



1717 Wakonade Drive
Welch, MN 55089

October 6, 2022

L-PI-22-040
10 CFR 50.90

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Prairie Island Nuclear Generating Plant Units 1 and 2
Docket Nos. 50-282 and 50-306
Renewed Facility Operating License DPR-42 and DPR-60

Supplement to Application to Revise Technical Specifications to Adopt TSTF-577,
“Revised Frequencies for Steam Generator Tube Inspections”

References: 1) Letter L-PI-22-003 from NSPM to NRC, Application to Revise
Technical Specifications to Adopt TSTF-577, “Revised Frequencies
for Steam Generator Tube Inspections,” dated June 7, 2022

In Reference 1, Northern States Power Company, a Minnesota corporation, doing business as Xcel Energy (hereafter “NSPM”), submitted a license amendment request (LAR) to the Technical Specifications (TS) for the Prairie Island Nuclear Generating Plant (PINGP), Units 1 and 2. The LAR proposed to adopt TSTF-577 to revise the TS related to steam generator tube inspections and reporting based on operating history.

The enclosed updated TS 5.6.7 markups (Enclosure 1 to this letter) and clean TS 5.6.7 pages (Enclosure 2 to this letter) resolve formatting issues with the TS markups and TS clean pages provided with Reference 1 and will align the proposed TS 5.6.7 punctuation and conjunctions with the standard TS 5.6.7 and TSTF-577.

The information in this letter does not alter the evaluations performed in accordance with 10 CFR 50.92 in Reference 1.

NSPM is notifying the State of Minnesota of this supplement by transmitting a copy of this letter and enclosures to the designated State Official.

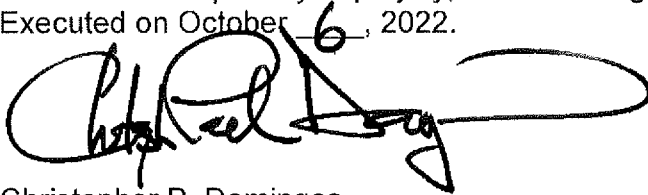
Please contact Mr. Jeff Kivi at (612) 330-5788 or Jeffrey.L.Kivi@xcelenergy.com if there are any questions or if additional information is needed.

Document Control Desk
L-PI-22-040
Page 2

Summary of Commitments

This letter makes no new commitments and no revisions to existing commitments.

I declare under penalty of perjury, that the foregoing is true and correct.
Executed on October 6, 2022.

A handwritten signature in black ink, appearing to read "Christopher P. Domingos", with a large, sweeping flourish extending to the right.

Christopher P. Domingos
Site Vice President, Monticello, and Prairie Island Nuclear Generating Plants
Northern States Power Company – Minnesota

Enclosures

cc: Administrator, Region III, USNRC
Project Manager, Prairie Island, USNRC
Resident Inspector, Prairie Island, USNRC
State of Minnesota

ENCLOSURE 1

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

Supplement to Application to Revise Technical Specifications to Adopt
TSTF-577, "Revised Frequencies for Steam Generator Tube Inspections"

TECHNICAL SPECIFICATION PAGES (Markups)

(2 pages follow)

5.6 Reporting Requirements (continued)

5.6.7 Steam Generator Tube Inspection Report

A report shall be submitted within 180 days after the initial entry into MODE 4 following completion of an inspection performed in accordance with the Specification 5.5.8, Steam Generator (SG) Program. The report shall include:

- a. The scope of inspections performed on each SG;
- ~~b. Degradation mechanisms found;~~
- ~~c. Nondestructive examination techniques utilized for each degradation mechanism;~~
- ~~d. Location, orientation (if linear), and measured sizes (if available) of service induced indications;~~
- ~~e. Number of tubes plugged during the inspection outage for each degradation mechanism;~~
- ~~f. The number and percentage of tubes plugged to date, and the effective plugging percentage in each steam generator, and~~
- ~~g. The results of condition monitoring, including the results of tube pulls and in-situ testing.~~

INSERT B

INSERT B

- b. The nondestructive examination techniques utilized for tubes with increased degradation susceptibility;
- c. For each degradation mechanism found:
 - 1. The nondestructive examination techniques utilized;
 - 2. The location, orientation (if linear), measured size (if available), and voltage response for each indication. For tube wear at support structures less than 20 percent through-wall, only the total number of indications needs to be reported;
 - 3. A description of the condition monitoring assessment and results, including the margin to the tube integrity performance criteria and comparison with the margin predicted to exist at the inspection by the previous forward-looking tube integrity assessment; and
 - 4. The number of tubes plugged during the inspection outage.
- d. An analysis summary of the tube integrity conditions predicted to exist at the next scheduled inspection (the forward-looking tube integrity assessment) relative to the applicable performance criteria, including the analysis methodology, inputs, and results;
- e. The number and percentage of tubes plugged to date, and the effective plugging percentage in each SG; and
- f. The results of any SG secondary side inspections.

ENCLOSURE 2

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

Supplement to Application to Revise Technical Specifications to Adopt
TSTF-577, "Revised Frequencies for Steam Generator Tube Inspections"

TECHNICAL SPECIFICATION PAGES (Re-typed)

(1 page follows)

5.6 Reporting Requirements (continued)

5.6.7 Steam Generator Tube Inspection Report

A report shall be submitted within 180 days after the initial entry into MODE 4 following completion of an inspection performed in accordance with the Specification 5.5.8, “Steam Generator (SG) Program.” The report shall include:

- a. The scope of inspections performed on each SG;
- b. The nondestructive examination techniques utilized for tubes with increased degradation susceptibility;
- c. For each degradation mechanism found:
 1. The nondestructive examination techniques utilized;
 2. The location, orientation (if linear), measured size (if available), and voltage response for each indication. For tube wear at support structures less than 20 percent through-wall, only the total number of indications needs to be reported;
 3. A description of the condition monitoring assessment and results, including the margin to the tube integrity performance criteria and comparison with the margin predicted to exist at the inspection by the previous forward-looking tube integrity assessment; and
 4. The number of tubes plugged during the inspection outage.
- d. An analysis summary of the tube integrity conditions predicted to exist at the next scheduled inspection (the forward-looking tube integrity assessment) relative to the applicable performance criteria, including the analysis methodology, inputs, and results;
- e. The number and percentage of tubes plugged to date, and the effective plugging percentage in each SG; and
- f. The results of any SG secondary side inspections.