

## **Enclosure 4**

### **Westinghouse Closed Session Slide Package for the ACRS Thermal-Hydraulic Phenomena Subcommittee Meeting on WCAP-18482-P/NP**

**(Non-Proprietary)**

**(25 pages including this cover page)**

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# Westinghouse Advanced Doped Pellet Technology (**ADOPT™**) Fuel, WCAP-18482-P/NP

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# Agenda

Introduction to **ADOPT** Fuel

Topical Report Overview

Qualification Data

Analytical Methods

- Fuel Performance
- Nuclear Design
- Thermal-Hydraulic Design
- Safety Analyses

Conclusions

# Introduction to **ADOPT** Fuel

# Introduction to ADOPT Fuel

- **ADOPT** (Advanced DOped Pellet Technology) fuel is a standard  $\text{UO}_2$  fuel that has been doped with  $[\text{ }^{a,c}\text{Cr}_2\text{O}_3]$  and  $[\text{ }^{a,c}\text{Al}_2\text{O}_3]$ .
- The additives facilitate densification and diffusion during sintering resulting in a higher density and enlarged grain size compared to undoped  $\text{UO}_2$ .

[

]<sup>a,c</sup>

# Operating Experience

- **ADOPT** fuel is a commercial product for the European market with extensive operating experience and superior performance
  - ✓ 23 years of irradiation data
  - ✓ 17 years of reloads
  - ✓ [ ]<sup>a,c</sup> burnup
  - ✓ 3400 fuel assemblies
  - ✓ 680 metric tons
- Current efforts are focused on bringing **ADOPT** technology to the US PWR market

Deliveries of ADOPT Fuel in European Market

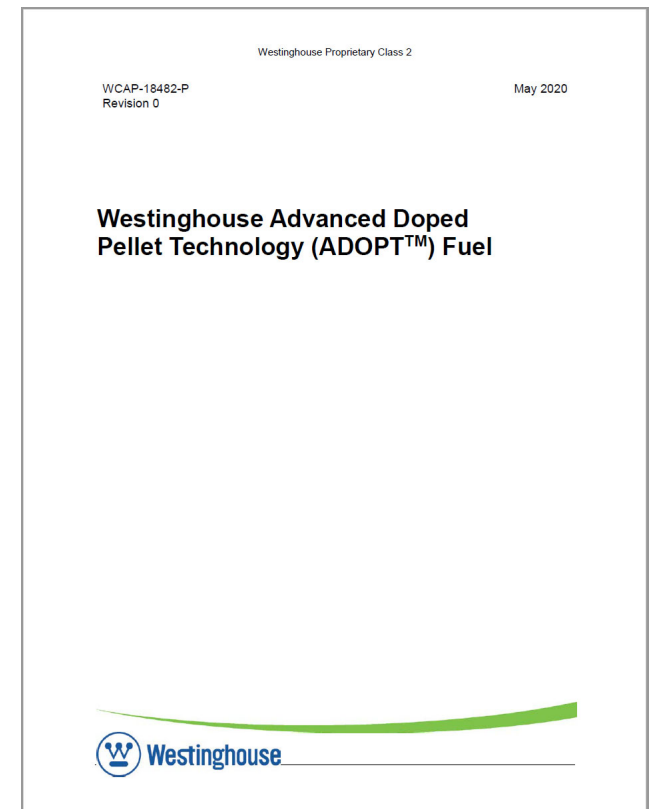


**ADOPT fuel is a mature technology**

# Topical Report Overview

# WCAP-18482-P/NP

- WCAP-18482-P/NP is a topical report to enable efficient licensing and implementation of Westinghouse Advanced Doped Pellet Technology (**ADOPT™**) Fuel
- Describes licensing topical report for region implementation of the Westinghouse **ADOPT** fuel material
  - Proposed limits of applicability
  - Discusses interaction with other topical reports and licensing considerations for implementation
  - Discusses available qualification data
  - Demonstrates applicability of existing analytical methods and models including:
    - Nuclear Design
    - Fuel Rod Design
    - Thermal-Hydraulics
    - and Safety Analysis



# Requested Limits of Applicability

## Reactor & Fuel Assembly Design Constraints

For use with NRC-approved:

- ✓ PWR reactor designs
- ✓ Westinghouse and CE fuel designs with corresponding pellet dimensions
- ✓ Zirconium-based cladding materials

## Fuel Design Constraints

- ✓ Fuel burnup up to 62 MWd/kgU peak rod average
- ✓ With or without annular pellets
- ✓ With or without application of  $\text{ZrB}_2$  integral fuel burnable absorber (IFBA)
- ✓ Nominal pellet density ranging from [ ]<sup>a,c</sup>
- ✓ Fuel grain sizes ranging from [ ]<sup>a,c</sup>
- ✓ Inclusion of Cr ranging from [ ]<sup>a,c</sup>
- ✓ Inclusion of Al ranging from [ ]<sup>a,c</sup>

# Qualification Data Overview

# ADOPT behavior relative to $\text{UO}_2$

## Summary of Thermophysical Properties

- Experimental data collected for
  - **thermal diffusivity** using laser flash technique
  - **heat capacity** using differential scanning calorimeter
  - **thermal expansion** using a differential dilatometer
  - **melting temperature** using laser-pulse melting
- Data agrees well with the published literature on the thermo-physical properties for  $\text{UO}_2$
- Recommend no additional changes to current licensed thermo-physical models to implement **ADOPT** fuel; the current licensed  $\text{UO}_2$  models are sufficient

**No changes to thermo-physical models to  
implement ADOPT fuel**

# ADOPT behavior relative to $\text{UO}_2$

## Summary of Steady State FGR

- The steady state fission gas release (FGR) database contains both thermal and athermal data sets including:
  - Performed poolside measurements, as well as destructive PIE, on commercially irradiated rods
  - Performed online FGR measurements in Halden [ ]<sup>a,c</sup> experiment
- Data confirms that dominant effect driving steady state FGR is the fuel centerline temperature. Any effect of the larger grains of doped pellets is a much smaller, second order effect.

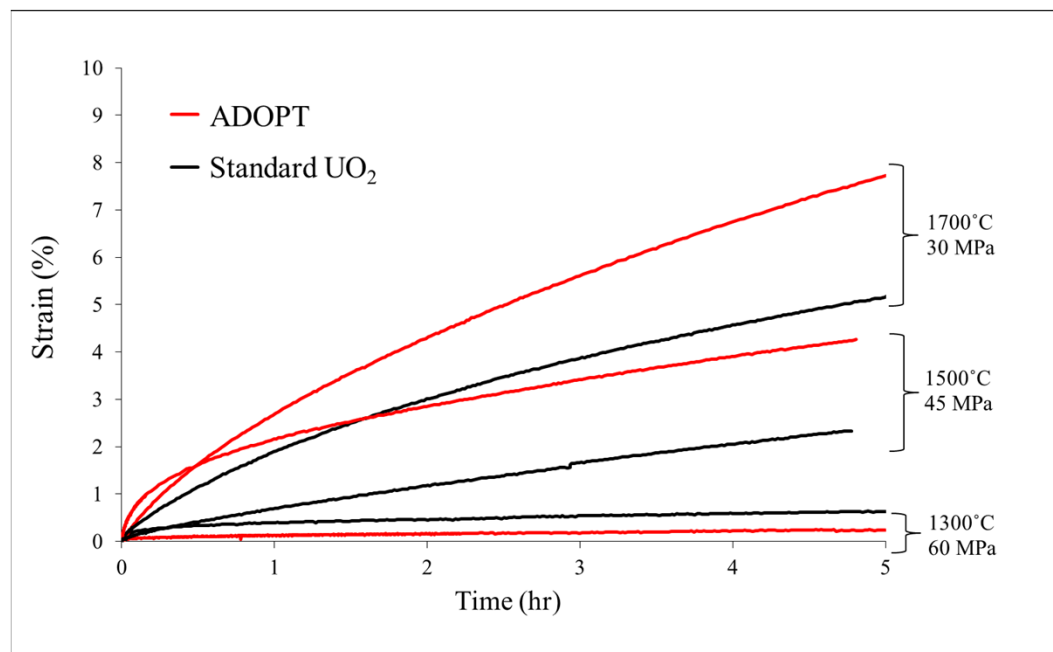
b,c

**No difference in steady-state FGR  
between ADOPT and standard fuel**

# ADOPT behavior relative to $\text{UO}_2$

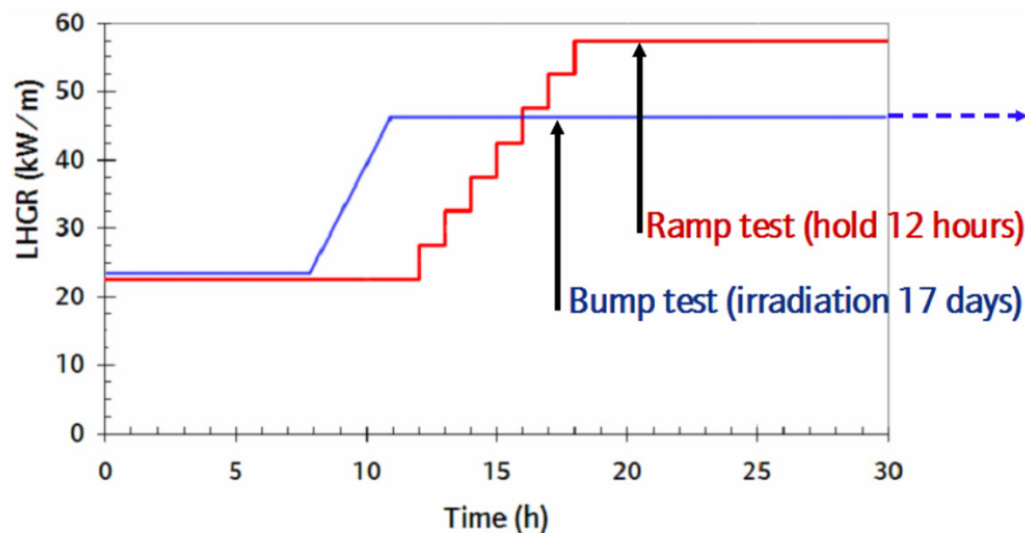
## Summary of creep behavior

- Fuel movement in the central high-temperature region of the pellet helps to reduce the pellet-clad stress
- At temperatures greater than  $1500^\circ\text{C}$  (i.e. accident conditions or ramp testing), the **ADOPT** fuel exhibits higher visco-plasticity as compared to the reference  $\text{UO}_2$  pellets



**ADOPT fuel creeps more than  $\text{UO}_2$ , but only at high temperatures beyond normal operation**

# ADOPT behavior relative to $\text{UO}_2$



Test	Standard $\text{UO}_2$	ADOPT
Ramp test (57 kW/m)	30.2% (12 hour)	17.2% (7.7 hour)
Bump test (45 kW/m)	29.7% (17 day)	20.5% (17 day)

## Transient FGR

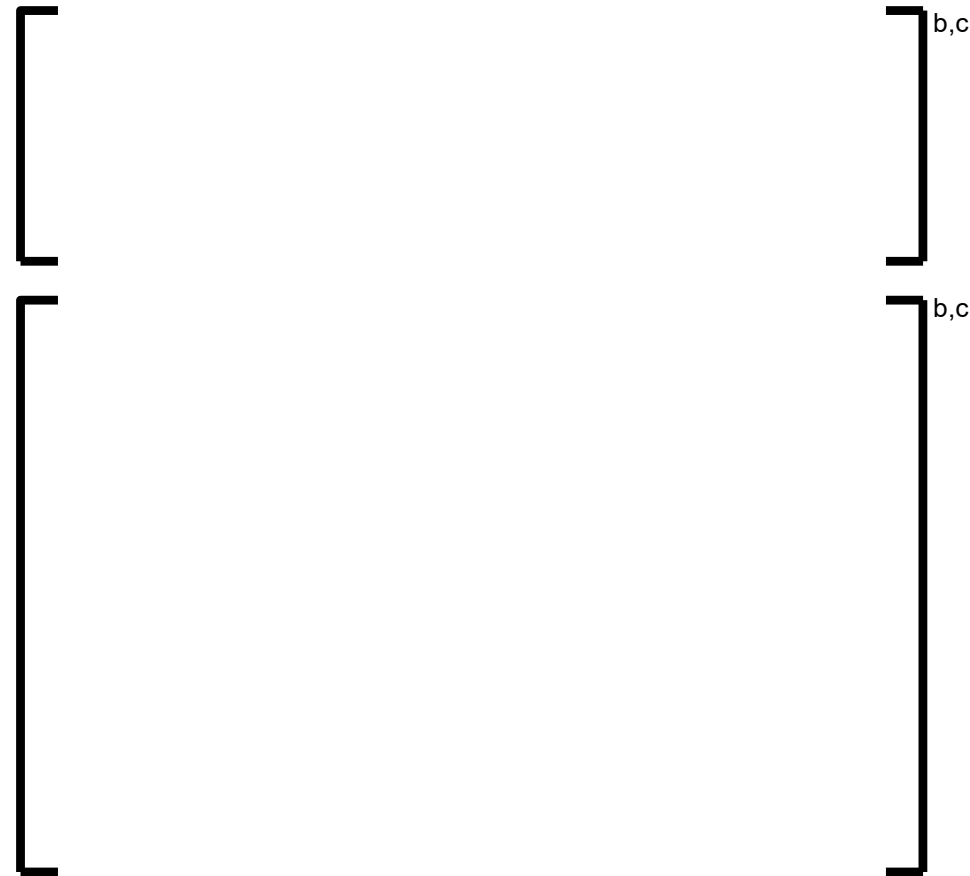
- Material base irradiated in Barsebäck
- Ramp tested at R2 reactor; the rodlets were punctured to examine the amount of FGR, and the **ADOPT** pellets have about 30% less FGR than the standard  $\text{UO}_2$  pellets.
- A better intragranular gas retention capability anticipated to decrease the rod internal pressure during an accident. This is beneficial in reducing the clad ballooning and (burst) failure risk.

**ADOPT pellets have less transient FGR than the standard  $\text{UO}_2$  pellets**

# ADOPT behavior relative to $\text{UO}_2$

## Densification & Fuel Rod Growth

- A reduction in **ADOPT** fuel's densification is observed due to the material already achieving high density during the sintering process.
- As a result of **ADOPT** fuel's reduced in-reactor densification, the pellet-to-clad gap will close earlier than standard  $\text{UO}_2$ .
  - Operating experience demonstrates that **ADOPT** rods will exhibit slightly higher rod growth due to earlier pellet-cladding contact.



## Summary of Data

- The **ADOPT** pellet is characterized by its increased density and larger grain size
- The thermophysical property measurements show a negligible difference between **ADOPT** and standard  $\text{UO}_2$  pellets
- **ADOPT** fuel creeps more than  $\text{UO}_2$  but only at high temperatures beyond normal operation.
- **ADOPT** fuel is characterized by a higher dimensional stability
  - Operating experience demonstrates that ADOPT rods will exhibit slightly higher rod growth due to earlier pellet-cladding contact.
- **ADOPT** fuel has same steady state FGR behavior as  $\text{UO}_2$
- **ADOPT** fuel has lower FGR than  $\text{UO}_2$  in transient conditions

# Analytical Methods & Modeling

*Fuel Performance*

*Nuclear Design*

*Thermal-Hydraulic Design*

*Safety Analyses*

# Fuel Performance Key Differences in PAD5

**ADOPT** fuel properties are largely the same as regular  $\text{UO}_2$ , however key differences are addressed:

a,c

Minor adjustments in PAD5 fuel rod growth and fission gas swelling models

# PAD5 Fuel Rod Growth Model Adjustment

- The rod growth data for **ADOPT** fuel were used to develop a correction to the current PAD5 model for standard  $\text{UO}_2$  fuel. Chose  $\text{UO}_2$  dataset most similar to data for **ADOPT** fuel (by fabrication and operating conditions) and fit trendline models to both datasets
- Difference was estimated by comparing datasets obtained under similar conditions and fit trendline models to both datasets
  - [ ]<sup>a,c</sup>
  - [ ]<sup>a,c</sup>

# PAD5 Transient Fission Gas Swelling Model Adjustment

## Model Development

- Adjusted model uncertainty
  - [ ]<sup>a,c</sup>
  - Based on 1000ppm Cr<sub>2</sub>O<sub>3</sub>, which represents the UB Cr dopant (worst possible **ADOPT** performance)

- [ ]<sup>a,c</sup>

## Other Analytical Areas

- Material property changes do not impact the approved methods
- Modeling of **ADOPT** pellets can be incorporated through revised input – many areas utilize input from the fuel performance code

Analytical Area	Additional Clarification	a,c
Nuclear Design		
LOCA		
Thermal-Hydraulic Design		
Non-LOCA Transient Analysis		
Containment Integrity Analysis		
Radiological Consequences		

**No updates are needed to the other NRC-approved methods to implement ADOPT pellets**



# Summary and Conclusion

# Summary

- Westinghouse has extensive experience manufacturing and deploying **ADOPT** pellets in Europe
- Westinghouse is requesting near-term region implementation of **ADOPT** pellets for PWR applications
  - Significant qualification data and operating experience supports licensing activities
- Existing analytical methods and models including Nuclear Design, Fuel Rod Design, Thermal-Hydraulics, and Safety Analysis have been assessed and are applicable to **ADOPT** fuel with minimal changes.