects of auxiliary feedwater systems are no longer relevant.

Section	Title	Change	Description of Change
9.8.1	Design basis	Modify	This section is modified by deleting discussion of non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV and other essential HVAC systems, plant operational and emergency modes, and cold shutdown. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, DBAs are no longer possible, and Amendment 272 removed Tech Spec 3.7.10 Control Room Ventilation (CRV) Filtration, 3.7.11 Control Room Ventilation (CRV) Cooling, 3.7.12 Fuel Handling Area Ventilation System, and 3.7.13 Engineered Safeguards Room Ventilation (ESRV) Dampers. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit HVAC. Therefore, non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV, plant operational and emergency modes, and cold shutdown are no longer relevant.

Section	Title	Change	Description of Change
9.8.2.1	System Description	Modify	Section is revised to: Remove reference to DBA, normal, cold and hot shutdown conditions. Remove reference to PCS leakage detection (See 4.7). Remove reference to containment air and engineered safeguards room cooler operations. Remove reference to SSCs as "essential" coolers This section is modified by deleting discussion of non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV and other essential HVAC systems, plant operational and emergency modes, and cold shutdown. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, DBAs are no longer possible, and Amendment 272 removed Tech Spec 3.7.10 Control Room Ventilation (CRV) Filtration, 3.7.11 Control Room Ventilation (CRV) Cooling, 3.7.12 Fuel Handling Area Ventilation System, and 3.7.13 Engineered Safeguards Room Ventilation (ESRV) Dampers. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit HVAC. Therefore, non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV, plant operational and emergency modes, and cold shutdown are no longer relevant.

Section	Title	Change	Description of Change
9.8.2.2	Component Description	Modify	Section is revised to remove reference to emergency operation This section is modified by deleting discussion of non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV and other essential HVAC systems, plant operational and emergency modes, and cold shutdown. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, DBAs are no longer possible, and Amendment 272 removed Tech Spec 3.7.10 Control Room Ventilation (CRV) Filtration, 3.7.11 Control Room Ventilation (CRV) Cooling, 3.7.12 Fuel Handling Area Ventilation System, and 3.7.13 Engineered Safeguards Room Ventilation (ESRV) Dampers. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit HVAC. Therefore, non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV, plant operational and emergency modes, and cold shutdown are no longer relevant.

Section	Title	Change	Description of Change
9.8.2.4	Operation	Modify	Section is revised to: Remove reference to CR HVAC emergency modes. Remove cooling requirements for RPS. Remove operational information on engineered safeguards room coolers. This section is modified by deleting discussion of non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV and other essential HVAC systems, plant operational and emergency modes, and cold shutdown. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, DBAs are no longer possible, and Amendment 272 removed Tech Spec 3.7.10 Control Room Ventilation (CRV) Filtration, 3.7.11 Control Room Ventilation (CRV) Cooling, 3.7.12 Fuel Handling Area Ventilation System, and 3.7.13 Engineered Safeguards Room Ventilation (ESRV) Dampers. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit HVAC. Therefore, non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV, plant operational and emergency modes, and cold shutdown are no longer relevant.

Section	Title	Change	Description of Change
9.8.3	Tests and Inspections	Modify	This section is modified by deleting reference to Technical Specification testing requirements. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, DBAs are no longer possible, and Amendment 272 removed Tech Spec 3.7.10 Control Room Ventilation (CRV) Filtration, and 3.7.12 Fuel Handling Area Ventilation System.
9.8.4	Loss of instrument air to ventilation dampers	Modify	This section is modified by deleting discussion of non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV and other essential HVAC systems, plant operational and emergency modes, and cold shutdown. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, DBAs are no longer possible, and Amendment 272 removed Tech Spec 3.7.10 Control Room Ventilation (CRV) Filtration, 3.7.11 Control Room Ventilation (CRV) Cooling, 3.7.12 Fuel Handling Area Ventilation System, and 3.7.13 Engineered Safeguards Room Ventilation (ESRV) Dampers. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit HVAC. Therefore, non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV, plant operational and emergency modes, and cold shutdown are no longer relevant.

Section	Title	Change	Description of Change
9.8.5.1	Introduction	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and Amendment 272 removed Tech Spec 3.7.5, Auxiliary Feedwater (AFW) System. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit any active SSCs. Therefore, functional requirements and operational aspects of auxiliary feedwater systems are no longer relevant.

Section	Title	Change	Description of Change
9.8.5.2	Evaluation	Modify	This section is modified by deleting discussion of non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV and other essential HVAC systems, plant operational and emergency modes, and cold shutdown. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, DBAs are no longer possible, and Amendment 272 removed Tech Spec 3.7.10 Control Room Ventilation (CRV) Filtration, 3.7.11 Control Room Ventilation (CRV) Cooling, 3.7.12 Fuel Handling Area Ventilation System, and 3.7.13 Engineered Safeguards Room Ventilation (ESRV) Dampers. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, which do not credit HVAC. Therefore, non-FHA accident scenarios, safety functional requirements of CR HVAC, safety functional requirements and operation of ESRV, plant operational and emergency modes, and cold shutdown are no longer relevant.
9.9.1	Design Basis	Modify	This section is modified by eliminating discussions of plant operation, DBAs, and post accident sampling. Physical descriptions of SSCs are retained. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, sampling for plant operation, DBAs and post accident conditions is no longer relevant.

Section	Title	Change	Description of Change
9.9.2	System Description and Operation	Modify	This section is modified by eliminating discussions of plant operation, DBAs, and post accident sampling. Physical descriptions of SSCs are retained. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, sampling for plant operation, DBAs and post accident conditions is no longer relevant.
9.10.2.2	Volume Control	Modify	The section is revised to remove discussion of pressurizer level control. See 4.3.7
9.10.2.4	Reactivity Control	Delete	This section is deleted in its entirety. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and Amendment 272 deleted Tech Spec Section 3.1, REACTIVITY CONTROL SYSTEMS. Therefore, discussion of reactivity control is no longer relevant.

Section	Title	Change	Description of Change
9.10.2.5	Pressure Leakage Test System	Delete	This section is deleted in its entirety. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and Amendment 272 deleted Tech Spec Section 3.4 Primary Coolant System (PCS), and 3.4.13, PCS Operational Leakage. Therefore, discussion of PCS leak testing while the plant is at power and hydrostatically testing the PCS at design pressure is no longer relevant.
9.10.3	Operations	Delete	This section is deleted in its entirety. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, discussion of CVC system operation in normal or emergency conditions is no longer relevant.
9.10.4	Design Analysis	Modify	This section is revised to delete discussion of diesel generators operating under emergency conditions. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible, and Amendment 272 deleted Tech Spec Section 3.8, ELECTRICAL POWER SYSTEMS. Therefore, discussion of EDGs operating under emergency conditions is no longer relevant.

Section	Title	Change	Description of Change
9.10.5	Testing and Inspection	Delete	This section is deleted in its entirety. Amendment 272 removed the TS definition of inservice testing and associated references in Section 3.0. Inservice testing in accordance with 10 CFR 50.55a is not required in the permanently defueled condition.
9.11.2	New Fuel Storage	Modify	This section is revised to delete discussion of new fuel handling. Physical descriptions are retained. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, new fuel will no longer be received or handled.
9.11.3.1	Original Design	Modify	Section was modified to change tense for fuel transfer activities from present ("is") to past tense ("was"). Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, fuel transfer to/from containment is longer allowed.
9.11.3.4	Prevention of Criticality During Transfer and Storage	Modify	Section title is modified to "Prevention of Criticality During Storage." Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, fuel transfer to/from containment is longer allowed.

Section	Title	Change	Description of Change
9.11.3.5.1	Radiation Shielding	Modify	Section was modified to delete reference to refueling. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, fuel transfer to/from containment is longer allowed.
9.11.3.5.3	Airborne Doses	Modify	Editorial - "UFSAR" revised to "DSAR"
9.11.3.5.4	General Area Doses	Modify	Editorial - "UFSAR" revised to "DSAR"
9.11.3.5.5	Protection Against Radioactivity Release	Modify	Section was modified to delete reference to refueling. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, fuel transfer to/from containment is longer allowed.
9.11.4.1	General	Modify	Section is revised to delete reference to new fuel handling, refueling and fuel transfer. "UFSAR" is revised to "DSAR." Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, no new fuel will be handled, and fuel transfer to/from containment is longer allowed.

Section	Title	Change	Description of Change
9.11.4.2	Fuel Handling Structures	Modify	Section is revised to delete reference to refueling activities. Physical descriptions of SSCs is retained. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, refueling activities are longer allowed.
9.11.4.3	Major Fuel Handling Equipment	Modify	Section is revised to delete reference to refueling activities. Physical descriptions of equipment is retained. Spent fuel handling and crane information is retained. "UFSAR" is revised to "DSAR." Reference numbering is corrected. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, refueling activities are longer allowed.
9.11.4.4	System Evaluation	Modify	Section is revised to delete reference to refueling activities and limit discussion to fuel handing in the SFP. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, refueling activities are longer allowed.
9.11.4.5	Test Program	Delete	Section is deleted in its entirety. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Therefore, refueling activities are longer allowed.

Section	Title	Change	Description of Change
9.11.5	Spent Fuel Storage At An Independent Spent Fuel Storage Installation	Modify	Editorial – Corrected References.
9.12.1	Design Basis	Modify	This section is modified by eliminating discussions of plant operation, shutdown and DBAs. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 deleted Tech Spec 3.7.9 Ultimate Heat Sink (UHS). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, discussion of plant operation, DBAs and post accident conditions is no longer relevant.
9.12.2.1	System Description	Modify	This section is modified by eliminating discussions of cooling tower operation, plant operation, shutdown and DBAs. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 deleted Tech Spec 3.7.9 Ultimate Heat Sink (UHS). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, discussion of plant operation, DBAs and post accident conditions is no longer relevant.

Section	Title	Change	Description of Change
9.12.2.2	System Operation	Modify	This section is modified by eliminating discussions plant operation, shutdown and DBAs. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 deleted Tech Spec 3.7.9 Ultimate Heat Sink (UHS). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. Therefore, discussion of plant operation, DBAs and post accident conditions is no longer relevant.
9.12.3	Design Analysis	Delete	This section is deleted in its entirety. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Amendment 272 deleted Tech Spec 3.7.9 Ultimate Heat Sink (UHS). Thus, power operations and electrical generation can no longer occur, core related design basis accidents are no longer possible, and the UHS is not credited in any safety analyses. Therefore, there are no safety functional requirements or safety margins, no required testing or inspections, and hot shutdown is no longer relevant.
9.0	References	Modify	Revised list of references to denote deleted references.
Table 9-1	Service Water System Flow Requirements	Modify	Table is modified to remove operational and DBA flow values. See 9.1.1

Section	Title	Change	Description of Change
Table 9-2	Service Water System Design Ratings and Construction of Components	Modify	Table is modified to remove design values and revise title. Physical descriptions are retained. See 9.1.1
Table 9-3	Reactor Primary Shield Cooling System Design Ratings and Construction of Components	Modify	Table is modified to remove design values and revise title. Physical descriptions are retained. See 9.2.1
Table 9-4	Component Cooling System Heat Loads	Modify	Table is modified to remove shutdown cooling and DBA heat loads. See 9.3.1
Table 9-5	Component Cooling Water System Design Ratings and Construction of Components	Modify	Table is modified to remove operational and design values. Physical descriptions are retained. See 9.3.1
Table 9-6	Component Cooling System Required Flow Rates	Modify	Table is modified to remove operational and DBA flow values. See 9.3.1
Table 9-8	Instrument Air System Design Ratings and Construction of Components	Modify	Deleted reference to Nitrogen Bottle design pressure. See 9.5.3.1

Section	Title	Change	Description of Change
Table 9-9	Effect of Loss of Air to Air-Operated Valves	Modify	Table is modified to remove position information for primary, secondary, and engineered safeguards systems which are no longer in service. Information for SWS, CCW and radwaste systems are retained. Amendment 272 deleted Tech Spec Section 3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS), Section 3.6 CONTAINMENT SYSTEMS, and LCO 3.7.5 Auxiliary Feedwater (AFW) System. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. The only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, none of which credit active components. Therefore, there are no credited "Safety Positions" for air-operated valves and discussion of loss of air position is no longer relevant.
Table 9-12	Auxiliary Feedwater System Design Ratings and Construction of Components	Modify	Table is modified to remove design values and revise title. Physical descriptions are retained. See 9.7.1
Table 9-13	Design Basis Ambient Conditions	Modify	Table is modified to remove accident conditions. See 9.8.1

Section	Title	Change	Description of Change
Table 9-15	Ventilation Dampers: Functions and Positions for Various Modes of Plant Operation	Modify	Table is modified to remove shutdown and DBA auto actuation positions. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and core related design basis accidents are no longer possible. The only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids, none of which credit active components. Therefore, shutdown and DBA auto actuation positions are no longer relevant.
Table 9-17	Sample Point Summary	Modify	Table is revised to remove sample points for NSSS and Steam Cycles. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, and sampling for NSSS and Steam Cycles are no longer relevant.
Table 9-18	Chemical and Volume Control System Design Parameters	Modify	Table is modified to remove operational values. See 9.10.3
Table 9-19	Fuel Handling Data	Modify	Table is modified to remove reference to refueling. See 9.10.3

Section	Title	Change	Description of Change
Figure 9-3	Nominal Operating Conditions Cooling Coil Centerline	Delete	Figure is deleted in its entirety. See 9.2.2.3.
Figure 9-4	Nominal Operating Conditions Mid- Span of Two Cooling Coils	Delete	Figure is deleted in its entirety. See 9.2.2.3.
Figure 9-5	Bounding Design Cooling Coil Centerline	Delete	Figure is deleted in its entirety. See 9.2.2.3.
Figure 9-6	Bounding Design Conditions and Mid- Span of Two Cooling Coils	Delete	Figure is deleted in its entirety. See 9.2.2.3.
Figure 9-19	Boron Concentration vs Core Lifetime	Delete	Figure is deleted in its entirety. See 9.10.2.4.
10.1	Design Basis	Modify	This Chapter is modified to remove functional requirements and operational information. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore, functional requirements and operational information for the steam and power conversion systems is no longer pertinent.

Section	Title	Change	Description of Change
10.2.1	System General Description	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
10.2.2	System Turbine	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
10.2.2.1	High-Pressure Turbine	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
10.2.2.2	Low-Pressure Turbine	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
10.2.2.3	Electrical Generator	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
10.2.2.4	Exciter	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
10.2.3.1	Condensate System	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
10.2.3.3	Feedwater Regulating System	Delete	This section is deleted in its entirety. See 10.1

Section	Title	Change	Description of Change
10.2.4	Circulating Water System	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
10.2.4.1	Cooling Towers	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
10.2.4.2	Makeup and Blowdown	Delete	This section is deleted in its entirety. See 10.1
10.2.4.3	Dilution	Delete	This section is deleted in its entirety. See 10.1
10.2.5	Codes and Standards	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
10.3	System Analysis	Delete	This section is deleted in its entirety. See 10.1
10.4	Tests and Inspections	Modify	This Section is modified to remove functional requirements and operational information. See 10.1
Table 10-3	Signal System Operating Conditions	Delete	Table is deleted. See 10.1

Section	Title	Change	Description of Change
11.1	Source Terms	Modify	This section is modified by eliminating discussions of coolant activity during plant operation. Editorial: Reference to Table 11-10 is corrected to Table 11-17. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, plant operation can no longer occur and tube leaks are no longer pertinent.
11.2.2.1	Clean Waste Section	Modify	This section is modified by eliminating discussions of plant operations. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, plant startup and shutdowns can no longer occur.
11.2.3.1	Clean Waste Section	Modify	This section is modified by eliminating discussions of plant operations and refueling. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, plant operation and refueling can no longer occur.
11.2.5	System Evaluation	Modify	This section is modified by eliminating discussions of steam generator tube leakage. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, plant operation can no longer occur and tube leaks are no longer pertinent.

Section	Title	Change	Description of Change
11.4.2.2	Radioactive Waste Storage Facilities	Modify	This section is modified by correcting errors and making administrative changes.
11.4.2.3	Alternate Radwaste Processing Systems Standard Dewatering System	Modify	This section is modified by correcting errors and making administrative changes.
11.5.1	Design Basis	Modify	This section is modified by making administrative changes.
11.5.4	System Evaluation	Modify	This section is modified to remove functional requirements for containment high range gamma monitors. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.7, Post Accident Monitoring (PAM) Instrumentation, and since the certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, the 10 CFR Part 50 license no longer contains requirements for the RG 1.97 instrumentation.
11.6.1.1	Radiation Exposure of Personnel	Modify	This section is modified by replacing "Maximum Hypothetical Accident (MHA)" with "Chapter 14 accidents." RFOL Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore the MHA is no longer possible.

Section	Title	Change	Description of Change
11.6.1.2	Radiation Exposure of Materials and Components	Modify	This section is modified by deleting reference to "Maximum Hypothetical Accident (MHA)." Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore the MHA is no longer possible.
11.6.3	General Design Considerations	Modify	This section is modified by making administrative changes.
11.6.3.1	Specific Design Values	Modify	This section is modified by making administrative changes.
11.6.4.1	Containment Building Shell	Modify	This section is modified by eliminating reference to plant operations and the maximum hypothetical accident. Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, plant operations and the maximum hypothetical accident are no longer relevant.

Section	Title	Change	Description of Change
11.6.4.2	Containment Building Interior	Modify	This section is modified by removing reference to dose rates incurred during operation. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore operational dose rates are no longer applicable.
11.6.4.3	Auxiliary Building (Including Radwaste Building Addition)	Modify	This section is modified by eliminating discussions of plant operations and DBAs. Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore plant operations and DBAs are no longer relevant.

Section	Title	Change	Description of Change
11.6.4.4	Turbine Building	Modify	This section is modified by eliminating discussions of plant operations and DBAs. Amendment 272 removed accident scenarios related to the NSSS. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids. Therefore, plant operations and DBAs are no longer relevant.
11.6.4.5	General Facility Yard Areas	Modify	This section is modified by eliminating discussions of plant operations. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore plant operations is no longer relevant.
11.6.5.2	System Description	Modify	This section is modified to remove functional requirements for containment high range gamma monitors and reference to NUREG 737. Amendment 272 removed Tech Spec Section 3.3, Instrumentation, including TS 3.3.7, Post Accident Monitoring (PAM) Instrumentation, and since the certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, the 10 CFR Part 50 license no longer contains requirements for the RG 1.97 instrumentation.

Section	Title	Change	Description of Change
11.6.6.1	Facilities	Modify	This section is modified by eliminating discussions of primary coolant samples. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore, primary coolant sampling is no longer relevant.
11.6.6.5	Shielding	Modify	This section is modified by eliminating discussions of plant operations. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur.
11.6.6.6	Access Control	Modify	This section is modified by changing reference to primary thermoluminescence dosemeter to just primary dosimeters and administrative changes related to program management. This is a administrative change to more accurately describe radiation protection practices.
11.6.6.10	External Radiation Dose Determination	Modify	This section is modified by changing reference to primary thermoluminescence dosemeter to just primary dosimeters. This is a administrative change to more accurately describe radiation protection practices.
11.6.7.4	Personal Monitoring Instrumentation	Modify	This section is modified by changing reference to primary thermoluminescence dosemeter to just primary dosimeters. This is a administrative change to more accurately describe radiation protection practices.

Section	Title	Change	Description of Change
11.6.8.1	Shielding	Modify	This section is modified to remove reference to surveys conducted during plant operations. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2).
11.6.8.4	Radiation Protection Instrumentation	Modify	This section is modified by changing reference to primary thermoluminescence dosemeter to just primary dosimeters. This is an administrative change to more accurately describe radiation protection practices.
Table 11-1	Primary Coolant Fission and Corrosion Product Activities	Modify	This table is modified to remove data on primary coolant activity during normal operations. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, normal primary coolant activites levels are no longer applicable.
Table 11-2	Radioactive Waste Quantities of Significant Activity	Modify	This table is modified to remove data on radioactive waste generated during startup, operation, shutdown or refueling. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, there are no radioactive wastes from startup, operation, shutdown or refueling.

Section	Title	Change	Description of Change
Table 11-11	Activity in Gaseous Waste	Modify	This table is modified to remove data on primary coolant activity during normal operations. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, normal primary coolant activities levels are no longer applicable.
12.2	Training	Modify	Removed reference to plant operations. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur.
12.2.1.1	Plant Access Training	Modify	Removed reference to plant operations. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations can no longer occur.
12.2.1.3	Licensed and Nonlicensed Operator Training Program	Modify	Revised title to Certified Fuel Handler Training Program in accordance with Amendment 266 and replaced Operator training programs with Fuel Handler training
12.3.1	Procedure Control Requirements	Modify	Revised procedure scope in safe storage of spent fuel and deleted reference to refueling. Deleted reference to SFCP in accordance with PDTS.

Section	Title	Change	Description of Change
12.3.2	Upgrade and Maintenance of Emergency Operating Procedures	Delete	EOPs have been removed in accordance with Amendment 266.
12.3.3	Operating Requirements Manual (ORM)	Modify	Deleted reference to SFCP in accordance with PDTS.
Table 12-1	Entergy Nuclear Site Leadership	Modify	Corrected name of Table
Table 12-2	Plant Specific Titles for Generic Titles Located in the Technical Specifications	Delete	Deleted Table 12-2 as titles in section 5.1 and 5.2 have been revised in Amend 266 to match Plant Specific Title and therefore the need for an "equivalent title" table is no longer necessary.
Figure 12-1	Entergy Nuclear Site Leadership	Modify	Figure is revised to correct title and revise "GM, Plant Operations" to "Plant Manager"
Figure 12-2	Plant Specific Titles for Generic Titles Located in the Technical Specifications	Delete	Table is deleted. Functions described in the table are obsolete

Section	Title	Change	Description of Change
13.1	Tests Prior to Reactor Fueling	Modify	Section modified to remove reference to preop tests on systems which can no longer operate. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore, preops for primary, steam and power conversion systems are no longer pertinent.
13.2	Reactor Fueling and Physics Tests	Delete	Section deleted in its entirety. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, refueling and physics test are longer pertinent.
13.3	Post Criticality and Power Escalation	Delete	Section deleted in its entirety. Certifications have been submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur. Therefore, criticality and power escalation tests are no longer pertinent.
14.1	Introduction	Modify	This section is modified to replace the word "plant" with the word "facility." The references for this section are revised to add the calculation for the spent fuel cask drop accident. EA-CDA-98-01, Revision 1, "Offsite Radiological Dose Consequences of a Cask Drop in the Spent Fuel Pool," February 7, 2001, (4931/1906) is inserted Reference 24 in support of Table 14.1-4.

Section	Title	Change	Description of Change
14.1.1	Background	Delete	The chronology and discussion of PNP licensing and analysis activities prior to issues of Amendment 272 have no bearing on the post defueled facility and are deleted.
14.1.2	Analyses at Nominal Power Level of 2,650 MWt	Delete	After Amendment 272 is implemented there are no analyses required for 2,650 MWt. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.1.3	Analyses Performed at 2,580.6 MWT Including Uncertainty	Delete	After Amendment 272 is implemented there are no analyses required for 2,580 MWt. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.2	Uncontrolled Control Rod Withdrawal	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
14.3	Boron Dilution	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.4	Control Rod Drop	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.5	Core Barrel Failure	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
14.6	Control Rod Misoperation	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.7	Decreased Reactor Coolant Flow	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.8	Start-up of An Inactive Loop	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
14.10	Increase in Steam Flow (Excess Load)	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.11.1	Event Description	Modify	Section is modified by changing "plant" to "facility."
14.11.2.1.1	Analysis of Cask Drop Scenarios	Modify	This section is modified to delete the cask drop scenarios with 30 day decayed fuel. The cask drop scenario uses 90 day decayed fuel. The 30 day scenarios are no longer credible considering new License Condition C.(5), which prohibits cask movement when irradiated fuel assemblies decayed less than 90 days are in the spent fuel pool.
14.11.3.1.1	Radiological Consequences of a Cask Drop in the Spent Fuel Pool.	Modify	This section is modified to remove reference to previous Cask Drop Scenarios crediting FHB charcoal filters. In the NRC SER for Amendment 272, the only remaining cask drop scenario is a cask drop onto 90 day decayed fuel without the FHB Charcoal Filter operating.
14.11.3.2.1	Radiological Consequences of a Cask Drop in the Spent Fuel Pool.	Modify	This section is modified to remove reference to previous Cask Drop Scenarios crediting FHB charcoal filters. In the NRC SER for Amendment 272, the only remaining cask drop scenario is a cask drop onto 90 day decayed fuel without the FHB Charcoal Filter operating
14.11.3.3.1	Radiological Consequences of a Cask Drop in the Spent Fuel Pool.	Modify	This section is modified to remove reference to previous Cask Drop Scenarios crediting FHB charcoal filters. In the NRC SER for Amendment 272, the only remaining cask drop scenario is a cask drop onto 90 day decayed fuel without the FHB Charcoal Filter operating

Section	Title	Change	Description of Change
14.11	References	Modify	Reference 11 is removed as it is no longer applicable.
14.12	Loss of External Load	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.13	Loss of Normal Feedwater	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.14	Steam Line Rupture Incident	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
14.15	Steam Generator Tube Rupture With a Loss of Offsite Power	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.16	Control Rod Ejection	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.17	Loss of Coolant Accident	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
14.18	Containment Pressure and Temperature Analysis	Delete	This section is deleted in its entirety. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.19.1	Event Description	Modify	This section is modified to remove discussion of accidents during in containment refueling activities. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After permanent shutdown and full core offload, all fuel will be in the spent fuel pool (SFP) or the Independent Spent Fuel Storage Installation (ISFSI).
14.19.3.2	Bounding Event Input	Modify	This section is modified to define the bounding FHA event as 17 days after shutdown in the spent fuel pool. "Plant" is also revised to "facility." The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Since the certifications required by 10 CFR 50.82(a)(1) have been docketed the irradiated fuel assembly decay time requirement of 17 days, established in the analysis of the fuel handling accident in the fuel handling building, has been met.

Section	Title	Change	Description of Change
14.19.3.3	Analysis of Results	Modify	This section is modified to delete reference to a fuel handling accident inside containment and to delete reference to the control room charcoal filters. The licensing basis FHA identified in Amendment 272 takes place in the spent fuel pool and does not credit charcoal filtration. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement `of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The approved license amendment will become effective once the certifications required by 10 CFR 50.82(a)(1) have been docketed, at which time the described FHA becomes effective.
14.19.4	Conclusions	Modify	This section is modified to delete reference to a fuel handling accident inside containment. The licensing basis FHA identified in Amendment 272 takes place in the spent fuel pool and does not credit charcoal filtration. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement `of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). The approved license amendment will become effective once the certifications required by 10 CFR 50.82(a)(1) have been docketed, at which time the described FHA becomes effective.
14.20.1	Event Description	Modify	Deleted reference to the VCT rupture. The VCT rupture scenario is deleted in Amendment 272.
14.20.3.1	Analysis Method	Modify	Deleted reference to the VCT rupture. The VCT rupture scenario is deleted in Amendment 272.

Section	Title	Change	Description of Change
14.20.4	Conclusions	Modify	"Plant" is revised to "Facility"
14.21	Waste Gas Incident	Modify	Calculation No. EA-EC89582-01, Revision 0, "Post-Permanent Shutdown Fuel Handling Accident and Waste Gas Tank Rupture," dated April 26, 2021 is added at end of section as Reference 3
14.21.1.3.1	Analysis Method	Modify	Description changed from "waste gas accident" to "waste gas incident"
14.21.1.3.2	Bounding Event Input	Modify	"Plant" is revised to "Facility"
14.21.1.3.3	Analysis of Results	Modify	This section is revised to reflect analysis presented in Amendment 272 dated June 1, 2021. The Gas Decay Tank Rupture (GDTR) consequences are bounded by the FHA.
14.21.2	Volume Control Tank Rupture	Delete	This section is deleted in its entirety. The VCT rupture scenario was deleted in Amendment 272.
14.21.3	Conclusions	Modify	This section is revised to reflect analysis presented in Amendment 272 dated June 1, 2021. The GDTR consequences are bounded by the FHA.
14.22	Maximum Hypothetical Accident	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
14.23	Radiological Consequences of Failure of Small Lines Carrying Primary Coolant Outside Containment	Delete	This section is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
14.24	Control Room Radiological Habitability	Delete	This section provides the basis for determining required start times for the CRHVAC during accident scenarios. Per Amendment 272, CRHVAC is no longer credited in any accident analysis.
Table 14.1-1	Specified Acceptable Fuel Design Limits	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Fuel will only be located in the SFP and, therefore, fuel design limits are no longer pertinent.
Table 14.1-3	Trip Setpoints for Analysis of Palisades Reactor at 2,530 MWt	Delete	This table is deleted in it's entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur, related design basis accidents are no longer possible, and trip setpoints are no longer relevant.

Section	Title	Change	Description of Change
Table 14.1-4	Disposition of Events Summary for Palisades Cycle 21	Modify	This table is modified by eliminating the discussions of core related design basis accidents and changing the title to Disposition of Events Summary for Palisades Post-Permanent Shutdown. FHA, cask drop and accidents involving the potential release of gaseous wastes or radioactive liquids are retained and the consequences updated per Amendment 272. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Table 14.1-5	Cycle 21 Summary of Results for Standard Review Plan Chapter 15 Events	Delete	This table is deleted in its entirety. The results of the remaining accidents, cask drop, FHA and accidents involving the potential release of gaseous wastes or radioactive liquids, are addressed in Table 14.1-6. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
Table 14.1-6	Summary of Radiological Consequences of the Chapter 14 Events	Modify	This table is modified by eliminating the discussions of core related design basis accidents, and modifying the FHA scenario to 17 day decay times, and modifying the spent fuel cask drop to 90 day decay times in accordance with License Condition C.(5). Accidents involving the potential release of gaseous wastes or radioactive liquids are retained and the consequences updated per Amendment 272. Certifications for permanent cessation of operations and permanent removal of fuel from the reactor vessel were submitted to the NRC in accordance with 10 CFR 50.82(a)(1)(i) and (ii) and are docketed for PNP, therefore the 10 CFR Part 50 license no longer permits operation of the reactor or placement of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Table 14.2.1-1	Event Summary for the Uncontrolled Bank Withdrawal from a Low Power Event	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Table 14.2.2-1	Event Summary for the Uncontrolled Rod Bank Withdrawal Event from Power	Delete	See Table 14.2.1-1

Section	Title	Change	Description of Change
Table 14.7.1-1	Event Summary for The Loss of Forced Reactor Coolant Flow	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Table 14.7.2-1	Event Summary for the Reactor Coolant Pump Rotor Seizure	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
Table 14.10-1	Sequence of Events for Excess Load Limiting MDNBR Case	Delete	This Table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
Table 14.11-3	Spent Fuel Cask Drop Radiological Analysis – Inputs and Assumptions	Modify	This table is modified to reflect the remaining cask drop scenario of 90 days. Per Amendment 272, control room ventilation and filtration are no longer credited and the 30 day scenarios are no longer credible. Table references are corrected to reflect the DSAR.

Section	Title	Change	Description of Change
Table 14.12-1	Event Summary for Loss of Load	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Table 14.13-1	Initial Conditions for the Loss of Normal Feedwater Analysis	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Table 14.13-2	Sequence of Events for Loss of Normal Feedwater Flow Analysis with Offsite Power Available and Steam Dump System Disabled	Delete	See Table 14.13-1

Section	Title	Change	Description of Change
Table 14.13-3	Sequence of Events for Loss of Normal Feedwater Flow Analysis with Offsite Power Available and Steam Dump System Available	Delete	See Table 14.13-1
Table 14.13-4	Sequence of Events for Loss of Normal Feedwater Flow Analysis without Offsite Power Available and Steam Dump System Disabled	Delete	See Table 14.13-1
Table 14.13-5	Results Summary for Loss of Normal Feedwater	Delete	See Table 14.13-1
Table 14.14-1	Main Steam Line Break Input Parameters and Assumptions	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
Table 14.14-2	Steam Line Break Sequence of Events During LHR-Limiting Transient (HZP, Offsite Power Available)	Delete	See Table 14.14-1
Table 14.14-3	Overall Core Conditions at Time of Peak LHR	Delete	See Table 14.14-1
Table 14.14-4	Steam Line Break Sequence of Events During DNBR-Limiting Transient (HZP, Loss of Offsite Power)	Delete	See Table 14.14-1
Table 14.14-5	Overall Core Conditions at Time of MDNBR	Delete	See Table 14.14-1
Table 14.14-6	Main Steam Line Break (MSLB) Radiological Analysis – Inputs and Assumptions	Delete	See Table 14.14-1

Section	Title	Change	Description of Change
Table 14.14-7	Main Steam Line Break (MSLB) Radiological Analysis – Intact SG Steam Release Rate	Delete	See Table 14.14-1
Table 14.15-1	Initial Conditions for The Steam Generator Tube Rupture with a Loss of Offsite Power	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Table 14.15-2	Setpoints for The Steam Generator Tube Rupture with a Loss of Offsite Power	Delete	See Table 14.15-1
Table 14.15-3	Sequence of Events for The Steam Generator Tube Rupture with a Loss of Offsite Power	Delete	See Table 14.15-1

Section	Title	Change	Description of Change
Table 14.15-4	Integrated Parameters for The Steam Generator Tube Rupture with a Loss of Offsite Power	Delete	See Table 14.15-1
Table 14.15-5	Steam Generator Tube Rupture (SGTR) Radiological Analysis – Inputs and Assumptions	Delete	See Table 14.15-1
Table 14.15-6	SGTR Radiological Analysis – Integrated Mass Releases	Delete	See Table 14.15-1
Table 14.15-7	SGTR Radiological Analysis – Flashing Fraction for Flow from Broken Tube	Delete	See Table 14.15-1
Table 14.18-8	SGTR Radiological Analysis – 40 µCI/GM D.E. I-131 Activities	Delete	See Table 14.15-1. Note typo in UFSAR TOC: 14.18 in lieu of 14.16.

Section	Title	Change	Description of Change
Table 14.15.9	SGTR Radiological Analysis – Iodine Equilibrium Appearance Assumptions	Delete	See Table 14.15-1
Table 14.15-10	SGTR Radiological Analysis – Concurrent (335 X) Iodine Spike Appearance Rate	Delete	See Table 14.15-1
Table 14.15-11	SGTR Radiological Analysis – Affected Steam Generator Water Level and Decontamination Factors for Flashed Flow	Delete	See Table 14.15-1
Table 14.16-1	Event Summary for The EOC HZP Control Rod Ejection	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
Table 14.16-2	Control Rod Ejection Radiological Analysis – Inputs and Assumptions	Delete	See Table 14.16-1
Table 14.16-3	Control Rod Ejection Radiological Analysis – Steam Release	Delete	See Table 14.16-1
Table 14.17.1-1	Sampled LBLOCA Parameters	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop, and the potential release of gaseous wastes or radioactive liquids.
Table 14.17.1-2	Plant Operating Range Supported by the LOCA Analysis	Delete	See Table 14.17.1-1
Table 14.17.1-3	Statistical Distribution Used for Process Parameters	Delete	See Table 14.17.1-1
Table 14.17.1-4	Summary of Major Parameters for the Limiting PCT Case	Delete	See Table 14.17.1-1

Section	Title	Change	Description of Change
Table 14.17.1-5	Summary of Hot Rod Limiting PCT Results	Delete	See Table 14.17.1-1
Table 14.17.1-6	Calculated Event Times for The Limiting PCT Case	Delete	See Table 14.17.1-1
Table 14.17.1-7	Containment Heat Sink Data	Delete	See Table 14.17.1-1
Table 14.17.1-8	Containment Initial and Boundary Conditions	Delete	See Table 14.17.1-1
Table 14.17.2-1	System Parameters and Initial Conditions used in The Palisades SBLOCA Analysis	Delete	See Table 14.17.1-1
Table 14.17.2-2	PCT Results of The Palisades SBLOCA Analysis	Delete	See Table 14.17.1-1
Table 14.17.2-3	Sequence of Events for The Palisades SBLOCA Event	Delete	See Table 14.17.1-1
Table 14.17.2-4	SBLOCA Analysis Calculation Results	Delete	See Table 14.17.1-1

Section	Title	Change	Description of Change
Table 14.17.3-1	Maximum Stresses, Pressures and Deflections in Critical Reactor Internals Following a Major Loss of Coolant Accident	Delete	See Table 14.17.1-1
Table 14.17.3-2	Asymmetric Loads Analysis - Reactor Vessel Internal Component Stress Margins	Delete	See Table 14.17.1-1
Table 14.18.1-1	LOCA Analysis Containment Building Heat Sinks/Sources	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Table 14.18.1-2	LOCA Analysis Engineered Safeguards Equipment Alignment	Delete	See Table 14.18.1-1
Table 14.18.1-3	LOCA Initial Conditions	Delete	See Table 14.18.1-1

Section	Title	Change	Description of Change
Table 14.18.1-4	Containment Building Response to LOCA Double Ended Guillotine Break in a Hot Leg	Delete	See Table 14.18.1-1
Table 14.18.1-5	LOCA Analysis Parameter Assumptions	Delete	See Table 14.18.1-1
Table 14.18.2-1	Initial Conditions for The MSLB Containment Analysis	Delete	See Table 14.18.1-1
Table 14.18.2-2	Initial Conditions for The MSLB Containment Analysis	Delete	See Table 14.18.1-1
Table 14.18.2-3	MSLB Containment Analysis Results	Delete	See Table 14.18.1-1
Table 14.18.2-4	MSLB Analysis Containment Building Heat Sinks/Sources	Delete	See Table 14.18.1-1
Table 14.18.3-1	Reactor Cavity Geometric Factors	Delete	See Table 14.18.1-1

Section	Title	Change	Description of Change
Table 14.18.3-2	Geometry and Peak Pressures in Steam Generator Compartments	Delete	See Table 14.18.1-1
Table 14.18.3-3	Differential Pressures at Various Locations	Delete	See Table 14.18.1-1
Table 14.19-1	Fuel Handling Accident (FHA) Radiological Analysis – Inputs and Assumptions	Modify	The table is revised to reflect the analysis described in Amendment 272; 17day delay in fuel movement, 22.6 water level above spent fuel, no control room or FHB filtration, and revised dispersion factors.
Table 14.19-2	Fuel Handling Accident Radiological Analysis – Source Term	Modify	Table is replaced with new Table, POST-PERMANENT SHUTDOWN AST FHA - SOURCE TERM. The new source term values are those provided in Amendment 272 accident analysis.
Table 14.22-1	MHA Sequence of Events for The Dose Consequence Analysis	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, power operations and electrical generation can no longer occur and core related design basis accidents are no longer possible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
Table 14.22-2	Maximum Hypothetical Accident / Loss of Coolant Accident (MHA/LOCA) Radiological Analysis – Inputs and Assumptions	Delete	See Table 14.22-1
Table 14.22-3	MHA/LOCA Source Term	Delete	See Table 14.22-1
Table 14.22-4	MHA/LOCA Release Phases	Delete	See Table 14.22-1
Table 14.22-5	MHA/LOCA Time Dependent SIRWT pH	Delete	See Table 14.22-1
Table 14.22-6	MHA/LOCA Time Dependent SIRWT Total Iodine Concentration	Delete	See Table 14.22-1
Table 14.22-7	MHA/LOCA Time Dependent SIRWT Liquid Temperature	Delete	See Table 14.22-1
Table 14.22-8	MHA/LOCA Time Dependent SIRWT Elemental Iodine Fraction	Delete	See Table 14.22-1

Section	Title	Change	Description of Change
Table 14.22-9	MHA/LOCA Time Dependent SIRWT Partition Coefficient	Delete	See Table 14.22-1
Table 14.22-10	MHA/LOCA Adjusted Release Rate from SIRWT	Delete	See Table 14.22-1
Table 14.23-1	Small Line Break Outside of Containment Radiological Analysis – Inputs and Assumptions	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Table 14.23-2	Small Line Break Outside of Containment Radiological Analysis – Concurrent (500 X) Iodine Spike Appearance Rate	Delete	This table is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Table 14.24-1	Time Dependent Control Room Parameters	Delete	This table is deleted in its entirety. The control room emergency HVAC was deleted from the PDTS, no credit is taken for filtration in any remaining Chapter 14 accident, and there are no time dependent parameters in any remaining analyses.

Section	Title	Change	Description of Change
Table 14.24-2	Control Room Atmospheric Dispersion (X/Q) Factors for AST Analysis Events	Delete	This table is deleted in its entirety. The atmospheric dispersion factors used in the current accident analyses were provided in Amendment 272 and are contained in Table 14.19-4.
Table 14.24-3	Release-Receptor Point Pairs Assumed for AST Analysis Events	Delete	This table is deleted in its entirety. The atmospheric dispersion factors used in the current accident analyses were provided in Amendment 272 and are contained in Table 14.19-4.
Figure 14.1-3	Palisades Scram Curve	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Fuel will only be located in the SFP and, therefore, fuel design limits are no longer pertinent.
Figure 14.2.1-1	Control Rod Withdrawal Incident HZP Reactivity Insertion Curve	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Figure 14.2.1-2	Control Rod Withdrawal Incident HZP Reactivity Feedbacks	Delete	See Figure 14.2.1-1

Section	Title	Change	Description of Change
Figure 14.2.1-3	Control Rod Withdrawal Incident HZP Total Reactivity	Delete	See Figure 14.2.1-1
Figure 14.2.1-4	Control Rod Withdrawal Incident HZP Power and Heat Flux	Delete	See Figure 14.2.1-1
Figure 14.2.1-5	Control Rod Withdrawal Incident HZP System Pressure	Delete	See Figure 14.2.1-1
Figure 14.2.1-6	Control Rod Withdrawal Incident HZP Inlet Enthalpy	Delete	See Figure 14.2.1-1
Figure 14.2.2-1	Reactivities for Uncontrolled Bank Withdrawal at Full Power	Delete	See Figure 14.2.1-1
Figure 14.2.2-2	Reactor Power Level for Uncontrolled Bank Withdrawal Full Power	Delete	See Figure 14.2.1-1

Section	Title	Change	Description of Change
Figure 14.2.2-3	Core Average Heat Flux for Uncontrolled Bank Withdrawal at Full Power	Delete	See Figure 14.2.1-1
Figure 14.2.2-4	Pressurizer Pressure for Uncontrolled Bank Withdrawal at Full Power	Delete	See Figure 14.2.1-1
Figure 14.2.2-5	Pressurizer Liquid Level for Uncontrolled Bank Withdrawal at Full Power	Delete	See Figure 14.2.1-1
Figure 14.2.2-6	PCS Mass Flow Rate for Uncontrolled Bank Withdrawal at Full Power	Delete	See Figure 14.2.1-1
Figure 14.2.2-7	PCS Temperatures for Uncontrolled Bank Withdrawal at Full Power	Delete	See Figure 14.2.1-1
Figure 14.2.2-8	Secondary Pressure for Uncontrolled Bank Withdrawal at Full Power	Delete	See Figure 14.2.1-1

Section	Title	Change	Description of Change
Figure 14.2.2-9	S/G Liquid Level for Uncontrolled Bank Withdrawal at Full Power	Delete	See Figure 14.2.1-1
Figure 14.7.1-1	Primary Coolant System Mass Flow Rate for Loss of Forced Flow	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Figure 14.7.1-2	Reactor Power Level for Loss of Forced Flow	Delete	See Figure 14.7.1-1
Figure 14.7.1-3	Core Average Heat Flux for Loss of Forced Flow	Delete	See Figure 14.7.1-1
Figure 14.7.1-4	Pressurizer Pressure for Loss of Forced Flow	Delete	See Figure 14.7.1-1
Figure 14.7.1-5	Primary Coolant System Temperatures for Loss of Forced Flow	Delete	See Figure 14.7.1-1

Section	Title	Change	Description of Change
Figure 14.7.2-1	Primary Coolant System Mass Flow Rate for Reactor Coolant Pump Rotor Seizure	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Figure 14.7.2-2	Reactor Power Level for Reactor Coolant Pump Rotor Seizure	Delete	See Figure 14.7.2-1
Figure 14.7.2-3	Core Average Heat Flux for Reactor Coolant Pump Rotor Seizure	Delete	See Figure 14.7.2-1
Figure 14.7.2-4	Pressurizer Pressure for Reactor Coolant Pump Rotor Seizure	Delete	See Figure 14.7.2-1
Figure 14.7.2-5	Primary Coolant System Temperatures for Reactor Coolant Pump Rotor Seizure	Delete	See Figure 14.7.2-1

Section	Title	Change	Description of Change
Figure 14.10-1	Power Comparisons – Excess Load	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Figure 14.10-2	PCS Coolant Temperature - Excess Load	Delete	See Figure 14.10-1
Figure 14.10-3	Pressurizer Pressure - Excess Load	Delete	See Figure 14.10-1
Figure 14.10-4	Pressurizer Collapsed Liquid Level - Excess Load	Delete	See Figure 14.10-1
Figure 14.10-5	Components of Reactivity – Excess Load	Delete	See Figure 14.10-1

Section	Title	Change	Description of Change
Figure 14.12-1	Reactor Power Level for Loss of External Load Event	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Figure 14.12-2	Primary Pressures for Loss of External Load Event	Delete	See Figure 14.12-1
Figure 14.12-3	Pressurizer Liquid Volume for Loss of External Load Event	Delete	See Figure 14.12-1
Figure 14.12-4	Primary Coolant System Temperatures for Loss of External Load Event	Delete	See Figure 14.12-1
Figure 14.12-5	Secondary Pressures for Loss of External Load Event	Delete	See Figure 14.12-1

Section	Title	Change	Description of Change
Figure 14.13-1	Reactor Power, LNFF Analysis with Off-Site Power Available and Steam Dump System Disabled	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Figure 14.13-2	Primary Coolant System Loop Temperatures, LNFF Analysis with Offsite Power Available and Steam Dump System Disabled	Delete	See Figure 14.13-1
Figure 14.13-3	Primary Coolant System Loop Flow, LNFF Analysis with Off-Site Power Available and Steam Dump System Disabled	Delete	See Figure 14.13-1
Figure 14.13-4	Pressurizer Pressure, LNFF Analysis with Off-Site Power Available and Steam Dump System Disabled	Delete	See Figure 14.13-1

Section	Title	Change	Description of Change
Figure 14.13-5	Pressurizer Spray Flow, LNFF Analysis with Off-Site Power Available and Steam Dump System Disabled	Delete	See Figure 14.13-1
Figure 14.13-6	Pressurizer SRV Flow, LNFF Analysis with Off-Site Power Available and Steam Dump System Disabled	Delete	See Figure 14.13-1
Figure 14.13-7	Pressurizer Level, LNFF Analysis with Off-Site Power Available and Steam Dump System Disabled	Delete	See Figure 14.13-1
Figure 14.13-8	SG Auxiliary Feedwater Flow, LNFF Analysis with Off-Site Power Available and Steam Dump System Disabled	Delete	See Figure 14.13-1

Section	Title	Change	Description of Change
Figure 14.13-9	SG Dome Pressure, LNFF Analysis with Off-Site Power Available and Steam Dump System Disabled	Delete	See Figure 14.13-1
Figure 14.13-10	SG Liquid Mass Inventory, LNFF Analysis with Off-Site Power Available and Steam Dump System Disabled	Delete	See Figure 14.13-1
Figure 14.13-11	Reactor Power, LNFF Analysis with Off- Site Power Available and Steam Dump System Available	Delete	See Figure 14.13-1
Figure 14.13-12	Primary Coolant System Loop Temperatures, LNFF Analysis with Off- Site Power Available and Steam Dump System Available	Delete	See Figure 14.13-1

Section	Title	Change	Description of Change
Figure 14.13-13	Primary Coolant System Loop Flow, LNFF Analysis with Off-Site Power Available and Steam Dump System Available	Delete	See Figure 14.13-1
Figure 14.13-14	Pressurizer Pressure, LNFF Analysis with Off-Site Power Available and Steam Dump System Available	Delete	See Figure 14.13-1
Figure 14.13-15	Pressurizer Spray Flow, LNFF Analysis with Off-Site Power Available and Steam Dump System Available	Delete	See Figure 14.13-1
Figure 14.13-16	Pressurizer SRV Flow, LNFF Analysis with Off-Site Power Available and Steam Dump System Available	Delete	See Figure 14.13-1

Section	Title	Change	Description of Change
Figure 14.13-17	Pressurizer Level, LNFF Analysis with Off-Site Power Available and Steam Dump System Available	Delete	See Figure 14.13-1
Figure 14.13-18	SG Auxiliary Feedwater Flow, LNFF Analysis with Off-Site Power Available and Steam Dump System Available	Delete	See Figure 14.13-1
Figure 14.13-19	SG Dome Pressure, LNFF Analysis with Off-Site Power Available and Steam Dump System Available	Delete	See Figure 14.13-1
Figure 14.13-20	SG Liquid Mass Inventory, LNFF Analysis with Off-Site Power Available and Steam Dump System Available	Delete	See Figure 14.13-1

Section	Title	Change	Description of Change
Figure 14.13-21	Reactor Power, LNFF Analysis without Off-Site Power Available and Steam Dump Systems Disabled	Delete	See Figure 14.13-1
Figure 14.13-22	Primary Coolant System Loop Temperatures, LNFF Analysis without Off-Site Power Available and Steam Dump Systems Disabled	Delete	See Figure 14.13-1
Figure 14.13-23	Primary Coolant System Loop Flow, LNFF Analysis without Off-Site Power Available and Steam Dump Systems Disabled	Delete	See Figure 14.13-1
Figure 14.13-24	Pressurizer Pressure, LNFF Analysis without Off-Site Power Available and Steam Dump Systems Disabled	Delete	See Figure 14.13-1

Section	Title	Change	Description of Change
Figure 14.13-25	Pressurizer Level, LNFF Analysis without Off-Site Power Available and Steam Dump Systems Disabled	Delete	See Figure 14.13-1
Figure 14.13-26	SG Auxiliary Feedwater Flow, LNFF Analysis without Off-Site Power Available and Steam Dump Systems Disabled	Delete	See Figure 14.13-1
Figure 14.13-27	SG Dome Pressure, LNFF Analysis without Off-Site Power Available and Steam Dump Systems Disabled	Delete	See Figure 14.13-1
Figure 14.13-28	SG Liquid Mass Inventory, LNFF Analysis without Off-Site Power Available and Steam Dump Systems Disabled	Delete	See Figure 14.13-1

Section	Title	Change	Description of Change
Figure 14.14-1	Break Flow Rates During LHR-Limiting Transient	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Figure 14.14-2	Steam Generator Pressures During LHR-Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-3	Steam Generator Heat Transfer Rates During LHR -Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-4	Steam Generator Secondary-Side Total Fluid Inventories During LHR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-5	Core Inlet Temperatures During LHR - Limiting Transient	Delete	See Figure 14.14-1

Section	Title	Change	Description of Change
Figure 14.14-6	Core Inlet Flow Rates During LHR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-7	Pressurizer Pressure During LHR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-8	Pressurizer liquid level during LHR - limiting transient	Delete	See Figure 14.14-1
Figure 14.14-9	Total HPSI Flow Rate During LHR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-10	Reactivity During LHR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-11	Reactor Power During LHR -Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-12	Break Flow Rates During DNBR - Limiting Transient	Delete	See Figure 14.14-1

Section	Title	Change	Description of Change
Figure 14.14-13	Steam Generator Pressures During DNBR -Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-14	Steam Generator Heat Transfer Rates During DNBR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-15	Steam Generator Secondary-Side Total Fluid Inventories During DNBR Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-16	Core Inlet Temperatures During DNBR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-17	Core Inlet Flow Rates During DNBR - Limiting Transient	Delete	See Figure 14.14-1

Section	Title	Change	Description of Change
Figure 14.14-18	Pressurizer Pressure During DNBR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-19	Pressurizer Liquid Level During DNBR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-20	Total Hpsi Flow Rate During DNBR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-21	Reactivity During DNBR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.14-22	Reactor Power During DNBR - Limiting Transient	Delete	See Figure 14.14-1
Figure 14.15-1	SGTR with LOAC: Core Power vs Time	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
Figure 14.15-2	SGTR with LOAC: Core Coolant Temperatures vs Time	Delete	See Figure 14.15-1
Figure 14.15-3	SGTR with LOAC: Primary Coolant System Pressure vs Time	Delete	See Figure 14.15-1
Figure 14.15-4	SGTR with LOAC: Steam Generator Pressure vs Time	Delete	See Figure 14.15-1
Figure 14.15-5	SGTR with LOAC: Tube Leak Flow Rate vs Time	Delete	See Figure 14.15-1
Figure 14.15-6	SGTR with LOAC: Integrated Tube Leak Flow vs Time	Delete	See Figure 14.15-1
Figure 14.15-7	SGTR with LOAC: Pressurizer Liquid Volume vs Time	Delete	See Figure 14.15-1
Figure 14.15-8	SGTR with LOAC: Affected Steam Generator Safety Valve (MSSV) Flow Rate vs Time	Delete	See Figure 14.15-1

Section	Title	Change	Description of Change
Figure 14.15-9	SGTR with LOAC: Affected Steam Generator Safety Valve (MSSV) Integrated Flow vs Time	Delete	See Figure 14.15-1
Figure 14.15-10	SGTR with LOAC: Steam Generators Liquid Mass vs Time	Delete	See Figure 14.15-1
Figure 14.15-11	SGTR with LOAC: Core Power vs Time	Delete	See Figure 14.15-1
Figure 14.15-12	SGTR with LOAC: Core Coolant Temperatures vs Time	Delete	See Figure 14.15-1
Figure 14.15-13	SGTR with LOAC: Primary Coolant System Pressure vs Time	Delete	See Figure 14.15-1
Figure 14.15-14	SGTR with LOAC: Steam Generators Pressure vs Time	Delete	See Figure 14.15-1
Figure 14.15-15	SGTR with LOAC: Pressurizer Liquid Volume vs Time	Delete	See Figure 14.15-1

Section	Title	Change	Description of Change
Figure 14.15-16	SGTR with LOAC: Tube Leak Flow Rate vs Time	Delete	See Figure 14.15-1
Figure 14.15-17	SGTR Integrated Leak Flow vs Time	Delete	See Figure 14.15-1
Figure 14.15-18	SGTR ADV Flow Rate vs Time	Delete	See Figure 14.15-1
Figure 14.15-19	SGTR Integrated ADV Flow vs Time	Delete	See Figure 14.15-1
Figure 14.15-20	SGTR PCS Subcooling vs Time	Delete	See Figure 14.15-1
Figure 14.15-21	SGTR HPSI Flow Rate vs Time	Delete	See Figure 14.15-1
Figure 14.16-1	Control Rod Ejection, EOC HZP case: Core Power	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.

Section	Title	Change	Description of Change
Figure 14.16-2	Control Rod Ejection, EOC HZP Case: Core Average Heat-Flux-Based LHR	Delete	See Figure 14.16-1
Figure 14.16-3	Control Rod Ejection, EOC HZP Case: Total Core Reactivity	Delete	See Figure 14.16-1
Figure 14.17.1-1	Scatter Plot of Operational Parameters	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Figure 14.17.1-2	PCT Versus PCT Time Scatter Plot From Transient Calculations	Delete	See Figure 14.17.1-1
Figure 14.17.1-3	PCT Versus Break Size Scatter Plot From Transient Calculations	Delete	See Figure 14.17.1-1

Section	Title	Change	Description of Change
Figure 14.17.1-4	Total Oxidation Vs. PCT Scatter Plot from Transient Calculations	Delete	See Figure 14.17.1-1
Figure 14.17.1-5	Maximum Oxidation Versus PCT Scatter Plot from Transient Calculations	Delete	See Figure 14.17.1-1
Figure 14.17.1-6	Peak Cladding Temperature for the Limiting Case	Delete	See Figure 14.17.1-1
Figure 14.17.1-7	Break Flow for the Limiting Case	Delete	See Figure 14.17.1-1
Figure 14.17.1-8	Core Inlet Mass Flux for the Limiting Case	Delete	See Figure 14.17.1-1
Figure 14.17.1-9	Core Outlet Mass Flux for the Limiting Case	Delete	See Figure 14.17.1-1
Figure 14.17.1-10	Void Fraction at PCS Pumps for the Limiting Case	Delete	See Figure 14.17.1-1

Section	Title	Change	Description of Change
Figure 14.17.1-11	ECCS Flows (Includes SIT, HPSI, and LPSI) for the Limiting Case	Delete	See Figure 14.17.1-1
Figure 14.17.1-12	Upper Plenum Pressure for the Limiting Case	Delete	See Figure 14.17.1-1
Figure 14.17.1-13	Collapsed Liquid Level in the Downcomer for the Limiting Case	Delete	See Figure 14.17.1-1
Figure 14.17.1-14	Collapsed Liquid Level in the Lower Plenum for the Limiting Case	Delete	See Figure 14.17.1-1
Figure 14.17.1-15	Collapsed Liquid Level in the Core for the Limiting Case	Delete	See Figure 14.17.1-1
Figure 14.17.1-16	Containment and Loop Pressures for the Limiting Case	Delete	See Figure 14.17.1-1

Section	Title	Change	Description of Change
Figure 14.17.2-1	Break Mass Flow Rate (Limiting Case)	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Figure 14.17.2-2	Primary and Secondary Pressures (Limiting Case)	Delete	See Figure 14.17.2-1
Figure 14.17.2-3	Normalized Reactor Power (Limiting Case)	Delete	See Figure 14.17.2-1
Figure 14.17.2-4	Total HPSI Mass Flow Rate (Limiting Case)	Delete	See Figure 14.17.2-1
Figure 14.17.2-5	Total SIT Mass Flow Rate (Limiting Case)	Delete	See Figure 14.17.2-1
Figure 14.17.2-6	Loop Seal Void Fractions (Limiting Case)	Delete	See Figure 14.17.2-1
Figure 14.17.2-7	Break Void Fraction (Limiting Case)	Delete	See Figure 14.17.2-1

Section	Title	Change	Description of Change
Figure 14.17.2-8	Reactor Vessel and PCS Mass Inventories (Limiting Case)	Delete	See Figure 14.17.2-1
Figure 14.17.2-9	Hot Channel Collapsed Level (Limiting Case)	Delete	See Figure 14.17.2-1
Figure 14.17.2-10	Fluid and Cladding Temperatures (Limiting Case)	Delete	See Figure 14.17.2-1
Figure 14.17.2-11	SG Narrow Range Liquid Levels (Limiting Case)	Delete	See Figure 14.17.2-1
Figure 14.17.2-12	AFW Flow Rates (Limiting Case)	Delete	See Figure 14.17.2-1
Figure 14.17.2-13	Total MSSV Flow (Limiting Case)	Delete	See Figure 14.17.2-1

Section	Title	Change	Description of Change
Figure 14.18.1-1	LOCA Containment Pressure Profile	Delete	This figure is deleted in its entirety. The certifications required by 10 CFR 50.82(a)(1) are docketed for PNP, and therefore the 10 CFR Part 50 license prohibits operation of the reactor or emplacement or retention of fuel in the reactor vessel in accordance with 10 CFR 50.82(a)(2). Thus, this accident scenario is not plausible. After implementation of Amendment 272, the only remaining accidents are the FHA, cask drop and the potential release of gaseous wastes or radioactive liquids.
Figure 14.18.1-2	LOCA Containment Temperature Profile	Delete	See Figure 14.18.1-1
Figure 14.18.2-1	MSLB Containment Response Maximum Pressure Profile	Delete	See Figure 14.18.1-1
Figure 14.18.2-2	MSLB Containment Response Environmental Qualification Profile	Delete	See Figure 14.18.1-1

Enclosure 2 to

HDI PNP 2023-002

Palisades Nuclear Plant

CD-ROM Containing Final (Defueled) Safety Analysis Report (FSAR/DSAR) Revision 36

(Public Version)

Enclosure 3 to

HDI PNP 2023-002

Palisades Nuclear Plant

CD-ROM Containing Final (Defueled) Safety Analysis Report (FSAR/DSAR) Revision 36

(Non-Public Version)

(Security-Related Information, Withhold Under 10 CFR 2.390)