



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

May 17, 2022

Mr. James Barstow  
Vice President, Nuclear Regulatory Affairs  
and Support Services  
Tennessee Valley Authority  
1101 Market Street, LP 4A-C  
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNIT 2 – CORRECTION TO AMENDMENT NO. 350 REGARDING ONE-TIME CHANGE TO TECHNICAL SPECIFICATION 3.4.12, “LOW TEMPERATURE OVERPRESSURE PROTECTION SYSTEM,” (EPID L-2021-LLA-0194)

Dear Mr. Barstow:

By letter dated October 27, 2021 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML21298A031), the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 350 to Renewed Facility Operating License (RFOL) No. DPR-79 for the Sequoyah Nuclear Plant, Unit 2. The amendment revised Technical Specification 3.4.12, “Low Temperature Overpressure Protection (LTOP) System,” to add a one-time note to allow operation of one safety injection pump and one charging pump capable of injecting into the reactor coolant system during MODE 5 or MODE 6 with the pressurizer manway cover removed.

An error was identified in this license amendment in that RFOL No. DPR-79 page 3 was inadvertently not included as part of Enclosure 1 to the license amendment.

Enclosed with this letter is RFOL No. DPR-79 page 3 for License Amendment No. 350.

The error does not change any of the conclusions in the safety evaluation associated with the October 27, 2021, Amendment No. 350, nor does it affect the no significant hazard consideration, as published in the *Chattanooga Times Free Press* on October 26, 2021.

If you have any questions, please contact me at (301) 415-1383 or via e-mail at [Perry.Buckberg@nrc.gov](mailto:Perry.Buckberg@nrc.gov).

Sincerely,

**/RA/**

Perry H. Buckberg, Senior Project Manager  
Plant Licensing Branch II-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-328

Enclosure:  
Current Technical Specification Page 3

cc: Listserv

**ENCLOSURE**

CORRECT PAGE 3 OF RENEWED FACILITY OPERATING LICENSE NO. DPR-79

FOR LICENSE AMENDMENT NO. 350

TENNESSEE VALLEY AUTHORITY

SEQUOYAH NUCLEAR PLANT, UNIT 2

DOCKET NO. 50-328

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the Sequoyah and Watts Bar Unit 1 Nuclear Plants.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

- (1) Maximum Power Level  
The Tennessee Valley Authority is authorized to operate the facility at reactor core power levels not in excess of 3455 megawatts thermal.
- (2) Technical Specifications  
The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 350 are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.
- (3) Initial Test Program  
The Tennessee Valley Authority shall conduct the post-fuel-loading initial test program (set forth in Section 14 of Tennessee Valley Authority's Final Safety Analysis Report, as amended), without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:
  - a. Elimination of any test identified in Section 14 of TVA's Final Safety Analysis Report as amended as being essential;

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