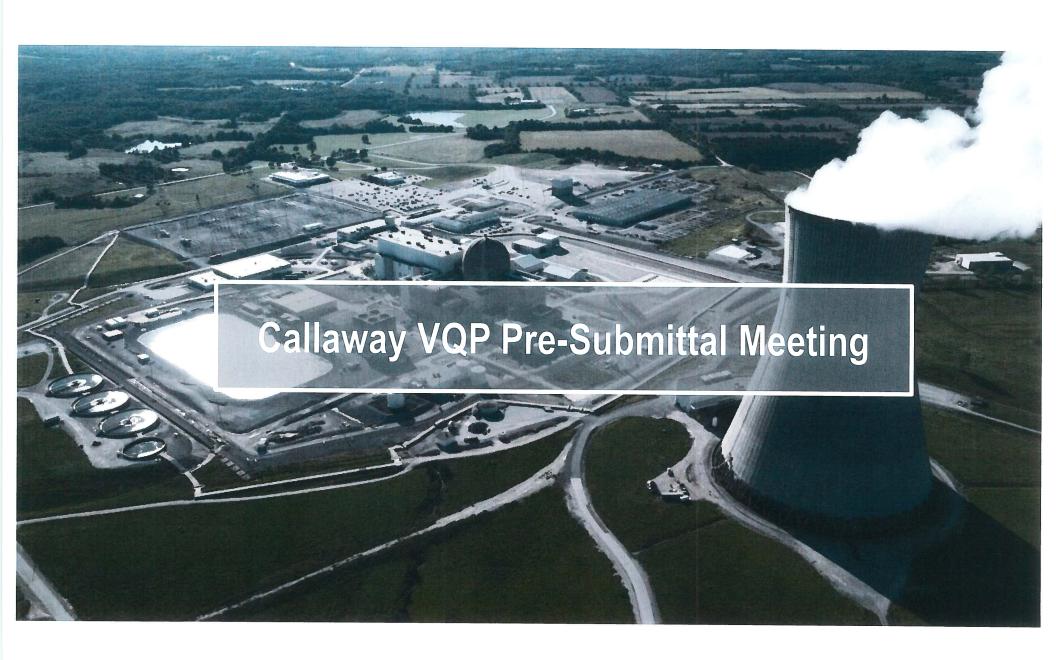
Callaway Unit 1 Presentation
Information for Planned
Pre-Application Meeting
(Public Meeting)











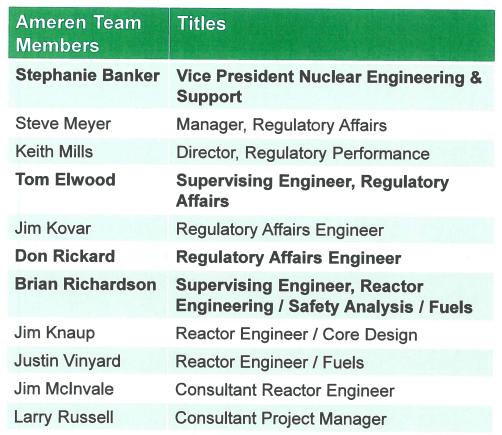
Callaway VQP Pre-Submittal Meeting

Stephanie Banker - Ameren - Vice President Nuclear Engineering & Support

- Welcome
- Meeting Purpose to provide the NRC with information on the upcoming License Amendment Request (LAR) for the Vendor Qualification Program (VQP) of Framatome as a supplier of nuclear fuel to Callaway Energy Center

Team Members

Presenters in Bold





Framatome Team Members	Titles
Rick Williamson	Contract Manager
Tom Gardner	Project Manager
Christy White	Technical PM
Greg Borza	Thermal-Hydraulics
Morris Byram	Licensing
Kevin Segard	Neutronics
Michael Harris	Thermal-Mechanics
Tim Lindquist	Safety Analysis
Gordo Wissinger	LOCA Analysis
Pam Reed	Mechanical Design
Brian Painter	Mechanical Analysis



Agenda

Purpose of Meeting / Desired Outcomes
 Callaway

Background
 Callaway

GAIA Fuel Assembly Design Description
 Framatome

Summary of NRC Approved Methods
 Framatome

Analyses
 Framatome

Technical Specification Changes Callaway

Schedule
 Callaway

Background



- Callaway LFA / VQP Program for Framatome Fuel
 - Lead Fuel Assemblies (LFA) 4 Fuel Assemblies
 - 3 cycles (Cycles 25, 27, 28)
 - Non-Limiting Operation (Cycle 25 only)
 - Vendor Qualification Program (VQP)
 - Fuel Transition Licensing Submittal
 - VQP Lead Fuel Assemblies 4 Additional Fuel Assemblies
 - 3 cycles (Cycles 27, 28, 29)
 - Fully Licensed Operation
- GAIA Fuel Design
- Based on currently approved methods (with some changes)
- Technical Specification changes to add Framatome methods

Fuel Assembly Design Description

(ANP-10342P-A Revision 0, GAIA Fuel Assembly Mechanical Design)



- M5 Fuel Rods
- M5 GAIA Mixing Grid (6x)
- M5 Intermediate GAIA Mixer (IGM) Grids (3x)
- Alloy 718 HMP Top (Relaxed) and Bottom Spacer Grids
- Q12 MONOBLOC Guide Tubes (24x)
- Q12 Instrument Tube
- GRIP Bottom Nozzle







Summary of Methods (1 of 2)

Functional Area	Topical Report
Neutronics	ARCADIA (ANP-10297P-A and ANP-10297P-A Supp. 1P-A, Revision 1)
Thermal–Hydraulic	XCOBRA-IIIC (XN-NF-82-21P-A and XN-NF-75-21P-A Revision 2),
	COBRA-FLX (ANP-10311P-A Revision 1)
Non-LOCA	S-RELAP5 (EMF-2310P-A Revision 1)
DNBR Correlation	GAIA (ANP-10341P-A)
Control Rod Ejection	AREA (ANP-10338P-A)
Control Rod Ejection	GALILEO (ANP-10323P-A)
SB LOCA	S-RELAP5 (EMF-2328P-A and S1P-A)
LB LOCA	S-RELAP5 (EMF-2103P-A Revision 3)

These NRC Approved Methods will be added to the Callaway Tech Specs / COLR





Functional Area	Topical Report
Fuel Performance Code	COPERNIC (BAW-10231P-A),
	CROV (BAW-10084PA Revision 3)
External Loads	ANP-10337P-A
GAIA Mechanical Design	ANP-10342P-A
Cladding	BAW-10240PA,
Cladding	BAW-10227P-A Revision 1
Statistical Setpoints	EMF-92-081P-A
Fuel Rod Bow	XN-75-32P-A
DNB Propagation	XN-NF-82-06P-A

These NRC Approved Methods will be added to the Callaway Tech Specs / COLR





- Palo Verde Fuel Transition ML20031C947
- St. Lucie 2 Fuel Transition ML16063A121
- Calvert Cliffs Fuel Transition ML110390263
- Calvert Cliffs Accident Tolerant Fuel LTAs ML20363A242
- St. Lucie 1 Extended Power Uprate ML12181A019
- Shearon Harris GAIA Implementation ML21047A470

VQP Analyses



- Analyses were completed in the following disciplines, using methods discussed on the subsequent pages:
 - Fuel rod thermal-mechanical
 - Fuel assembly mechanical
 - Neutronics
 - Thermal-hydraulics
 - Non-LOCA
 - Rod Ejection Analysis Method meets new RG 1.236 (analysis underway)
 - LOCA (Large and Small Break)



Fuel Rod Thermal-Mechanical

Analysis / Basis	Method	Results / Comments
Fuel Rod Analyses*	ANP-10342P-A	Acceptable, accounts for
	BAW-10231P-A	TCD
	BAW-10084P-A, Rev. 3	
Cladding Performance / Properties	BAW-10240P-A	Acceptable, M5 cladding
	BAW-10227P-A, Rev. 1	

^{*} Modifications to Approved Methods described on "Methods Changes" slides





Fuel Assembly Mechanical

Analysis / Basis	Method	Results
Mechanical Compatibility	N/A	Acceptable, compatible with reactor and resident fuel/components
Structural Design Evaluations	ANP-10342P-A	Acceptable
Seismic / LOCA Mechanical Faulted Analysis	ANP-10337P-A and	Acceptable, coolable geometry and RCCA insertion requirements met



Neutronics

Analysis / Basis	Method	Results
Core Design (ARTEMIS)	ANP-10297P-A, Revision 0 ANP-10297P-A, Revision 0, Supplement 1P-A Revision 1)	 Acceptable, Modeled cycles 13-24 Created representative transition cycles Generated power histories, axial shapes and other safety inputs
Power Distribution Control*	Defined in LAR	Acceptable
Rod Ejection	AREA (ANP-10338P-A) GALILEO (ANP-10323P-A)	Expected to be acceptable, meets requirements from RG 1.236

^{*} Modifications to Approved Methods described on "Methods Changes" slides



Thermal Hydraulic

Analysis / Basis	Method	Results
XCOBRA-IIIC	XN-75-21(P)(A) Rev 2	XCOBRA-IIIC is used for setpoint verification and Ch. 15 DNB analysis
COBRA-FLX	ANP-10311P-A, Rev 1	COBRA-FLX is used for thermal- hydraulic compatibility analysis and Ch. 15 DNB analysis.
Mixed Core Departure from Nucleate (DNB) Penalty	XN-NF-82-21(P)(A), Rev 1	Penalty applied to MDNBR for mixed cores.
Thermal-Hydraulic Compatibility	N/A	Verified that GAIA is thermal-hydraulically compatible with the co-resident fuel





Thermal Hydraulic (Cont.)

Method	Results
XN-NF-75-32P-A Supplements 1-4*	No rod bow penalties for fresh fuel
Defined in LAR	Gap closure model updated with BAW-10227 Revision 2, Q3P Revision 0
XN-NF-82-06P-A Rev 1	No fuel failure predicted for VQP Will be used to quantify fuel failures if necessary
ANP-10341P-A Rev 0*	Re-validated in XCOBRA-IIIC and re- validated with PV-Solver in COBRA-FLX
EMF-92-081P-A Rev 1* BAW-10231P-A Rev 1	Cycle specific FCM limit calculated RODEX2 replaced with COPERNIC
	XN-NF-75-32P-A Supplements 1-4* Defined in LAR XN-NF-82-06P-A Rev 1 ANP-10341P-A Rev 0* EMF-92-081P-A Rev 1*

^{*} Modifications to Approved Methods described on "Methods Changes" slides



• Thermal Hydraulic (Cont.)

Analysis / Basis	Method	Results
Statistical Setpoints (OTDT/OPDT/CSLL)	EMF-92-081P-A Rev 1	Verified that the existing setpoints for OTDT, OPDT, and the CSLLs are applicable for GAIA at Callaway
Non-LOCA Chapter 15 Events (DNB/FCM)	EMF-92-081P-A Rev 1 EMF-2310(P)(A) Rev 1*	Acceptance criteria met

^{*} Modifications to Approved Methods described on "Methods Changes" slides



Non-LOCA

Analysis / Basis	Method	Results
Chapter 15 Non-LOCA Events	EMF-2310P-A Revision 1*	Acceptable
analyzed:		
 Decrease in FW Temp 		
- Increase in FW Flow		
- Increase in Steam Flow		
- Pre-Scram SLB		
- Post-Scram SLB		
- Loss of RCP Flow		
- RCP Shaft Seizure		
- Uncontrolled Bank Withdrawal		
- Dropped Rod		
- Boron Dilution		
- Inadv. Opening of PORV		

^{*} Modifications to Approved Methods described on "Methods Changes" slides



Non-LOCA (Cont.)

Analysis / Basis	Method	Results
 Chapter 15 Non-LOCA Events dispositioned: Inadvertent opening of a SG relief valve Turbine Trip Loss of non-emergency AC power Partial loss of forced reactor coolant flow Reactor coolant pump shaft break Steam generator tube rupture 	EMF-2310P-A Revision 1*	Acceptable

^{*} Modifications to Approved Methods described on "Methods Changes" slides



Loss of Coolant Accident - LOCA

Analysis / Basis	Method	Results
Realistic Large Break LOCA	EMF-2103P-A Revision 3*	Acceptable
Small Break LOCA	EMF-2328P-A and S1P-A*	Acceptable

RLBLOCA:

- Best-estimate uncertainty analysis
- Included LOOP and no-LOOP cases

SBLOCA:

- Break spectrum and SI line break analyses
- SI Line break not limiting

PCT and Oxidation confirmed to meet limits

* Modifications to Approved Methods described on "Methods Changes" slides





- Other Analyses Impacted
 - Mass & Energy Release
 - Fuel Assembly Handling
 - Radiological Consequence
 - GSI-191 (Debris Generation and Transport)
 - Spent Fuel Pool Criticality

Technical Specification Change Implications



- Cycle 26 starts late May 2022
 - No Framatome fuel in core
- Cycle 27 starts late October 2023
 - 8 Framatome GAIA Assemblies (4 LFAs and 4 VQP) unrestricted placement
 - Balance of core continues to be Westinghouse-supplied fuel assemblies
 - Presence of Framatome fuel assemblies dictates changes to:
 - TS 2.1.1 Reactor Core Safety Limits
 - TS 4.2.1 Fuel Assemblies
 - TS 5.6.5 Core Operating Limits Report (COLR)
 - Westinghouse methods continue to govern COLR development
 - Framatome evaluations ensure compliance with limits
- New fuel supplier contract to be established late in Cycle 26 or early in Cycle 27





- Cycle 28
 - Batch load quantities of fresh fuel from selected supplier
 - If Westinghouse is chosen:
 - Westinghouse methods continue to govern COLR development
 - Framatome evaluations ensure compliance with limits
 - No additional TS changes are needed (current Power Distribution Specs apply)
 - Follow-up LAR after completion of Cycle 29 to remove changes needed for the Framatome assemblies (present in cycles 27, 28 and 29)
 - If Framatome is chosen:
 - Framatome methods will govern COLR development
 - Westinghouse evaluations ensure compliance with limits
 - New Power Distribution Limits Specs (TS 3.2.1-3.2.3) need to be in place reflecting Framatome methods
 - Follow-up LAR to remove obsolete Westinghouse specifications

Tech Spec Implications (continued)



- Ameren Missouri will provide a commitment to have only one fuel vendor in order to eliminate concern regarding "mixed fresh fuel batches."
- The requested License Amendment ensures the revised TS are fully in place to allow use of Framatome fuel at the time that the contracts are established
- Therefore, this License Amendment Request (LAR) will:
 - Revise TS 2.1.1 Reactor Core Safety Limits; TS 4.2.1 Fuel Assemblies; TS 5.6.5 COLR as discussed earlier to accommodate Framatome fuel (needed by 9/30/2023)
 - Retain the current Westinghouse TS 3.2.x Power Distribution Limits TS and COLR methods
 - · Required for cycle 27 regardless of vendor; and thereafter, if Westinghouse is selected
 - Add the Framatome TS 3.2.x Power Distribution Limits TS and COLR methods
 - Required for cycle 28 if Framatome is selected
 - LCO Applicability will be mutually exclusive to ensure only one vendor's methods govern COLR development
 - Both vendors may perform evaluations to ensure compliance with limits

Technical Specification (TS) Changes



- TS 2.1.1 Reactor Core Safety Limits
 - Revise to add Framatome DNBR and fuel centerline temperature limits

Designate Westinghouse Methods-Based as "A" (Retains Westinghouse-based COLR Methods)

Designate Framatome Methods-Based as "B" (Adopts Framatome-based COLR Methods)

- TS 3.2.1A Heat Flux Hot Channel Factor (F_O(Z))
- TS 3.2.1B Heat Flux Hot Channel Factor (F_O(Z))
- TS 3.2.2A Nuclear Enthalpy Rise Hot Channel Factor (F^N_{ΔH})
- TS 3.2.2B Nuclear Enthalpy Rise Hot Channel Factor (F^N _{AH})
- TS 3.2.3A Axial Flux Difference (AFD) (RAOC)
- TS 3.2.3B Axial Flux Difference (AFD)
- TS 3.2.4 Quadrant Power Tilt Ratio (QPTR)
 - Revise Required Actions to refer to Surveillance Requirements with "A" and "B" designations
- TS 4.2.1- Fuel Assemblies
 - Revise to refer to zirconium clad in lieu of Zircalloy or ZIRLO
- TS 5.6.5 Core Operating Limits Report (COLR)
 - Revise to add Framatome methods to list of methods used to support COLR





- M5 Cladding 10 CFR 50.46 and 10 CFR 50 Appendix K Exemption Request
- Similar to M5 cladding exemption requests for other plants

Separate TS License Amendment Request

- Currently updating the spent fuel pool criticality analyses for GAIA fuel
- May require change to TS 4.3, Fuel Storage
- May require change to TS 3.7.16, Fuel Storage Pool Boron Concentration, and TS 3.7.17, Spent Fuel Assembly Storage

Schedule



- LAR submittal
 - Pre-submittal Meeting February 28, 2022
 - Ameren/Framatome can support NRC Audit, if requested
- Requested NRC approval of LAR

Supports Fuel Cycle 27, RFO begins October 2, 2023

March 30, 2022

September 30, 2023

Questions

