



Sequoyah Nuclear Plant, P.O. Box 2000, Soddy Daisy, Tennessee 37384

February 4, 2022

10 CFR 50.4
10 CFR 50.55a

ATTN: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555-0001

Sequoyah Nuclear Plant, Unit 2
Renewed Facility Operating License No. DPR-79
NRC Docket No. 50-328

Subject: **Unit 2 Cycle 24 Refueling Outage - 90-Day Inservice Inspection Summary Report**

In accordance with the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, Article IWA-6240 and Code Case N-532-5, the Tennessee Valley Authority is providing the Sequoyah Nuclear Plant (SQN), Unit 2, Inservice Inspection (ISI) Summary Report within 90 days from completion of the Unit 2 Cycle 24 Refueling Outage (U2C24). The U2C24 refueling outage ended on November 9, 2021. Accordingly, this report is required to be submitted by February 7, 2022.

The Enclosure to this letter contains the Owner's Activity Report, a list of items with flaws or relevant conditions that required evaluation for continued service (Table 1), an abstract of repair and replacement activities required for continued service due to a flaw or relevant condition (Table 2), and the evaluation of acceptability of inaccessible areas of the steel containment vessel in accordance with 10 CFR 50.55a(b)(2)(ix)(A).

There are no regulatory commitments associated with this submittal. Should you have any questions, please contact Jeffery Sowa, Site Licensing Manager at (423) 843-8098.

Respectfully,

Marshall, Thomas B.  Digitally signed by Marshall, Thomas B.
Date: 2022.02.04 08:35:20 -05'00'

Thomas Marshall
Site Vice President
Sequoyah Nuclear Plant

U.S. Nuclear Regulatory Commission
Page 2
February 4, 2022

Enclosure: ASME Section XI Inservice Inspection Summary Report,
Unit 2 Cycle 24 Refueling Outage

cc (Enclosure):

NRC Regional Administrator – Region II
NRC Senior Resident Inspector – Sequoyah Nuclear Plant
NRC Project Manager – Sequoyah Nuclear Plant

ENCLOSURE

**TENNESSEE VALLEY AUTHORITY
SEQUOYAH NUCLEAR PLANT, UNIT 2**

**ASME SECTION XI
INSERVICE INSPECTION SUMMARY REPORT
UNIT 2 CYCLE 24 REFUELING OUTAGE**

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number U2R24

Plant SEQUOYAH NUCLEAR PLANT, P.O. Box 2000, Soddy-Daisy, TN 37384-2000

Unit No. 2 Commercial Service Date June 1, 1982 Refueling Outage No. 24
(if applicable)

Current Inspection Interval 4th
(1st, 2nd, 3rd, 4th, other)

Current Inspection Period 2nd
(1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the Inspection Plans 2007 Edition, 2008 Addenda - Inservice Inspection (ISI)
2013 Edition - Containment Inservice Inspection (CISI)

Date / Revision of Inspection Plans 0-PI-DXI-000-114.4 R0006 (ISI)
0-PI-DXI-000-116.3 R0000 (CISI)

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans 2007 Edition, 2008 Addenda

Code Cases used: N-508-4, N-513-4, N-532-5, N-586-1, N-716-1, N-722-1, N-729-6, N-770-5

CERTIFICATE OF CONFORMANCE

I certify that (a) the statements made in this report are correct; (b) the examinations and tests, meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of conform to the requirements of Section XI

U2R24

(refueling outage number)

Signe

d


Owner or Owner's Designee, Title

Date

1/27/22**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors

and employed by The Hartford Steam Boiler Inspection and Insurance Company of Hartford, Connecticut

have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection


Inspector's Signature

Commissions

NB 14730 1NR

(National Board Number and Endorsement)

Date

27 January 2022

TABLE 1
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT REQUIRED EVALUATION FOR
CONTINUED SERVICE

Examination Category and Item Number	Item Description	Evaluation Description
F-A / F1.30C	8" AFWS Piping Support 2-AFDH-223 was misaligned.	CR 1727564 The evaluation is documented in Calculation 2AFDH0223 R3 and found the support to be acceptable as-is.
D-B / D2.10	Pipe Chase Room Cooler 2A-A approximate 20-25 drops per minute leak from solder joint.	CR 1742174 This condition was evaluated using TVA's Prompt Determination of Operability process and found to be acceptable for continued service.
D-B / D2.10	714' Penetration Room Cooler 2B-B approximate 1 GPM leak.	CR 1738092 This condition was evaluated using TVA's Prompt Determination of Operability process and found to be acceptable for continued service.
D-B / D2.10	2B-B CRDM Cooler approximate 20 drops per minute leak.	CR 1733327 This condition was evaluated using TVA's Prompt Determination of Operability process and found to be acceptable for continued service.
D-B / D2.10	'669 Pipe Chase Room Cooler 2B-B has an internal leak of ~0.75 GPM.	CR1693400 This condition was evaluated using TVA's Prompt Determination of Operability process and found to be acceptable for continued service.
D-B / D2.10	EGTS Room Cooler B-B leak in the heat affected area of tube braze.	CR 1611076 This condition was evaluated using TVA's Prompt Determination of Operability process and found to be acceptable for continued service.

TABLE 2**Abstract of Repairs/replacement Activities Required for Continued Service**

Code Class	Item Description	Description of Work	Date Completed	Repair/Replacement Plan Number
3	0.75" Emergency Raw Water System Piping	Replace 3/4" Pipe due to through wall leak discovered.	11/1/2021	122502602

REPORTING REQUIRED BY 10 CFR 50.55a(b)(2)(ix)(A)(2)
ASME Section XI, Subsection IWE
Steel Containment Vessel Inspection Program

10 CFR 50.55a(b)(2)(ix)(A)(2) requires licensee to provide the following in the ISI Summary Report as required by IWA-6000:

- (i) A description of the type and estimated extent of degradation, and the conditions that led to the degradation;
- (ii) An evaluation of each area, and the result of the evaluation; and
- (iii) A description of necessary corrective actions.

There were no conditions identified during the Unit 2 Cycle 24 refueling outage on the items examined that prompted an evaluation of the inaccessible area of the Steel Containment Vessel, as required.