



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION II
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200
ATLANTA, GEORGIA 30303-1200

July 29, 2022

Mr. Steven Snider
Site Vice President
Duke Energy Carolinas, LLC
7800 Rochester Highway
Seneca, SC 29672-0752

SUBJECT: OCONEE NUCLEAR STATION – INTEGRATED INSPECTION REPORT
05000269/2022002 AND 05000270/2022002 AND 05000287/2022002

Dear Mr. Snider:

On June 30, 2022, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Oconee Nuclear Station. On July 20, 2022, the NRC inspectors discussed the results of this inspection with you and other members of your staff. The results of this inspection are documented in the enclosed report.

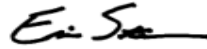
One finding of very low safety significance (Green) is documented in this report. This finding involved a violation of NRC requirements. We are treating this violation as a non-cited violation (NCV) consistent with Section 2.3.2 of the Enforcement Policy.

If you contest the violation or the significance or severity of the violation documented in this inspection report, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region II; the Director, Office of Enforcement; and the NRC Resident Inspector at Oconee Nuclear Station.

If you disagree with a cross-cutting aspect assignment in this report, you should provide a response within 30 days of the date of this inspection report, with the basis for your disagreement, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region II; and the NRC Resident Inspector at Oconee Nuclear Station.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

A handwritten signature in black ink, appearing to read "Eric Stamm".

Signed by Stamm, Eric
on 07/29/22

Eric J. Stamm, Chief
Reactor Projects Branch 1
Division of Reactor Projects

Docket Nos. 05000269 and 05000270 and 05000287
License Nos. DPR-38 and DPR-47 and DPR-55

Enclosure:
As stated

cc w/ encl: Distribution via LISTSERV

SUBJECT: OCONEE NUCLEAR STATION – INTEGRATED INSPECTION REPORT
05000269/2022002 AND 05000270/2022002 AND 05000287/2022002

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OFFICE	RII/DRP	RII/DRP	RII/DRP		
NAME	J. Nadel	D. Jackson	E. Stamm		
DATE	7/28/2022	7/28/2022	07/29/2022		

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U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Numbers: 05000269, 05000270 and 05000287

License Numbers: DPR-38, DPR-47 and DPR-55

Report Numbers: 05000269/2022002, 05000270/2022002 and 05000287/2022002

Enterprise Identifier: I-2022-002-0026

Licensee: Duke Energy Carolinas, LLC

Facility: Oconee Nuclear Station

Location: Seneca, South Carolina

Inspection Dates: April 1, 2022, to June 30, 2022

Inspectors: J. Nadel, Senior Resident Inspector
A. Ruh, Resident Inspector
N. Smalley, Resident Inspector
P. Cooper, Senior Reactor Inspector
J. Diaz-Velez, Senior Health Physicist
S. Downey, Senior Reactor Inspector
A. Goldau, Operations Engineer
M. Magyar, Reactor Inspector
A. Nielsen, Senior Health Physicist
N. Peterka, Senior Project Engineer
L. Pressley, Reliability and Risk Analyst

Approved By: Eric J. Stamm, Chief
Reactor Projects Branch 1
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Oconee Nuclear Station, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

Failure to prepare and deliver a radioactive material shipment for transportation by a carrier in accordance with 10 CFR 71.5 and 49 CFR 173.421			
Cornerstone	Significance	Cross-Cutting Aspect	Report Section
Public Radiation Safety	Green NCV 05000269,05000270,05000287/ 2022002-01 Open/Closed	[H.12] - Avoid Complacency	71124.08
A Green self-revealing, non-cited violation (NCV) of 10 CFR 71.5 was identified for the failure to comply with Department of Transportation (DOT) external surface dose rate limits for a limited quantity package as specified in 49 CFR 173.421.			

Additional Tracking Items

Type	Issue Number	Title	Report Section	Status
URI	05000269,05000270,05000287/2022001-03	Essential Siphon Vacuum Testing	71111.22	Discussed
LER	05000270/2021-005-00	Unit 2, Automatic Reactor Trip Due to Spurious Trip Signal Concurrent with System Testing	71153	Closed
LER	05000270/2022-001-00	Unit 2, Regarding Automatic Reactor Trip Due to Loss of Power to Reactor Coolant Pumps	71153	Closed
LER	05000270/2022-002-00	Unit 2, Regarding Automatic Actuation of Emergency Feedwater System due to Main Feedwater Pump Malfunction	71153	Closed
LER	05000270/2022-003-00	Unit 2, Manual Reactor Trip due to Main Feedwater Control Valve Positioner Malfunction	71153	Closed
LER	05000287/2022-001-00	Unit 3, Response Actions Resulted in Brief Inoperability of Both Onsite and Offsite Emergency AC Power Paths	71153	Closed

PLANT STATUS

Unit 1 operated at or near 100 percent rated thermal power (RTP) for the entire inspection period.

Unit 2 operated at or near 100 percent RTP for the entire inspection period.

Unit 3 began the inspection period operating near 100 percent RTP. On May 6, 2022, the unit was shut down for a scheduled refueling outage. The unit was returned to 100 percent RTP on June 2, 2022, and remained at or near 100 percent RTP for the remainder of the inspection period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors performed activities described in IMC 2515, Appendix D, "Plant Status," observed risk significant activities, and completed on-site portions of IPs. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71111.01 - Adverse Weather Protection

Seasonal Extreme Weather Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated readiness for seasonal extreme weather conditions prior to the onset of seasonal hot temperatures for the following system:
 - 230kV switchyard on June 16, 2022

Impending Severe Weather Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated the adequacy of the overall preparations to protect risk-significant systems from impending severe weather due to a tornado warning issued for the local area on May 23, 2022.

71111.04 - Equipment Alignment

Partial Walkdown Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) Unit 3 low pressure injection system during lowered inventory configuration on May 10, 2022
- (2) Instrument air system during planned maintenance of primary instrument air compressor on April 18 and 19, 2022
- (3) Unit 3 protected service water system following restoration from refueling outage on June 17, 2022

Complete Walkdown Sample (IP Section 03.02) (1 Sample)

- (1) Standby shutdown facility following restoration from Unit 3 refueling outage on June 30, 2022

71111.05 - Fire Protection

Fire Area Walkdown and Inspection Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) Fire zone 89: Unit 3 equipment room on April 6, 2022
- (2) Fire zone 105: Unit 2 cable room on April 7, 2022
- (3) Fire zone 85: Unit 1 second floor hallway on April 27, 2022
- (4) Fire zone 112: Unit 3 control room on April 27, 2022
- (5) Fire zone 124: Unit 3 containment on May 10, 2022

71111.06 - Flood Protection Measures

Inspection Activities - Internal Flooding (IP Section 03.01) (1 Sample)

The inspectors evaluated internal flooding mitigation protections in the:

- (1) Unit 3 east penetration room and risk mitigation actions during breach of low pressure service water system

71111.08P - Inservice Inspection Activities (PWR)

PWR Inservice Inspection Activities Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated pressurized-water reactor non-destructive testing by reviewing the following examinations from May 9 to May 19, 2022:
 1. Ultrasonic Testing (UT)
 - a. 3PSL-133, B Elbow Base Metal, Augmented (reviewed)
 - b. 3FDW-267-12, Reducer to Pipe, Class 1 (reviewed)
 - c. 3-PZR-WP26-3, Nozzle to Shell, Class 1 (reviewed)
 2. Penetrant Testing (PT)
 - a. 3-54A-3-0-2439A-H25, Lug to Pipe Weld, Class 2 (reviewed)
 3. Eddy Current Examination (ECT)
 - a. Steam Generator 3A - ECT for tubes R136C14, R22C77, R77C119, R87C123
 - b. Steam Generator 3B – ECT for tubes R149C7, R143C21, R137C67

The inspectors evaluated the licensee's boric acid corrosion control program performance.

71111.11Q - Licensed Operator Requalification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (1 Sample)

- (1) The inspectors observed and evaluated licensed operator performance in the control room during Unit 3 reactor shutdown on May 6, 2022.

Licensed Operator Requalification Training/Examinations (IP Section 03.02) (1 Sample)

- (1) The inspectors observed and evaluated simulator training for an operating crew using simulator exercise guide SEG 2201 PSE-1(1) on June 21, 2022.

71111.12 - Maintenance Effectiveness

Maintenance Effectiveness (IP Section 03.01) (3 Samples)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

- (1) Primary instrument air compressor with 10 CFR 50.65(a)(1) evaluation from nuclear condition report (NCR) 2386883
- (2) Unit 2 main feedwater system with 10 CFR 50.65(a)(1) evaluation from NCR 2417011
- (3) Unit 3 4kV electrical system with 10 CFR 50.65(a)(1) evaluation from NCR 2413538

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (6 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

- (1) Units 1, 2, and 3 elevated green risk due to planned maintenance on primary instrument air compressor and turbine building heavy lifts, on April 11, 2022
- (2) Units 1, 2, and 3 elevated green risk due to planned maintenance on primary instrument air compressor, on April 18 and 19, 2022
- (3) Units 1, 2, and 3 yellow risk due to a tornado watch, on May 6, 2022
- (4) Unit 3 yellow shutdown risk due to reactor coolant system draining, on May 8, 2022
- (5) Unit 3 yellow shutdown risk due to cold mid-loop operations, on May 22, 2022
- (6) Units 1 and 2 elevated green risk due to heavy lift of feedwater heaters over Units 1 and 2 main feeder buses, on May 13, May 18, and June 20, 2022

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (6 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) NCR 2428395, use of gagging device to maintain 3SF-61 containment isolation valve closed
- (2) NCR 2429587, Unit 3 wide range reactor coolant system 'A' hot leg temperature indication failure for the standby shutdown facility and use of spare element on alternate resistance temperature detector
- (3) NCR 2428753, failure of PT/3/A/0610/029, 4160V bus transfer testing
- (4) NCR 2432531, 3A low pressure service water pump approached high motor stator temperature alarm
- (5) NCR 2426380, correction factor for essential siphon vacuum (ESV) pump seal water temperature variation may not be valid
- (6) NCR 2432379, preliminary evaluation of multi-point data collection for 2A essential siphon vacuum pump

71111.19 - Post-Maintenance Testing

Post-Maintenance Test Sample (IP Section 03.01) (6 Samples)

The inspectors evaluated the following post-maintenance testing activities to verify system operability and/or functionality:

- (1) PT/0/A/0600/021, Standby Shutdown Facility (SSF) Diesel - Generator Operation Test, following preventive maintenance on the SSF diesel engine service water pump, work order (WO) 20453345
- (2) Pressure testing of condenser circulating water pipe patch plate repair of through-wall defect, WO 20395451
- (3) 3CC-8 diagnostic testing following air operated valve actuator rebuild and adjustments, WO 20486686
- (4) PT/3/A/0202/012, Component Test of ES Channels 1 & 2, following Unit 3 refueling outage system maintenance, WO 20476205
- (5) PT/3/A/0400/007, SSF RC Makeup Pump Test, following Unit 3 refueling outage preventive maintenance, WO 20476110
- (6) PT/3/A/0251/024, High Pressure Injection Full Flow Test, following Unit 3 refueling outage system maintenance, WO 20476144

71111.20 - Refueling and Other Outage Activities

Refueling/Other Outage Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated refueling outage U3R31 activities from May 6, 2022, to May 30, 2022.

71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance testing activities to verify system operability and/or functionality:

Surveillance Tests (other) (IP Section 03.01) (3 Samples)

- (1) CP/2/A/2002/001, U2 RCS Sample on April 11, 2022
- (2) PT/3/A/0261/020, ECCW System Test, on May 8, 2022
- (3) PT/0/A/0711/001, Zero Power Physics Testing, on May 29, 2022

Containment Isolation Valve Testing (IP Section 03.01) (1 Sample)

- (1) PT/3/A/0151/019, Penetration 19 Leak Rate Test, on May 24, 2022

71114.06 - Drill Evaluation

Drill/Training Evolution Observation (IP Section 03.02) (1 Sample)

The inspectors evaluated:

- (1) The inspectors observed and evaluated simulator training for an operating crew using simulator exercise guide SEG 2201 PSE-1(1) on June 21, 2022.

RADIATION SAFETY

71124.01 - Radiological Hazard Assessment and Exposure Controls

Radiological Hazard Assessment (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated how the licensee identifies the magnitude and extent of radiation levels and the concentrations and quantities of radioactive materials and how the licensee assesses radiological hazards.

Instructions to Workers (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated how the licensee instructs workers on plant-related radiological hazards and the radiation protection requirements intended to protect workers from those hazards.

Contamination and Radioactive Material Control (IP Section 03.03) (2 Samples)

The inspectors observed/evaluated the following licensee processes for monitoring and controlling contamination and radioactive material:

- (1) Non-fuel items in the Unit 1, Unit 2, and Unit 3 spent fuel pools
- (2) Licensee surveys of potentially contaminated material leaving the radiologically controlled area (RCA)

Radiological Hazards Control and Work Coverage (IP Section 03.04) (3 Samples)

The inspectors evaluated the licensee's control of radiological hazards for the following radiological work:

- (1) Radiation work permit 5061 - Filter liner handling activities - lift and load into shipping cask
- (2) ALARA [as low as reasonably achievable] Plan 2022-ONS-3-O-001, Alloy 600, revision 0
- (3) ALARA Plan 2022-ONS-3-O-002, 3B1 Reactor Coolant Pump Motor Remove/Repair/Replace, revision 0

High Radiation Area and Very High Radiation Area Controls (IP Section 03.05) (3 Samples)

The inspectors evaluated licensee controls of the following High Radiation Areas and Very High Radiation Areas:

- (1) Unit 3 reactor coolant bleed holdup tank room
- (2) Seal supply filter cleaning and storage room
- (3) Incore probes in warehouse 10

Radiation Worker Performance and Radiation Protection Technician Proficiency (IP Section 03.06) (1 Sample)

- (1) The inspectors evaluated radiation worker and radiation protection technician performance as it pertains to radiation protection requirements.

71124.08 - Radioactive Solid Waste Processing & Radioactive Material Handling, Storage, & Transportation

Radioactive Material Storage (IP Section 03.01) (2 Samples)

The inspectors evaluated the licensee's performance in controlling, labeling, and securing radioactive materials the following areas:

- (1) Unit 3 auxiliary building
- (2) Steam generator mausoleum building

Radioactive Waste System Walkdown (IP Section 03.02) (2 Samples)

The inspectors walked down the following accessible portions of the solid radioactive waste systems and evaluated system configuration and functionality:

- (1) Unit 3 liquid waste monitoring tanks
- (2) Unit 3 liquid waste demineralizers skid

Waste Characterization and Classification (IP Section 03.03) (2 Samples)

The inspectors evaluated the following characterization and classification of radioactive waste:

- (1) 10 CFR 61 Dry Active Waste radwaste stream sample #ON21032900028, dated 03/29/2021
- (2) 10 CFR 61 Powdex Resin 19-21 sample #ON21041900006, dated 02/18/2021

Shipment Preparation (IP Section 03.04) (1 Sample)

- (1) The inspectors observed the preparation of radioactive shipment RSR# 22-2005

Shipping Records (IP Section 03.05) (4 Samples)

The inspectors evaluated the following non-excepted radioactive material shipments through a record review:

- (1) Radioactive Waste Shipment 20-2027, Low Specific Activity (LSA) II
- (2) Radioactive Waste Shipment 21-2005, LSA II
- (3) Radioactive Waste Shipment 21-2014, LSA II
- (4) Radioactive Waste Shipment 21-2017, LSA II

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

MS05: Safety System Functional Failures (SSFFs) Sample (IP Section 02.04) (3 Samples)

- (1) Unit 1 (April 1, 2021, through March 31, 2022)
- (2) Unit 2 (April 1, 2021, through March 31, 2022)
- (3) Unit 3 (April 1, 2021, through March 31, 2022)

MS08: Heat Removal Systems (IP Section 02.07) (3 Samples)

- (1) Unit 1 (April 1, 2021, through March 31, 2022)
- (2) Unit 2 (April 1, 2021, through March 31, 2022)
- (3) Unit 3 (April 1, 2021, through March 31, 2022)

MS10: Cooling Water Support Systems (IP Section 02.09) (3 Samples)

- (1) Unit 1 (April 1, 2021, through March 31, 2022)
- (2) Unit 2 (April 1, 2021, through March 31, 2022)
- (3) Unit 3 (April 1, 2021, through March 31, 2022)

BI02: RCS Leak Rate Sample (IP Section 02.11) (3 Samples)

- (1) Unit 1 (April 1, 2021, through March 31, 2022)
- (2) Unit 2 (April 1, 2021, through March 31, 2022)
- (3) Unit 3 (April 1, 2021, through March 31, 2022)

OR01: Occupational Exposure Control Effectiveness Sample (IP Section 02.15) (1 Sample)

- (1) December 4, 2021, through May 5, 2022

PR01: Radiological Effluent Technical Specifications/Offsite Dose Calculation Manual
Radiological Effluent Occurrences (RETS/ODCM) Radiological Effluent Occurrences Sample
(IP Section 02.16) (1 Sample)

- (1) December 4, 2021, through March 30, 2022

71152A - Annual Follow-up Problem Identification and Resolution

Annual Follow-up of Selected Issues (Section 03.03) (2 Samples)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issues:

- (1) Functional tests of 2CS-73 and 3CS-73 did not meet acceptance criteria (NCRs 2417396, 2323805)
- (2) Recurring essential siphon vacuum system float valve test failures and cause evaluations (NCRs 1765682, 1774724, 1816518, 1808705, 2238168, 2247826, 2352558, 2375175, 2386700)

71152S - Semiannual Trend Problem Identification and Resolution

Semiannual Trend Review (Section 03.02) (1 Sample)

- (1) The inspectors reviewed the licensee's corrective action program for potential adverse trends in operations department performance that might be indicative of a more significant safety issue.

71153 - Follow Up of Events and Notices of Enforcement Discretion

Event Report (IP Section 03.02) (5 Samples)

The inspectors evaluated the following licensee event reports (LERs):

- (1) LER 05000270/2021-005-00, Unit 2 Automatic Reactor Trip Due to Spurious Trip Signal Concurrent with Systems Testing (ADAMS Accession No. ML22039A333). The inspectors determined that it was not reasonable to foresee or correct the cause discussed in the LER, therefore, no performance deficiency was identified. The inspectors did not identify a violation of NRC requirements.
- (2) LER 05000270/2022-001-00, Unit 2 Automatic Reactor Trip Due to Loss of Power to Reactor Coolant Pumps (ADAMS Accession No. ML22096A333). The inspection conclusions associated with this LER are documented in Inspection Report 05000269, 05000270, 05000287/2022001 (ADAMS Accession No. ML22123A236) under Inspection Results Section 71153.
- (3) LER 05000270/2022-002-00, Automatic Actuation of Emergency Feedwater System due to Main Feedwater Pump Malfunction (ADAMS Accession No. ML22104A257). The inspection conclusions associated with this LER are documented in Inspection Report 05000269, 05000270, 05000287/2022001 (ADAMS Accession No. ML22123A236) under Inspection Results Section 71153.

- (4) LER 05000270/2022-003-00, Manual Reactor Trip Due to Main Feedwater Control Valve Positioner Malfunction (ADAMS Accession No. ML22111A361). The inspection conclusions associated with this LER are documented in Inspection Report 05000269, 05000270, 05000287/2022001 (ADAMS Accession No. ML22123A236) under Inspection Results Section 71153.
- (5) LER 05000287/2022-001-00, Response Actions Resulted in Brief Inoperability of Both Onsite and Offsite Emergency AC Power Paths (ADAMS Accession No. ML22087A512). The inspectors determined that it was not reasonable to foresee or correct the cause discussed in the LER, therefore, no performance deficiency was identified. The inspectors did not identify a violation of NRC requirements.

Personnel Performance (IP Section 03.03) (1 Sample)

- (1) The inspectors evaluated operator response to a 3A spent fuel pool cooler filter gasket failure and leakage of 6400 gallons of Unit 3 spent fuel pool water into the auxiliary building and licensee's performance on June 3, 2022.

OTHER ACTIVITIES – TEMPORARY INSTRUCTIONS, INFREQUENT AND ABNORMAL

71003 - Post-Approval Site Inspection for License Renewal

The NRC continued monitoring Duke's performance at Oconee by conducting a Post-Approval Site Inspection for License Renewal - Phase 4 in accordance with the license renewal inspection program (LRIP). Per IMC 2516, the LRIP is the process used by NRC staff to verify the adequacy of aging management programs (AMPs) and other activities associated with an applicant's request to renew an operating license of a commercial nuclear power plant beyond the initial licensing period under 10 CFR Part 54, "Requirements for the Renewal of Operating Licenses for Nuclear Power Plants."

The inspectors reviewed the licensee's implementation of the AMPs shown below by selecting a sample of SSCs within the scope of the respective AMPs. The inspectors performed the following activities, as applicable to each SSC, to determine that there is a reasonable assurance that the effects of aging are being adequately managed: walked down all accessible SSCs to observe their general condition and identify any signs of aging-related degradation; interviewed plant personnel; reviewed completed work orders; reviewed applicable monitoring and trending data; and reviewed the acceptability of inspection and test results. For each AMP shown below, the inspectors also reviewed a sample of aging-related issues entered into the licensee's corrective action program to verify that age-related degradation is being identified at an appropriate threshold and corrected.

Post-Approval Site Inspection for License Renewal (13 Samples)

- (1) Crane Inspection Program (UFSAR Section 18.3.5)
 - Unit 1 Polar Crane
 - Turbine Aisle Crane (160 T)
 - Unit 3 Equipment Hatch Hoist
 - Keowee Hydro Unit (KHU) Overhead Crane KLD-1
 - SSF Diesel Generator Room Manual Chain Hoist

- (2) Duke Power Five-Year Underwater Inspection of Hydroelectric Dams and Appurtenances (UFSAR Section 18.3.6)
 - Keowee Intake
 - Spillway
 - Powerhouse
- (3) Elevated Water Storage Tank Civil Inspection (UFSAR Section 18.3.7)
 - Elevated Water Storage Tank (interior)
- (4) Flow Accelerated Corrosion (FAC) Program (UFSAR Section 18.3.9)
 - Oconee Nuclear Station (ONS) FAC Program O1R31 Post Outage Report
 - ONS FAC Program O2R30 Post Outage Report
 - ONS FAC Program O3R30 Post Outage Report
- (5) Boric Acid Corrosion Control Program (UFSAR Section 18.3.10)
 - 3HP-500 (Action Request (AR) 02426830)
 - 1HP-365 (AR 02249989)
 - 3LWD-1138 (AR 01837052)
 - 3HPI-FT-0160 (AR 02045628)
 - 1LPI-PG-0007 (AR 02263770)
- (6) Inspection Program for Civil Engineering Structures and Components (UFSAR Section 18.3.13)
 - Underwater weir
 - Elevated water storage tank (exterior)
 - Tendon galleries
 - Discharge diversion wall (AR 02233964)
 - Standby shutdown facility
 - Trench from ESV building to radioactive waste facility trench
- (7) Insulated Cables and Connections Aging Management Program (UFSAR Section 18.3.14)
 - Accessible cables and connections in the auxiliary building, turbine building, and elevated water storage tank area
 - The following inaccessible or direct buried medium voltage cables: CT5c, CT5f, IPBT0010, IPBT0013, protected service water cables, 2TE11
- (8) Keowee Oil Sampling Program (UFSAR Section 18.3.15)
 - Governor oil system
 - Turbine building bearing oil system
- (9) Borated Water Storage Tank Internal Coatings Inspection (UFSAR Section 18.3.17.1)
 - Unit 1 borated water storage tank (WO 1467528)
 - Unit 2 borated water storage tank (WO 1466034)
 - Unit 3 borated water storage tank (WO 1462557)

- (10) Keowee Turbine Generator Cooling Water System Strainer Inspection (UFSAR Section 18.3.17.10)
- Train 1 Keowee turbine generator cooling water system strainer
 - PT/1/A/2200/002, "KHU-1 - Bi-Monthly Surveillance"
 - PT/1/A/2200/002, "KHU-1 - Semiannual Surveillance"
 - Train 2 Keowee turbine generator cooling water system strainer
 - PT/2/A/2200/002, "KHU-2 - Bi-Monthly Surveillance"
 - PT/2/A/2200/002, "KHU-2 - Semiannual Surveillance"
- (11) Reactor Vessel Internals Inspection Program (UFSAR Section 18.3.20)
- Baffle former bolts (AR 2355692)
 - Lower core barrel bolt (AR 1846141)
 - Upper core barrel bolt
- (12) Service Water Piping Corrosion Program (UFSAR Section 18.3.21)
- Pipe adjacent to 42" valve 2CCW-041 (AR 2371309)
 - Pipe downstream 2" valve HPSW-114 (AR 02371553)
 - Pipe at C1LPS024 (AR 02078087)
 - Pipe at UT location CK1WL004 and CK2WL004 (AR 02080095)
 - Service Water Health Report (AR 01858183)
 - Service Water Trend (AR 01910482)
- (13) Battery Rack Inspections (UFSAR Section 18.3.26)
- SSF battery DCSF
 - SSF battery DCSFS
 - Battery 1CA
 - Battery 3CB
 - Keowee battery 1
 - 230 kV switchyard battery SY-1

INSPECTION RESULTS

URI (Discussed)	Essential Siphon Vacuum Testing URI 05000269,05000270,05000287/2022001-03	71111.22
Description: This unresolved item was opened in Inspection Report 05000269, 05000270, 05000287/2022001. On June 8, 2022, the licensee collected pump performance test data over a larger range of suction pressures on the 2A essential siphon vacuum (ESV) pump. The results indicated that the pump's volumetric capacity declined as vacuum at the pump suction rose above 15 inches of mercury (inHg). Depending on the assumed lake level, calculation OSC-6961 expected the pump to operate at conditions between 21 and 18 inHg, which could amount to an 11 percent to 7 percent capacity reduction as compared to the performance being demonstrated during routine surveillance testing. However, since the pumps take suction from a sub-atmospheric source during a design event (caused by a water siphon in the condenser circulating water (CCW) piping), rather than the atmosphere (as during surveillance testing), choked flow present during testing would not be expected during a design event. The licensee is evaluating the test data for impacts from instrument uncertainties, temperature effects, and the differences between testing and design conditions. In the interim, test acceptance criteria are being adjusted to accommodate the observed capacity reductions until evaluations are complete.		

After this unresolved item was originally opened, inspectors had related questions regarding the formulas in OSC-6961 used to evaluate degassing of air from the lake water flowing into the high point of the CCW siphon. The combined rate of air ingress into the CCW piping through water degassing and general in-leakage through unsealed mechanical clearances is what the required ESV pump capacity is ultimately based on. At reduced lake levels, it appeared that the rate of degassing could be underestimated by up to 56 percent because the methodology reduced the total theoretical air release by a ratio of air densities for vacuum conditions in the CCW pipe to standard atmospheric conditions. It was not immediately clear that use of this ratio was consistent with the conditions being analyzed since the licensee did not have the original reference documents used to derive their methodology. Engineers are validating the formulation but performed a preliminary analysis which concluded that system performance would still remain adequate at the higher degassing rates based on current lake levels, CCW system tightness, and other design margins.

In order to determine whether a performance deficiency exists, the inspectors need additional information concerning whether the pump capacity is expected to be reduced for suction conditions of a design event and whether the current degassing methodology is appropriate.

Failure to prepare and deliver a radioactive material shipment for transportation by a carrier in accordance with 10 CFR 71.5 and 49 CFR 173.421

Cornerstone	Significance	Cross-Cutting Aspect	Report Section
Public Radiation Safety	Green NCV 05000269,05000270,05000287/ 2022002-01 Open/Closed	[H.12] - Avoid Complacency	71124.08

A Green self-revealing, non-cited violation (NCV) of 10 CFR 71.5 was identified for the failure to comply with Department of Transportation (DOT) external surface dose rate limits for a limited quantity package as specified in 49 CFR 173.421.

Description: On November 11, 2020, the licensee was notified about a discrepancy with a radiation survey measurement on shipment number LQ20-1177. The radioactive shipment had been transported as an “excepted package for limited quantities” pursuant to the DOT requirements of 49 CFR 173.421(a)(2) and did not include emergency response information. Upon receipt by the package recipient the contact dose rates on the external surface of the package were found to be greater than the 0.5 millirem per hour (mrem/h) limit allowed by the regulation. The recipient measured a contact radiation level of 0.8 mrem/h. The licensee’s survey conducted before shipment indicated a maximum dose rate on the package of 0.01 mrem/h. The package contained equipment belonging to a contractor and was being shipped to another utility. The licensee’s causal investigation concluded that an inadequate radiation survey was performed prior to the package leaving the site and that the shipment should not have been characterized as limited quantity.

Corrective Actions: Immediate corrective actions included a stand-down with all qualified licensee shippers, temporary inactivation of shipper qualifications, and implementation of a dose rate verification requirement for all shipments involving limited quantities of radioactive materials.

Corrective Action References: Condition report 02357711

Performance Assessment:

Performance Deficiency: The failure to prepare and deliver a radioactive material shipment for transportation in accordance with 10 CFR 71.5 and 49 CFR 173.421 was a performance deficiency.

Screening: The inspectors determined the performance deficiency was more than minor because it was associated with the Program and Process attribute of the Public Radiation Safety cornerstone and adversely affected the cornerstone objective to ensure adequate protection of public health and safety from exposure to radioactive materials released into the public domain as a result of routine civilian nuclear reactor operation. Specifically, the improper package preparation could have led to members of the public being exposed to higher radiation doses than anticipated for a limited quantity package. In addition, the shipment did not include the hazard communications for carriers and emergency responders that would normally be expected for a package exceeding 0.5 mrem/h on the external surface.

Significance: The inspectors assessed the significance of the finding using IMC 0609 Appendix D, "Public Radiation Safety SDP." Although the shipment lacked emergency response information, response efforts would not have been significantly hampered due to the extremely low radiological hazard. Therefore, the inspectors determined that the finding was of Green significance.

Cross-Cutting Aspect: H.12 - Avoid Complacency: Individuals recognize and plan for the possibility of mistakes, latent issues, and inherent risk, even while expecting successful outcomes. Individuals implement appropriate error reduction tools.

Enforcement:

Violation: 10 CFR 71.5(a) requires that a licensee who offers licensed radioactive material for transport comply with the DOT regulations in 49 CFR Parts 107, 171-180, and 390-397. 49 CFR 173.421 excepts limited quantity shipments of radioactive material from certain marking, labeling, and shipping paper requirements, including emergency response information, provided that the radiation level at any point on any external surface of the package does not exceed 0.5 millirem per hour. Contrary to this, on November 9, 2020, the licensee offered a package for transport as an excepted package-limited quantity of material, but the dose rate on the external surface exceeded 0.5 mrem per hour.

Enforcement Action: This violation is being treated as a non-cited violation, consistent with Section 2.3.2 of the Enforcement Policy.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On May 10, 2022, the inspectors presented the Radiation Protection inspection results to Paul Fisk, Plant Manager, and other members of the licensee staff.
- On May 19, 2022, the inspectors presented the Inservice inspection results to Steven Snider and other members of the licensee staff.
- On July 20, 2022, the inspectors presented the integrated inspection results to Steven Snider and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71003	Corrective Action Documents	NCRs (by number)	2242434, 2305360, 2341326, 2375598, 2384556, 2407211	
	Work Orders	Work Orders (by number)	20183682, 20296196, 20296203, 20322254, 20351458	
71111.01	Corrective Action Documents		2396552, 2396652	
	Procedures	AP/0/A/1700/006	Natural Disaster	33
		OP/0/B/1104/050	Weather Related Activities	006
		PT/0/A/0110/018	Hot Weather Protection	009
71111.04	Drawings	OEE-149-01	Pressurizer Heaters Arrangement and Legend	26
		OFD-101A-3.5	Flow Diagram of High Pressure Injection System (SSF Portion)	028
		OFD-102A-3.1	Flow Diagram of Low Pressure Injection System (Borated Water Supply and LPI Pump Suction)	65
		OFD-102A-3.2	Flow Diagram of Low Pressure Injection System (LPI Pump Discharge)	49
		OFD-102A-3.3	Flow Diagram of Low Pressure Injection System (Core Flood)	25
		OFD-104A-3.1	Flow Diagram of Spent Fuel Cooling System	060
		OFD-121D-3.1	Flow Diagram of Emergency Feedwater System	047
		OFD-131A-1.1	Flow Diagram of Protected Service Water System	2
		OFD-131A-3.2	Flow Diagram of Protected Service Water System (Steam Generator & HPI Pump Motor Cooling Service)	2
		OFD-133A-2.5	Flow Diagram of Condenser Circulating Water System (SSF Auxiliary Service)	062
		OFD-137B-1.1	Flow Diagram of Instrument Air System (Air Compressors, After Coolers, Receivers, Air Dryers)	28
	Miscellaneous	PT/3/A/1600/008	SSF Instrument Surveillance	030
	Procedures	AP/0/A/1700/025	Standby Shutdown Facility Emergency Operating Procedure	067
		OP/0/A/1600/002	Standby Shutdown Facility HVAC System Operation	039
		OP/0/A/1600/009	SSF Auxiliary Service Water System	039

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		OP/0/A/1600/010	Operation of the SSD Diesel Generator	098
		OP/0/A/1650/001	PSW System	7
		OP/0/A/1650/005	PSW AC Power	19
		OP/0/A/1650/006	PSW DC Power	11
		OP/2/A/1102/020 D	SSF and Outside Rounds	103
		OP/3/A/1102/027	Administrative Controls of Piping Subject to Possible Overpressurization	008
		OP/3/A/1104/004	Low Pressure Injection System	170
		OP/3/A/1600/008	SSF RC Makeup and Letdown Systems	030
71111.05	Fire Plans	CSD-ONS-PFP-1AB-0783	Pre-Fire Plan for U1 Auxiliary Building Elevation 783	000
		CSD-ONS-PFP-2AB-0809	Pre-Fire Plan for U2 Auxiliary Building Elevation 809	000
		CSD-ONS-PFP-3AB-0796	Pre-Fire Plan for U3 Auxiliary Building Elevation 796	000
		CSD-ONS-PFP-3AB-0822	Pre-Fire Plan for U3 Auxiliary Building Elevation 822	000
		CSD-ONS-PFP-3RB	Pre-Fire Plan for U3 Reactor Building for All Elevations	000
		O-0310-FZ-011	Auxiliary Building – Unit 3 Fire Protection Plan Fire Area and Fire Zone Boundaries Plan at EL 796+6 & EL 797+6	002
		O-310-K-09	Auxiliary and Reactor Building – Unit 3 Fire Protection Plan & Fire Barrier, Flood, & Pressure Boundaries Plan at EL 796+6 & EL 797+6	14
71111.06	Corrective Action Documents		2427343	
	Miscellaneous		Elevated Risk Activity Plan for 3LPS-VA-1116, PM Check Valve During O3R31	03/30/2022
		SD 3.2.16	Control of Passive Design Features	10
	Procedures	MP/0/A/1200/064A	Valve- Crane- Tilting Disc Check- Disassembly, Repair, and Reassembly	23
		MP/3/A/1705/018	Fire Protection- Penetration- Fire and Flood Barrier- Inspection and Minor Repair	46
	Work Orders	20172203-06	3-LPS-VA-1116 Remove Check Valve	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.08P	Miscellaneous		Oconee Unit 3 O3R31 Outage Steam Generator Degradation Assessment	1
			ROTSG Site Technique Validation for Oconee Nuclear Station	5
		3012-AST-102181	Steam Generator Monitoring and Final Operational Assessment for Oconee-3 O3R29 Outage	000
		3012-AST-102562	Two-Cycle Steam Generator Operational Assessment for Oconee Unit 3 Cycles 30 and 31	000
		51-9316606-001	Oconee Mathematically Calculated Calibration Curve	02/27/2018
71111.11Q	Procedures	AP/1/A/1700/002	Excessive RCS Leakage	017
		AP/1/A/1700/014	Loss of Normal HPI Makeup and/or RCP Seal Injection	022
		EP/1/A/1800/001 0L	Unit 1 EOP Rules and Appendix	005
		EP/1/A/1800/001 0M	Unit 1 EOP Enclosures 5.1-5.10	001
		EP/1/A/1800/00100	Unit 1 EOP Immediate Manual Actions and Subsequent Actions	003
		P/1/A/1800/001 0I	Unit 1 EOP LOCA Cooldown	2
71111.12	Corrective Action Documents		2370522, 2412228, 2412234, 2370522, 2426459, 2390606, 2391905, 2394232, 2351882, 2343892, 2344593	
	Procedures	AD-EG-ALL-1210	Maintenance Rule Program	3, 4
71111.13	Miscellaneous		Feedwater Heater Drop Evaluation	04/20/2022
		AP-OP-ALL-0111 Attachment 13	External Department Guidance: Feedwater Heater Replacement Heavy lifts on Turbine Deck – Elev. 822'+0"	0
		Standing Instruction 22-05	Feedwater Heater Replacement Heavy Lifts on Turbine Deck – Elev. 822'+0"	0
	Procedures	AD-WC-ALL-0420	Shutdown Risk Management	7
		AD-WC-ONS-0420	ONS Shutdown Risk Management	2
		OP/3/A/1103/011	Draining and Nitrogen Purging RCS	98
71111.15	Drawings	O-2785-F	Connection Diagram Integrated Control System Misc. Remote Mtd. Equip.	24
		OFD-104A-03-01	Flow Diagram of Spent Fuel Cooling System	59
		OM 245-2241.001	8" (DN200) Bolted Bonnet Gate Valve (Cast)	E
		OM 267-1336.001	Installation/Instruction/Operation Manual for N9031-1A RTD	3

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			and Associated Equipment (For WEED Triple Element RTD)	
	Engineering Changes		421144, 421168	
	Miscellaneous	OSS-0254.00-00-1006	Design Basis Specification for the Spent Fuel Cooling System	32
	Procedures	AP/0/A/1700/025	Standby Shutdown Facility Emergency Operating Procedure	67
		IP/0/A/0200/032	RTD Replacement	60
		IP/0/B/0200/049	Auxiliary Control System Reactor Coolant System Instrument Calibration	48
		IP/3/A/0370/002 B	Unit 3 SSF RC System Temperature	1
		OP/3/A/1104/010	Low Pressure Service Water	118
	Work Orders		20539195, 20538991	
71111.18	Miscellaneous	NEI 14-01	Emergency Response Procedures and Guidelines for Beyond Design Basis Events and Severe Accidents	1
71111.19	Calculations	OSC-10330	AOV Design Basis Capability Value CC-8	2
	Corrective Action Documents		NCR 02428250, 02428547	
	Drawings	OFD-101A-3.1	Flow Diagram of High Pressure Injection System (Letdown Section)	44
		OFD-101A-3.2	Flow Diagram of High Pressure Injection System (Storage Section)	44
		OFD-101A-3.3	Flow Diagram of High Pressure Injection System Charging Section	32
		OFD-101A-3.4	Flow Diagram of High Pressure Injection System Charging Section	48
		OFD-101A-3.5	Flow Diagram of High Pressure Injection System (SSF Portion)	28
		OFD-102A-3.1	Flow Diagram of Low Pressure Injection System (Borated Water Supply and LPI Suction)	65
		OFD-102A-3.2	Flow Diagram of Low Pressure Injection System (LPI Pump Discharge)	49
		OFD-104A-3.1	Flow Diagram of Spent Fuel Cooling System	59

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Engineering Changes		420718	
	Miscellaneous		AOV Post-Test Evaluation for 3CC-8	05/24/2022
			Quiklook Dynamic Scan Report 3CC0008 As-Left	05/24/2022
	Procedures	MP/0/A/1800/097	Accumulator – Suction Stabilizer – Greer Hydraulics – 10 Gallon – Checking, Precharging, Disassembly, and Reassembly	16
		OP/0/A/1600/010	Operation of the SSF Diesel Generator	098
		PT/0/A/0600/020	Standby Shutdown Facility Instrument Surveillance	025
		PT/0/A/0600/021	Standby Shutdown Facility Diesel - Generator Operation	017
		PT/0/A/0600/023	Standby Shutdown Facility Fuel Oil Inventory	007
		PT/3/A/0202/012	Component Test of ES Channels 1 & 2	29
		PT/3/A/0251/024	HPI Full Flow Test	51
		PT/3/A/0400/007	SSF RC Makeup Pump Test	77
	Work Orders		20476261, 20475755	
		20453345 01	Perform Annual Lube (PM) SSF Diesel Service Water Pump	04/27/2022
71111.20	Corrective Action Documents		2425517	
	Miscellaneous		Clearance PRT-3-22-REDUCED INV-0190	
			Clearance OPS-3-22-LPI-2NDDROPLOOPS-0963	
			Elevated Risk Activity Plan for Removal of Unit 3 Main Feeder Bus 2	03/30/2022
	Procedures	AD-OP-ALL-0106	Conduct of Infrequently Performed Tests or Evolutions	4
		AD-WC-ALL-0410	Work Activity Integrated Risk Management	12
		EP/1/A/1800/001 0Q	Unit 1 EOP Enclosures 5.41-5.47	8
		OP/0/A/1107/011 F	Sharing Startup Transformers Between Units	22
		OP/3/A/1102/001	Controlling Procedure For Unit Startup	281
		OP/3/A/1102/004	Operation At Power	140
		OP/3/A/1103/011	Draining and Nitrogen Purging RCS	98
		PT /0/ A/0711/001	Zero Power Physics Test	77
		PT/0/A/0711/002	Zero Power Physics Test Equipment Checkout	014

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.22	Calculations	PT/3/A/0610/025	Electrical System Weekly Surveillance (Unit 3)	23
		OSC-2280	LPSW NPSHA and Minimum Required Lake Level	22
		OSC-5349	Minimum Lake Level Required to Maintain Sufficient NPSH to the LPSW Pumps via Gravity Flow	6
		OSC-5670	Emergency CCW System First Siphon	8
	Corrective Action Documents	OSC-6961	ECCW Siphon Air Inleakage Model ESV System Performance Model and Test Acceptance Criteria	4
			1727606, 02428753	
	Drawings	OFD-110A-2.1	Flow Diagram of Chemical Addition System (Primary Sample Hood)	040
		ONTC-0-133A-0001-003	ECCW Test Acceptance Criteria	7
	Procedures	AD-EG-ALL-1450	Preconditioning of Structures, Systems and Components	0
		AD-WC-ALL-0260	Nuclear Generation Response to High or Low Grid System Load	5
		CP/0/A/2005/022	Determination of Reported Tech Spec Dose Equivalent Iodine-131	007
		CP/2/A/2002/001	Unit 2 Primary Sampling System	062
		PT/0/A/0261/008	Condenser Circulating Water Valve Leak Test	17
		PT/3/A/0610/029	4160V AC Bus Transfer Test	008
	Work Orders		20224034	
71124.01	Corrective Action Documents Resulting from Inspection	CR 02426218		
	Miscellaneous		ONS Alpha Program Information	03/02/2022
	Procedures	AD-RP-ALL-0007	Control of Radioactive Material	2
		AD-RP-ALL-2014	Work in Alpha Environments	8
71124.08	Corrective Action Documents	CR 02357623, 02357711, 02364735, 02373733, 02375797,	Corrective Action Documents	Various

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		02394135, 02402955, and 02412872		
	Shipping Records	LQ #20-1177 and LQ #21-1050	Shipping Documents	
71151	Corrective Action Documents		2432820, 239172, 2367963, 2391434	
	Miscellaneous		MSPI Cooling Water System URI Derivation and Margin Reports, Oconee Units 1, 2, 3 for period March 2022	
			MSPI Heat Removal System URI Derivation and Margin Reports, Oconee Units 1, 2, 3 for period March 2022	
		AD-PI-ALL-0700	Performance Indicators	5
		NEI 99-02	Regulatory Assessment Performance Indicator Guideline	7
		PT/1/A/0600/010	Reactor Coolant Leakage	100
		PT/2/A/0600/010	Reactor Coolant Leakage	80
		PT/3/A/0600/010	Reactor Coolant Leakage	85
71152A	Corrective Action Documents		2323805, 2352685, 2417396, 2207328, 2393005, 2341117, 2352787, 2408955, 2408602, 2424351, 2430822, 2401963, 2410028, 2415853	
	Drawings	O FD-101A-02-02	Flow Diagram of High Pressure Injection System (Storage Section)	046
		O FD-101A-02-03	Flow Diagram of High Pressure Injection System (Charging Section)	034
		O FD-101A-03-02	Flow Diagram of High Pressure Injection System (Storage Section)	044
		O FD-101A-03-03	Flow Diagram of High Pressure Injection System (Charging Section)	032
		O FD-106A-02-01	Flow Diagram of Coolant Treatment System (RC Bleed Holdup Tanks)	017
		O FD-106A-02-02	Flow Diagram of Coolant Treatment System (Concentrated Boric Acid Storage Tank)	013
		O FD-106A-03-01	Flow Diagram of Coolant Treatment System (RC Bleed Holdup Tanks)	016
		O FD-106A-03-02	Flow Diagram of Coolant Treatment System (Concentrated	016

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Boric Acid Storage Tank)	
	Miscellaneous		ASME Inservice Testing Program	28
		SLC 16.5.13	HPI and the Chemical Addition Systems	
	Procedures	AP/2/A/1700/003,	Boron Dilution	017
		PT/2/A/0251/008	2CS-73 Functional Test	024
		PT/3/A/0251/008	3CS-73 Functional Test	020
	Work Orders		20200673, 02132473, 20392734, 20429060, 20429061	
71153	Procedures	AP/3/A/1700/030	Auxiliary Building Flood	21
		AP/3/A/1700/035	Loss of SFP Cooling and/or Level	24
		OP/0/A/1108/001	Curves and General Information	118