



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
WASHINGTON, D.C. 20555-0001

July 11, 2022

Mr. Daniel G. Stoddard  
Senior Vice President and Chief Nuclear Officer  
Innsbrook Technical Center  
5000 Dominion Blvd.  
Glen Allen, VA 23060-6711

**SUBJECT:     MILLSTONE POWER STATION UNIT NOS. 1, 2, AND 3, NORTH ANNA  
POWER STATION, UNIT NOS. 1 AND 2, AND SURRY POWER STATION UNIT  
NOS. 1 AND 2 - ISSUANCE OF AMENDMENT NOS. 120, 344, AND 284  
(MILLSTONE), 293 AND 276 (NORTH ANNA), AND 307 AND 307 (SURRY) TO  
RELOCATE STAFF QUALIFICATION REQUIREMENTS (EPID L-2021-  
LLA-0195)**

Dear Mr. Stoddard:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment Nos. 120, 344, and 284 to Facility Operating License No. DPR-21 and Renewed Facility Operating License Nos. DPR-65 and NPF-49 for the Millstone Power Station, Unit Nos. 1, 2, and 3, Amendment Nos. 293 and 276 to Renewed Facility Operating License Nos. NPF-4 and NPF-7 for the North Anna Power Station (North Anna), Unit Nos. 1 and 2, and Amendment Nos. 307 and 307 to Subsequent Renewed Facility Operating License Nos. DPR-32 and DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. These amendments are in response to your application dated October 21, 2021 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML21294A338), as supplemented by letter dated June 13, 2022 (ML22164A925).

The amendments revise the respective Technical Specifications (TSs) regarding "Unit Staff Qualifications," by removing the licensee's staff qualification requirements from the TSs and relocating the control of unit staff qualification requirements to the licensee controlled Dominion Energy Nuclear Facility Quality Assurance Program Description (QAPD) consistent with the guidance in NRC Administrative Letter (AL) 95-06, "Relocation of Technical Specification Administrative Controls Related to Quality Assurance" (ADAMS Accession No. ML031110271).

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

**/RA/**

G. Edward Miller, Project Manager  
Plant Licensing Branch 2-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-245, 50-336, 50-423, 50-338,  
50-339, 50-280, and 50-281

Enclosures:

1. Amendment No. 120 to DPR-21
2. Amendment No. 344 to DPR-65
2. Amendment No. 284 to NPF-49
4. Amendment No. 293 to NPF-4
5. Amendment No. 276 to NPF-7
6. Amendment No. 307 to DPR-32
7. Amendment No. 307 to DPR-37
8. Safety Evaluation

cc: Listserv



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

DOMINION ENERGY NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-245

MILLSTONE POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 120  
License No. DPR-21

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Dominion Energy Nuclear Connecticut, Inc. (the licensee) dated October 21, 2021, as supplemented by letter dated June 13, 2022, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-21 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 120, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Shaun M. Anderson, Chief  
Reactor Decommissioning Branch  
Division of Decommissioning, Uranium Recovery  
and Waste Programs  
Office of Nuclear Material Safety and Safeguards

Attachment:  
Changes to Facility  
Operating License No. DPR-21  
and Technical Specifications

Date of Issuance: July 11, 2022



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

DOMINION ENERGY NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-336

MILLSTONE POWER STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 344  
Renewed License No. DPR-65

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Dominion Energy Nuclear Connecticut, Inc. (the licensee) dated October 21, 2021, as supplemented by letter dated June 13, 2022, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 344, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to Renewed Facility  
Operating License No. DPR-65  
and Technical Specifications

Date of Issuance: July 11, 2022



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

DOMINION ENERGY NUCLEAR CONNECTICUT, INC., ET AL

DOCKET NO. 50-423

MILLSTONE POWER STATION UNIT NO. 3

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 284  
Renewed License No. NPF-49

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Dominion Energy Nuclear Connecticut, Inc. (DENC, the licensee), dated October 21, 2021, as supplemented by letter dated June 13, 2022, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-49 are hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, revised through Amendment No. 284 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated into the license. DENC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley,  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Operation

Attachment:  
Changes to Renewed Facility  
Operating License No. NPF-49  
and Technical Specifications

Date of Issuance: July 11, 2022



ATTACHMENT TO  
LICENSE AMENDMENT NO. 120  
MILLSTONE POWER STATION, UNIT NO.1  
FACILITY OPERATING LICENSE NO. DPR-21  
DOCKET NO. 50- 245  
LICENSE AMENDMENT NO. 344  
MILLSTONE POWER STATION, UNIT NO. 2  
RENEWED FACILITY OPERATING LICENSE NO. DPR-65  
DOCKET NO. 50-336  
LICENSE AMENDMENT NO. 284  
MILLSTONE POWER STATION, UNIT NO. 3  
RENEWED FACILITY OPERATING LICENSE NOS. NPF-49  
DOCKET NO. 50-423

Replace the following pages of the Facility Operating License and Renewed Facility Operating Licenses with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

DPR-21, page 3  
DPR-65, page 3  
NPF-49, page 4

Insert

DPR-21, page 3  
DPR-65, page 3  
NPF-49, page 4

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

5.0-5

Millstone Unit No. 1

Insert

5.0-5

Remove

6-2

Millstone Unit No. 2

Insert

6-2

Remove

6-5

Millstone Unit No. 3

Insert

6-5

(1) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 120, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(2) Deleted.

(3) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which may contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Millstone Power Station Physical Security Plan," with revisions submitted through March 29, 1988; "Millstone Power Station Suitability, Training and Qualification Plan," with revisions submitted through July 21, 1986; and "Millstone Power Station Safeguards Contingency Plan," with revisions submitted through October 30, 1985. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

(4) On the closing date of the transfer of MP1 to DNC\*, DNC\* shall: 1) obtain from the Selling Owners of MP1 the decommissioning trust fund for MP1 in an amount no less than \$268,300,000; and 2) receive a parent company guarantee pursuant to 10 CFR 50.75(e)(1)(iii)(B) (to be updated annually as required under 10 CFR 50.75(f)(1) and 50.82(a)(8)(iv), unless otherwise approved by the NRC) in an amount which, when combined with the decommissioning trust fund for MP1, equals a total of the site-specific decommissioning funding cost as of the closing date of the transfer as estimated (in year 2000 dollars) in accordance with 10 CFR 50.82 (including the use of a 2 percent annual real rate of return as provided in 10 CFR 50.75(e)(1)(i)).

(5) The decommissioning trust agreement for Millstone, Unit No. 1 at the time the transfer of the unit to Dominion Nuclear Connecticut, Inc.\* is effected and thereafter, is subject to the following:

- (a) The decommissioning trust agreement must be in a form acceptable to the NRC.

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\* On May 12, 2017, the name "Dominion Nuclear Connecticut, Inc." changed to "Dominion Energy Nuclear Connecticut, Inc."

Connecticut, in accordance with the procedures and limitations set forth in this renewed operating license;

- (1) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
- (2) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (4) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

B. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter 1: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady-state reactor core power levels not in excess of 2700 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 344, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, revised through Amendment No. 284 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated into the license. DENC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- (3) DENC shall not take any action that would cause Dominion Energy, Inc. or its parent companies to void, cancel, or diminish DENC's Commitment to have sufficient funds available to fund an extended plant shutdown as represented in the application for approval of the transfer of the licenses for MPS Unit No. 3.
- (4) Immediately after the transfer of interests in MPS Unit No. 3 to DNC\*, the amount in the decommissioning trust fund for MPS Unit No. 3 must, with respect to the interest in MPS Unit No. 3, that DNC\* would then hold, be at a level no less than the formula amount under 10 CFR 50.75.
- (5) The decommissioning trust agreement for MPS Unit No. 3 at the time the transfer of the unit to DNC\* is effected and thereafter is subject to the following:
- (a) The decommissioning trust agreement must be in a form acceptable to the NRC.
  - (b) With respect to the decommissioning trust fund, investments in the securities or other obligations of Dominion Energy, Inc. or its affiliates or subsidiaries, successors, or assigns are prohibited. Except for investments tied to market indexes or other non-nuclear-sector mutual funds, investments in any entity owning one or more nuclear power plants are prohibited.
  - (c) The decommissioning trust agreement for MPS Unit No. 3 must provide that no disbursements or payments from the trust, other than for ordinary administrative expenses, shall be made by the trustee until the trustee has first given the Director of the Office of Nuclear Reactor Regulation 30 days prior written notice of payment. The decommissioning trust agreement shall further contain a provision that no disbursements or payments from the trust shall be made if the trustee receives prior written notice of objection from the NRC.
  - (d) The decommissioning trust agreement must provide that the agreement cannot be amended in any material respect without 30 days prior written notification to the Director of the Office of Nuclear Reactor Regulation.

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\* On May 12, 2017, the name "Dominion Nuclear Connecticut, Inc." changed to "Dominion Energy Nuclear Connecticut, Inc."

## ADMINISTRATIVE CONTROLS

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### 5.3 Facility Staff Qualifications

5.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications referenced for comparable positions as specified in the Nuclear Facility Quality Assurance Program Description.

5.3.2 The operations manager or at least one operations middle manager shall be a CERTIFIED FUEL HANDLER.

FACILITY STAFF (CONTINUED)

- d. A radiation protection technician shall be on site when fuel is in the reactor. (Table 6.2-1)
- e. ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. Deleted

6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications referenced for comparable positions as specified in the Nuclear Facility Quality Assurance Program Description.
- 6.3.2 The operations manager or at least one operations middle manager shall hold a senior reactor operator license for Millstone Unit No. 2.

REVERSE OF PAGE 6-2  
INTENTIONALLY LEFT BLANK

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications referenced for comparable positions as specified in the Nuclear Facility Quality Assurance Program Description.

6.3.2 The operations manager or at least one operations middle manager shall hold a senior reactor operator license for Millstone Unit No. 3.

### 6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff that meets or exceeds the requirements as specified in the Quality Assurance Program and 10 CFR Part 55.59 shall be maintained.

6.4.2 Deleted.

6.5 DELETED.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 293  
Renewed License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated October 21, 2021, as supplemented by letter dated June 13, 2022, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to paragraph 2.C (2) of Renewed Facility Operating License No. NPF-4, as indicated in the attachment to this license amendment, and is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 293, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Operation

Attachment:  
Changes to Renewed Facility  
Operating License No. NPF-4  
and Technical Specifications

Date of Issuance: July 11, 2022



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 276  
Renewed License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated October 21, 2021, as supplemented by letter dated June 13, 2022, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to paragraph 2.C (2) of Renewed Facility Operating License No. NPF-7, as indicated in the attachment to this license amendment, and is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 276, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Operation

Attachment:  
Changes to Renewed Facility  
Operating License No. NPF-7  
and Technical Specifications

Date of Issuance: July 11, 2022

ATTACHMENT TO  
NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2  
LICENSE AMENDMENT NO. 293  
RENEWED FACILITY OPERATING LICENSE NO. NPF-4  
DOCKET NO. 50-338  
LICENSE AMENDMENT NO. 276  
RENEWED FACILITY OPERATING LICENSE NO. NPF-7  
DOCKET NO. 50-339

Replace the following pages of the Renewed Facility Operating Licenses with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

NPF-4, page 3  
NPF-7, page 3

Insert

NPF-4, page 3  
NPF-7, page 3

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

5.3-1

Insert

5.3-1

- (2) Pursuant to the Act and 10 CFR Part 70, VEPCO to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report;
  - (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or component; and
  - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, VEPCO to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I; Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

VEPCO is authorized to operate the North Anna Power Station, Unit No. 1, at reactor core power levels not in excess of 2940 megawatts (thermal).
  - (2) Technical Specifications

Technical Specifications contained in Appendix A, as revised through Amendment No. 293 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material, without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or component; and
  - (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, VEPCO to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations as set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

VEPCO is authorized to operate the facility at steady state reactor core power levels not in excess of 2940 megawatts (thermal).
  - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 276 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.
  - (3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the insurance of the condition or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission:

    - a. If VEPCO plans to remove or to make significant changes in the normal operation of equipment that controls the amount of radioactivity in effluents from the North Anna Power Station, the

## 5.0 ADMINISTRATIVE CONTROLS

### 5.3 Unit Staff Qualifications

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- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications referenced for comparable positions as specified in the Nuclear Facility Quality Assurance Program Description. The SM, Unit Supervisor, Control Room Operator, and the individual providing advisory technical support to the unit operations shift crew, shall meet or exceed the minimum qualifications of 10 CFR 55.59(c) and 55.31(a)(4).
- 5.3.2 For the purpose of 10 CFR 55.4, a licensed SRO and a licensed RO are those individuals who, in addition to meeting the requirements of TS 5.3.1, perform the functions described in 10 CFR 50.54(m).
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE

Amendment No. 307  
Subsequent Renewed License  
No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated October 21, 2021, as supplemented by letter dated June 13, 2022, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specification as indicated in the attachment to this license amendment of the Subsequent Renewed Facility Operating License No. DPR-32 are hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 307 are hereby incorporated in the subsequent renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of the date of issuance of the subsequent renewed license and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to Subsequent Renewed Facility  
Operating License No. DPR-32  
and Technical Specifications

Date of Issuance: July 11, 2022



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE

Amendment No. 307  
Subsequent Renewed License  
No. DPR-37

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated October 21, 2021, as supplemented by letter dated June 13, 2022, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specification as indicated in the attachment to this license amendment of the Subsequent Renewed Facility Operating License No. DPR-37 are hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 307 are hereby incorporated in the subsequent renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael T. Markley, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to Subsequent Renewed Facility  
Operating License No. DPR-37  
and Technical Specifications

Date of Issuance: July 11, 2022

ATTACHMENT TO  
SURRY POWER STATION, UNIT NOS. 1 AND 2  
LICENSE AMENDMENT NO. 307  
SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-32  
DOCKET NO. 50-280  
LICENSE AMENDMENT NO. 307  
SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-37  
DOCKET NO. 50-281

Replace the following pages of the Subsequent Renewed Facility Operating Licenses with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

DPR-32, page 3  
DPR-37, page 3

Insert

DPR-32, page 3  
DPR-37, page 3

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

6.1-2

Insert

6.1-2

3. This subsequent renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2587 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 307 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.

E. Deleted by Amendment 65

F. Deleted by Amendment 71

G. Deleted by Amendment 227

H. Deleted by Amendment 227

3. This subsequent renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power Levels not in excess of 2587 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 307 are hereby incorporated in this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

The licensee shall keep facility operating records in accordance with the Requirements of the Technical Specifications.

E. Deleted by Amendment 54

F. Deleted by Amendment 59 and Amendment 65

G. Deleted by Amendment 227

H. Deleted by Amendment 227

## 2. Unit Staff

The unit staff organization shall include the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition for each unit as shown in Table 6.1-1.
- b. A radiation protection technician shall be on site when fuel is in the reactor. The position may be vacant for not more than 2 hours, in order to provide for unexpected absence, provided immediate action is taken to fill the position.
- c. All core alterations shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator limited to fuel handling who has no other concurrent responsibilities during this operation.
- d. The operations manager shall hold (or have previously held) a Senior Reactor Operator License for Surry Power Station or a similar design Pressurized Water Reactor plant. The Supervisor Nuclear Shift Operations shall hold an active Senior Reactor Operator License for Surry Power Station.
- e. Procedures will be established to insure that NRC policy statement guidelines regarding working hours established for employees are followed. In addition, procedures will provide for documentation of authorized deviations from those guidelines and that the documentation is available for NRC review.

### 6.1.3 Unit Staff Qualifications

1. Each member of the unit staff shall meet or exceed the minimum qualifications referenced for comparable positions as specified in the Nuclear Facility Quality Assurance Program Description. Incumbents in the positions of Shift Manager, Unit Supervisor (SRO), Control Room Operator (RO), and the individual providing advisory technical support to the unit operations shift crew, shall meet or exceed the requirements of 10 CFR 55.59(c) and 55.31(a)(4).
2. For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator and a licensed Reactor Operator are those individuals who, in addition to meeting the requirements of TS 6.1.3.1 perform the functions described in 10 CFR 50.54(m).





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO

AMENDMENT NO. 120 TO FACILITY OPERATING LICENSE NO. DPR-21

AMENDMENT NO. 344 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-65

AMENDMENT NO. 284 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-49

AMENDMENT NO. 293 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-4

AMENDMENT NO. 276 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-7

AMENDMENT NO. 307 TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO.  
DPR-32

AMENDMENT NO. 307 TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO.  
DPR-37

DOMINION ENERGY NUCLEAR CONNECTICUT, INC.

VIRGINIA ELECTRIC AND POWER COMPANY

MILLSTONE POWER STATION, UNIT NOS. 1, 2, AND 3

NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2

SURRY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-245, 50-336, 50-423, 50-338, 50-339, 50-280, AND 50-281

1.0 INTRODUCTION

By application dated October 21, 2021 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML21294A338), as supplemented by letter dated June 13, 2022 (ML22164A925), Dominion Energy Connecticut, Inc. and Virginia Electric and Power Company (hereafter referred to as Dominion or the licensee) submitted a request for license amendments (LARs) for the Millstone Power Station (Millstone or MPS), Unit Nos. 1, 2, and 3, North Anna Power Station (North Anna or NAPS), Unit Nos. 1 and 2, and Surry Power Station (Surry or SPS), Unit Nos. 1 and 2. The amendments would revise technical specifications (TSs) to relocate the requirements related to staff qualifications requirements to the Dominion Energy Nuclear Facility Quality Assurance Program Description (QAPD).

The supplement dated June 13, 2022, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the *Federal Register*.

## 2.0 REGULATORY EVALUATION

The NRC staff considered the following regulatory requirements in its review of the licensee's application:

- Title 10 of the *Code of Federal Regulations* (10 CFR) 50.36, "Technical specifications," provides requirements for the content of the TS in the category "Administrative Controls." Paragraph (c)(5) of 10 CFR 50.36 requires, in part, that technical specifications include administrative controls, which are "provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner."
- 10 CFR 50.120, "Training and qualification of nuclear power plant personnel," requires training and qualification for personnel to operate and maintain the facility in a safe manner in all modes of operation.
- Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants," to the Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, "Domestic Licensing of Production and Utilization Facilities."
- 10 CFR 50.54(a)(3), which provides that a licensee may make a change to a previously accepted quality assurance program description included or referenced in the Safety Analysis Report without prior NRC approval, provided the change does not reduce the commitments in the program description as accepted by the NRC.

The NRC staff considered the following guidance in its review of the licensee's application:

- NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR Edition," Chapter 13, "Conduct of Operations," Section 13.2.2, "Non-Licensed Plant Staff Training," Revision 4 (ADAMS Accession No. ML15006A129), provides guidance for the NRC review of training programs for non-licensed plant staff.
- Regulatory Guide (RG) 1.8, Revision 3, "Qualification and Training of Personnel for Nuclear Power Plants," May 2000 (ADAMS Accession No. ML003706932), describes a method acceptable for complying with those portions of the Commission's regulations with regard to the training and qualifications of nuclear power plant personnel.
- NRC Administrative Letter (AL) 95-06, "Relocation of Technical Specification Administrative Controls Related to Quality Assurance," dated December 12, 1995 (ADAMS Accession No. ML031110271), provides guidance for relocating requirements from the administrative controls section of TSs in cases where adequate controls are provided by other methods. AL 95-06 notes that the quality assurance program is a logical candidate for such relocations due to the controls imposed on the quality assurance program by regulations (e.g., 10 CFR 50, Appendix B, and 10 CFR 50.54(a)),

and the existence of NRC-approved quality assurance plans and commitments to industry quality assurance standards.

### 3.0 TECHNICAL EVALUATION

#### 3.1 Millstone Unit Nos. 1, 2, and 3

##### 3.1.1 Proposed Technical Specification Changes

##### MPS Unit 1 TS 5.3.1 currently states:

###### 5.3 Facility Staff Qualifications

- 5.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions. Exceptions to this requirement are specified in the Quality Assurance Program.

##### The proposed changes would revise MPS Unit 1 TS 5.3.1 to state:

###### 5.3 Facility Staff Qualifications

- 5.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications referenced for comparable positions as specified in the Nuclear Facility Quality Assurance Program Description.

##### MPS Units 2 & 3 TS 6.3.1 currently state:

###### 6.3 Facility Staff Qualifications

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971\* for comparable positions. Exceptions to this requirement are specified in the Quality Assurance Program.

\*As of November 1, 2001, applicants for reactor operator and senior reactor operator qualification shall meet or exceed the education and experience guidelines of Regulatory Guide 1.8, Revision 3. May 2000.

##### The proposed changes would revise MPS Units 2 & 3 TS 6.3.1 to state:

###### 6.3 Facility Staff Qualifications

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications referenced for comparable positions as specified in the Nuclear Facility Quality Assurance Program Description.

##### Proposed changes to the QAPD:

Move the referenced ANSI standard (ANSI N18.1-1971) in the facility staff qualification requirements of the TS to the QAPD. Future changes to the QAPD will be controlled via the 10 CFR 50.54(a) evaluation process.

### 3.1.2 NRC Staff Evaluation

Section 2.5.1.2, "Millstone Power Station," of the Dominion QAPD Rev. 30, dated June 17, 2021 (ADAMS Accession No. ML21168A339) states:

*"Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except that:*

- 1.. The radiation protection manager shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1, May 1977.*
- 2. As of November 1, 2001, applicants for reactor operator and senior reactor operator qualification shall meet or exceed the education and experience guidelines of Regulatory Guide 1.8, Revision 3, May 2000."*

The Dominion Energy QAPD statement is consistent with the information in the current MPS Unit 1 TS 5.3.1 and MPS Units 2 & 3 TS 6.3.1. In addition, the application does not propose any reduction in commitments in the quality assurance programs. Therefore, the licensee's staff qualification requirements will not be changed by this amendment, and these requirements remain consistent with RG 1.8. In addition, the relocation of the licensee's staff qualification requirements to the Dominion QAPD are consistent with the guidance in AL 95-06.

Based on the above, the NRC staff concludes that the proposed changes are acceptable as they will remove redundant references to ANSI N18.1-1971 and will refer to the existing equivalent commitments to RG 1.8 in the QAPD. The proposed changes are consistent with Paragraph (c)(5) of 10 CFR 50.36. This proposed change will also achieve consistency among the Dominion nuclear plants regarding identification of facility staff qualification requirements in their respective TS. Future changes to the licensee's staff qualification requirements that are relocated to the Dominion's QAPD must be made in accordance with 10 CFR 50.54(a). Future reduction in commitments associated with the licensee's staff qualification requirements that are relocated to the Dominion QAPD will require NRC approval, per 10 CFR 50.54(a)(4), prior to implementation. Because the staff qualification requirements located in the QAPD will continue to be subject to adequate change control via 10 CFR 50.54(a), the staff finds that the requirements of 10 CFR 50.36 and 50.120 will continue to be met and will not adversely affect the NRC's ability to maintain oversight of licensee programs.

### 3.2 North Anna Unit Nos. 1 and 2

#### 3.2.1 Proposed Technical Specification Changes

NAPS Units 1 and 2 TS 5.3 currently states:

#### 5.3 Unit Staff Qualifications

- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI 3.1 (12/79 Draft) for the comparable positions. Exceptions to this requirement are specified in the QA Program. The SM, Unit Supervisor, Control Room Operator, and the individual providing advisory technical support to the unit operations shift crew, shall meet or exceed the minimum qualifications of 10 CFR 55.59(c) and 55.31(a)(4).

The proposed changes will revise NAPS Units 1 & 2 TS 5.3 to state:

5.3 Unit Staff Qualifications

- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications referenced for comparable positions as specified in the Nuclear Facility Quality Assurance Program Description. The SM, Unit Supervisor, Control Room Operator, and the individual providing advisory technical support to the unit operations shift crew, shall meet or exceed the minimum qualifications of 10 CFR 55.59(c) and 55.31(a)(4).

Proposed changes to the QAPD:

Move the referenced ANSI standard (ANSI 3.1) in the unit staff qualification requirements of the TSs to the QAPD. Future changes to the QAPD will be controlled via the 10 CFR 50.54(a) evaluation process.

3.2.2 NRC Staff Evaluation

Section 2.5.1.3, "North Anna and Surry Power Stations," of the Dominion QAPD Rev. 30, dated June 17, 2021 states, in part:

*"...Each member of the facility staff shall meet or exceed the minimum qualifications of ANS-3.1 (12/79 Draft) for positions except that the Radiological Protection Manager shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975."*

The Dominion Energy QAPD statement is consistent with the information in the current NAPS Unit 1& 2 TS 5.3. In addition, the application does not propose any reduction in commitments in the quality assurance programs. Therefore, the licensee's staff qualification requirements will not be changed by this amendment, and these requirements remain consistent with RG 1.8. In addition, the relocation of the licensee's staff qualification requirements to the Dominion QAPD is consistent with the guidance in AL 95-06.

Based on the above, the NRC staff concludes that the proposed changes are acceptable as they will remove redundant references to ANSI/ANS-3.1 and will refer to the existing equivalent commitments to RG 1.8 in the QAPD. The proposed changes are consistent with Paragraph (c)(5) of 10 CFR 50.36. This proposed change will also achieve consistency among the Dominion nuclear plants regarding identification of facility staff qualification requirements in their respective TS. Future changes to the licensee's staff qualification requirements that are relocated to the Dominion's QAPD must be made in accordance with 10 CFR 50.54(a). Future reduction in commitments associated with the licensee's staff qualification requirements that are relocated to the Dominion QAPD will require NRC approval, per 10 CFR 50.54(a)(4), prior to implementation. Because the staff qualification requirements located in the QAPD will continue to be subject to adequate change control via 10 CFR 50.54(a), the staff finds that the requirements of 10 CFR 50.36 and 50.120 will continue to be met and will not adversely affect the NRC's ability to maintain oversight of licensee programs.

### 3.3 Surry Unit Nos. 1 and 2

#### 3.3.1 Proposed Technical Specification Changes

##### SPS Units 1 and 2 TS 6.1.3 currently states:

###### 6.1.3 Unit Staff Qualifications

Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI 3.1 (12/79 Draft) for comparable positions. Exceptions to this requirement are specified in the QA Program. Incumbents in the position of Shift Manager, Unit Supervisor (SRO), Control Room Operator (RO), and the individual providing advisory technical support to the unit operations shift crew, shall meet or exceed the requirements of 10 CFR 55.59(c) and 55.31(a)(4).

##### The proposed changes will revise SPS Units 1 & 2 TS 6.1.3 to state:

###### 6.1.3 Unit Staff Qualifications

Each member of the unit staff shall meet or exceed the minimum qualifications referenced for comparable positions as specified in the Nuclear Facility Quality Assurance Program Description. Incumbents in the position of Shift Manager, Unit Supervisor (SRO), Control Room Operator (RO), and the individual providing advisory technical support to the unit operations shift crew, shall meet or exceed the requirements of 10 GFR 55.59(c) and 55.31(a)(4).

##### Proposed changes to the QAPD:

Move the referenced ANSI standard (ANSI 3.1) in the unit staff qualification requirements of the TS to the QAPD. Future changes to the QAPD will be controlled via the 10 CFR 50.54(a) evaluation process.

#### 3.3.2 NRC Staff Evaluation

Section 2.5.1.3, "North Anna and Surry Power Stations," of the Dominion QAPD Rev. 30, dated June 17, 2021 states:

*"...Each member of the facility staff shall meet or exceed the minimum qualifications of ANS-3.1 (12/79 Draft) for positions except that the Radiological Protection Manager shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975."*

The Dominion QAPD statement is consistent with the information in the current SPS Unit 1& 2 TS 6.1.3. In addition, the application does not propose any reduction in commitments in the quality assurance programs. Therefore, the licensee's staff qualification requirements will not be changed by this amendment, and these requirements remain consistent with RG 1.8. In addition, the relocation of the licensee's staff qualification requirements to the Dominion QAPD is consistent with the guidance in AL 95-06.

Based on the above, the NRC staff concludes that the proposed changes are acceptable as they will remove redundant references to ANSI/ANS-3.1 and will refer to the existing equivalent

commitments to RG 1.8 in the QAPD. The proposed changes are consistent with Paragraph (c)(5) of 10 CFR 50.36. This proposed change will also achieve consistency among the Dominion nuclear plants regarding identification of facility staff qualification requirements in their respective TS. Future changes to the licensee's staff qualification requirements that are relocated to the Dominion's QAPD must be made in accordance with 10 CFR 50.54(a). Future reduction in commitments associated with the licensee's staff qualification requirements that are relocated to the Dominion QAPD will require NRC approval, per 10 CFR 50.54(a)(4), prior to implementation. Because the staff qualification requirements located in the QAPD will continue to be subject to adequate change control via 10 CFR 50.54(a), the staff finds that the requirements of 10 CFR 50.36 and 50.120 will continue to be met and will not adversely affect the NRC's ability to maintain oversight of licensee programs.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Commonwealth of Virginia official was notified of the proposed issuance of the amendments. On May 5, 2022, the state official confirmed that the Commonwealth had no comments.

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendments on May 5, 2022. The state official did not provide comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change administrative requirements. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding published in the *Federal Register* on December 28, 2021 (86 FR 73817). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22 (c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 6.0 CONCLUSION

The NRC staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: M. Keefe-Forsyth, NRR  
F. Vega, NRR

Date: July 11, 2022

SUBJECT: MILLSTONE POWER STATION UNIT NOS. 1, 2, AND 3, NORTH ANNA POWER STATION, UNIT NOS. 1 AND 2, AND SURRY POWER STATION UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENT NOS. 120, 344, AND 284 (MILLSTONE), 293 AND 276 (NORTH ANNA), AND 307 AND 307 (SURRY) TO RELOCATE STAFF QUALIFICATION REQUIREMENTS (EPID L-2021-LLA-0195) DATED JULY 11, 2022

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**ADAMS Accession No. ML22095A107****ADAMS Package Accession No.: ML22095A099****\*Via SE Input**

OFFICE	NRR/DORL/LSPB/PM	NRR/DORL/LPL2-1/LA	NRR/DRO/IVB/BC	NRR/DSS/STSB/BC
NAME	GEMiller	Kgoldstein	KKavanagh	VCusumano
DATE	5/17/2022	04/15/2022 & 06/30/22	4/27/2022	5/4/2022
OFFICE	OGC (NLO)	NMSS/DUWP/RDB/BC	NRR/DORL/LPL2-1/BC	
NAME	AGhosh Naber	SAnderson	MMarkley	
DATE	5/17/2022	7/11/2022	7/11/2022	

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