



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

January 20, 2022

Mr. David Rhoades
Senior VP, Exelon Generation Company, LLC
President and CNO, Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: LASALLE COUNTY STATION – BIENNIAL PROBLEM IDENTIFICATION AND
RESOLUTION INSPECTION REPORT 05000373/2021013 AND
05000374/2021013

Dear Mr. Rhoades:

On December 20, 2021, the U.S. Nuclear Regulatory Commission (NRC) completed a problem identification and resolution inspection at your LaSalle County Station and discussed the results of this inspection with Mr. Van Fleet, and other members of your staff. The results of this inspection are documented in the enclosed report.

The NRC inspection team reviewed the station's corrective action program and the station's implementation of the program to evaluate its effectiveness in identifying, prioritizing, evaluating, and correcting problems, and to confirm that the station was complying with NRC regulations and licensee standards for corrective action programs. Based on the samples reviewed, the team determined that your staff's performance in each of these areas adequately supported nuclear safety.

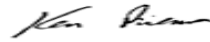
The team also evaluated the station's processes for use of industry and NRC operating experience information and the effectiveness of the station's audits and self-assessments. Based on the samples reviewed, the team determined that your staff's performance in each of these areas adequately supported nuclear safety.

Finally, the team reviewed the station's programs to establish and maintain a safety-conscious work environment, and interviewed station personnel to evaluate the effectiveness of these programs. Based on the team's observations and the results of these interviews the team found no evidence of challenges to your organization's safety-conscious work environment. Your employees appeared willing to raise nuclear safety concerns through at least one of the several means available.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,



Signed by Riemer, Kenneth
on 01/20/22

Kenneth R. Riemer, Chief
Branch 1
Division of Reactor Projects

Docket Nos. 05000373 and 05000374
License Nos. NPF-11 and NPF-18

Enclosure:
As stated

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Letter to David Rhoades from Kenneth R. Riemer dated January 20, 2022.

SUBJECT: LASALLE COUNTY STATION – BIENNIAL PROBLEM IDENTIFICATION AND
RESOLUTION INSPECTION REPORT 05000373/2021013 AND
05000374/2021013

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U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Numbers: 05000373 and 05000374

License Numbers: NPF-11 and NPF-18

Report Numbers: 05000373/2021013 and 05000374/2021013

Enterprise Identifier: I-2021-013-0017

Licensee: Exelon Generation Company, LLC

Facility: LaSalle County Station

Location: Marseilles, IL

Inspection Dates: November 29, 2021 to December 17, 2021

Inspectors: J. Bozga, Senior Reactor Inspector
G. Hansen, Sr. Emergency Preparedness Inspector
G. Hausman, Senior Reactor Inspector
P. Smagacz, Resident Inspector

Approved By: Kenneth R. Riemer, Chief
Branch 1
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a biennial problem identification and resolution inspection at LaSalle County Station, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

OTHER ACTIVITIES – BASELINE

71152B - Problem Identification and Resolution

Biennial Team Inspection (IP Section 02.04) (1 Sample)

- (1) The inspectors performed a biennial assessment of the licensee's corrective action program, use of operating experience, self-assessments and audits, and safety conscious work environment.
 - **Corrective Action Program Effectiveness:** The inspectors assessed the corrective action program's effectiveness in identifying, prioritizing, evaluating, and correcting problems. The inspectors also conducted a five-year review of the 125V DC systems and Core Standby Cooling systems.
 - **Operating Experience, Self-Assessments and Audits:** The inspectors assessed the effectiveness of the station's processes for use of operating experience, audits and self-assessments.
 - **Safety Conscious Work Environment:** The inspectors assessed the effectiveness of the station's programs to establish and maintain a safety-conscious work environment.

INSPECTION RESULTS

Assessment	71152B
Based on the samples reviewed, the team determined that the licensees' performance in each of these areas adequately supported nuclear safety.	
<u>Effectiveness of Problem Identification:</u>	
Overall, the licensee was effective at identifying issues at a low threshold and was properly entering them into the Corrective Action Program (CAP) as required by procedures. The inspectors determined that the station was generally effective at identifying negative trends that could potentially impact nuclear safety, at an appropriate threshold and at a healthy rate, team interviewed personnel, reviewed documents, walked down portions of the plant, and	

assessed licensee problem identification. Staff were encouraged to identify issues and felt comfortable raising concerns with management. However, there were numerous descriptions of the workforce feeling that very low-level items never get fixed, even though numerous condition reports are entered year after year or every time certain surveillances are performed.

The inspectors reviewed root cause and apparent cause evaluations and determined those evaluations were adequate. The inspectors noted that adding more detail to the event timelines will aid in an independent reviewer understanding all known facts. For example, during the review of the root cause report for the refuel bridge mast foreign material issue (AR 4407109), a statement "troubleshooting was performed, and fuel moves recommenced" was made. This statement could have been expounded upon and detailed the troubleshooting efforts, what was or was not identified, and the criteria for recommencing with fuel moves. Also, detailed descriptions on the actual physical limitations of the refuel mast after each failure would have aided an independent reviewer.

Effectiveness of Prioritization and Evaluation of Issues:

In depth reviews of a risk-informed sampling of Action Requests (ARs), work orders (WOs), and cause evaluations were completed, including 5-year reviews of issues associated with the 125V DC Battery Systems and Core Standby Cooling Systems. The inspectors identified that the licensee generally, appropriately prioritized and evaluated issues for resolution. The inspectors also noted that while issue evaluations were generally sound, there were several examples where additional research was needed to understand the technical basis, primarily due to a lack of clear documentation. This theme had previously been identified in earlier licensee self-assessments and CAP audits and continued to be a concern.

Effectiveness of Corrective Actions:

The inspectors concluded that the licensee was generally effective in developing Corrective Actions that were appropriately focused to correct the identified problem and correct conditions adverse to quality. For issues considered significant conditions adverse to quality, the licensee took action to address the root and contributing causes and to preclude repetition. Corrective actions were generally completed in a timely manner and in accordance with procedural requirements commensurate with the safety significance of the issue.

5-year Reviews of the 125V DC Systems and Core Standby Cooling Systems:

The inspectors performed an expanded 5-year review of the Units 1 and 2 125 Vdc system; specifically, by performing system walk-downs and evaluating condition reports, engineering changes, and work orders. The inspectors also interviewed the system engineer responsible for this system. Overall, the inspectors determined that the licensee was effectively managing issues associated with this system.

As part of the review of the Core Standby Cooling System, the team identified a condition report (AR 4406388) which was associated with the use of operability limits (ASME Section III Appendix F) for determination of pressure boundary integrity/continued operation of FAC component 2RA023, a 4" tee in the reactor bottom head drain line 2RR28B-4. The inspection of this condition report will continue as a Selected Issue for Follow-Up (71152 inspection procedure) during NRC inspections in 2022.

Operating Experience, Self-Assessment and Audits:

The inspectors reviewed operating experience captured in the CAP and sampled operating experience from NRC, industry, vendors, and third-party groups. Overall, for the samples selected, the licensee was generally performing the appropriate assessments for station applicability. Through interviews and observations at selected station meetings, the inspectors also concluded that operating experience was being appropriately used in daily plant activities.

The inspectors determined that the licensee was adequately performing self-assessments and audits in accordance with licensee procedures and implementing corrective actions as needed. The inspectors did note not all self-assessment templates developed included sufficient guidance to ensure consistent evaluations. This was specifically identified in security self-assessments where one assessment included a discussion on the quantity of records reviewed, but other assessments contained blanket statements of passing.

Safety Conscious Work Environment:

As part of the NRC's safety conscious work environment evaluation during the PI&R biennial inspection, NRC inspectors completed a limited scope inspection sample of an assessment performed by the licensee that evaluated human performance behaviors exhibited during the recent refueling outage, L1R18, and the potential impact on safety conscious work environment at the site. The inspectors performed an independent review that evaluated the licensee's internal assessment of the site's SCWE program and the actions that were developed and implemented in response to the licensee's identified areas for improvement to promote a healthy organizational safety conscious work environment. The NRC inspectors reviewed licensee evaluation records, internal organizational assessment survey results, training plans, and conducted interviews with the Employee Concerns Program Manager, Organizational Effectiveness Manager, and management staff from various departments.

No violations or findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On December 20, 2021, the inspectors presented the biennial problem identification and resolution inspection results to Mr. Van Fleet, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71152B	Calculations	L-003447	LaSalle Units 1 and 2, 125Vdc System Analysis Dated June 11, 2020	002
	Corrective Action Documents	04062322	Replace Battery 2DC18E Cell 44	10/12/2017
		04114041	Replace or Retest 1DC07E	03/12/2018
		04144731	Generate Forced Outage Work Order for Battery Replacement	06/06/2018
		04203137	U2 Division 1 Battery Replacement	12/14/2018
		04455972	Request WR to Replace 2DC16E K306 Relay	10/26/2021
		04455973	Request WR to Replace 2DC23E K306 Relay	10/26/2021
		1337255	TRM Appendix G, Table 2 Enhancements Needed	03/01/2012
		141231	VR Supply Duct Access Repair Patch Not Installed Per Design	01/27/2003
		1518242	Ops Crew 6 4/17/13 LOOP Lessons Learned	04/17/2013
		2387941	2E22-F369A Relief Valve Failed As-Found Test	09/29/2014
		2530386	FAC Program - OE Review of Davis Besse Steam Rupture	05/19/2015
		2618561	Degraded Condition Observed on CSCS Piping Line 2HP55A-4	01/28/2016
		3990967	Degraded Condition Observed on CSCS Piping Line 2HP54A-4	03/29/2017
		4048633	RM - 1C71A-K10C Failed to De-Energize During LOS-RP-Q2	09/03/2017
		4099862	Limitation For 2HP55BB Per EC 403264 Flaw Evaluation	02/01/2018
		4172339	Unit 2 LPCS Motor Cooler Inlet Pipe 2DG22A-1.5" is Degraded	09/12/2018
		4192778	2018 WANO AFI ER.2	11/06/2018
		4221635	VT-3 Exams Specified on Supports Exempted from Examination	02/19/2019
		4225076	ACB 1414 Had a Lockout Trip While Performing LOS-AP-R2	03/01/2019
		4228241	Low Power TIP Set not Completed Due to TIPs OOS	03/10/2019
		4251021	Review Code Requirements for Replacement VY Cooling Coils	05/22/2019
		4252624	Code Non-Conformance Identified For VY Coolers 1(2)VY03A	05/29/2019

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		4257366	4.0 Critique IR on Tendon Surveillances	06/17/2019
		4258604	Late Surveillance (LIS-OG-408)	06/21/2019
		4261844	Cardox Tank Compressor Tripped on Thermal Overloads	07/04/2019
		4262090	NRC ID - Dry Pipe FP Lines not Sloped Per NFPA13	07/05/2019
		4265814	1RE026 Closed Light Did Not Illuminate During LOS-PC-Q1	07/21/2019
		4265960	Large Steam Leak Found Downstream of 1G33-F040	07/22/2019
		4266264	Performance Level 3 OPEX Review of ICES 456532 for Applicability to LaSalle	08/23/2019
		4267062	MSIV Pilot Solenoid Leads Crossed	07/25/2019
		4267121	PCE Level 1	07/25/2019
		4270252	Historical 2C51-J004C App J Leakage Evaluations not Found	08/07/2019
		4271342	Unexpected MCR Alarm: Unit 1 Hydrogen Panel Trouble	08/12/2019
		4273172	Part 21 Flowserve Unqualified NDE Performance of Component	08/20/2019
		4273349	NER NC-19-010-Y OE Review- Potential LaSalle Vulnerability	08/20/2019
		4276944	Part 21 Contactors Failing to Release/Open When De-Energized	09/05/2019
		4277173	NRC IN 2019-05: Over Pressurization of Alpha Sources	09/06/2019
		4280047	AT 04257366-04 Needs to be Reevaluated	09/17/2019
		4281169	LOS-SY-SR1 Revision Needed	09/22/2019
		4284443	Through Wall Leak on ST Piping in U2 Div 1 CPCS Pump Room	10/02/2019
		4286161	Unacceptable Trend Identified on MSIV Timing Per LOS-PC-Q2	10/08/2019
		4288235	NRC IN 2019-08 OPEX Regarding FAC Causing RX Scrams	10/16/2019
		4288344	NRC ID: NRC Question on 1G33-F040	10/16/2019
		4288357	Replace Piping 1DG23B-6"	10/16/2019
		4294984	NRC IN 19-09: Spent Fuel Cask Movement Issues	10/30/2019
		4295415	Revise EC 363524 to Extend VY Fan SBM C\S Replacement PM's	11/07/2019
		4305470	Biennial Safety Culture Self-Assessment	02/04/2020
		4306254	Level 3 OPEX Review Requested #465166 Diablo Canyon	12/27/2019

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Unit 2	
		4306904	1B RR Pump Lower Oil Reservoir Hi Level	12/30/2019
		4307124	Level 3 OPEX Review Requested #464385 Salem Unit 2	12/31/2019
		4307131	Level 3 OPEX Review Requested #458702 Hatch Unit 2	12/31/2019
		4318198	L1R18 FAC; 1RH006 Low Wall Thickness	02/15/2020
		4319242	L1R18 FAC; Component 1RH014 Low Wall Thickness	02/19/2020
		4322219	1ES010B Indicating Closed	02/28/2020
		4322221	1ES010C Indicating Closed	02/28/2020
		4328259	P21- Potential Deviation of Pressure Transmitters	03/20/2020
		4330162	Rosemount Transmitters Part 21 for Changes in Heat Rise	03/27/2020
		4343229	Flex Generator Issue at Quad: LaSalle has Similar Generator	05/14/2020
		4357505	U1 EHC Pressure Low in Band with New Discharge Filter	07/18/2020
		4362593	NRC Question on 2CW03A Piping Support	08/10/2020
		4365890	RM - Step Change Reduction in U2 Jet Pump 15 Flow	08/27/2020
		4366965	Degraded Piping Support	09/02/2020
		4368035	LR Inspection of 2LP13B-3" Piping	09/08/2020
		4368037	LR Inspection of 2LP23AA-1/2" Piping	09/08/2020
		4368040	LR Inspection for 2LP19A-3" Piping	09/08/2020
		4380329	SR 3.4.3.1 Jet Pump# 15 Curve No Conservative Change	10/28/2020
		4382365	B DFP High Discharge Pressure	11/05/2020
		4383488	2A RHR Seal Cooler Flow Low	11/10/2020
		4389292	RM - JP 5/6 ACMP Flow Mismatch	12/10/2020
		4391800	Loss of U2 Div 3 Water Leg Pump	12/23/2020
		4391803	NRC IN 2020-04 - FP Mn Yard Buried Cast Iron Piping Failures	12/17/2020
		4404161	L2R18 FAC Component 2HJ016 Low Wall Thickness	02/23/2021
		4404521	L2R18 FAC; 2RH004, Low Wall Thickness	02/24/2021
		4404903	2DG01A Heat Exchanger Tubes Material Degradation	02/24/2021
		4404903	2DG01A Heat Exchanger Tubes Material Degradation	02/26/2021
		4405141	Unanticipated Does Rate Alarm in High Radiation Area	02/27/2021
		4406388	FAC Inspection on 2RA023 not Performed	03/03/2021
		4406519	NRC ID Failure to Write IR in Heat Exchanger Program	03/03/2021
		4406519	NRC ID Failure to Write IR in Heat Exchanger Program	03/04/2021

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		4406544	NRC ID LR Selective Leaching Program Perf. Deficiencies	03/01/2021
		4406555	L2R18 FAC; 2MS66B-RT-01, Low Wall Thickness	03/04/2021
		4407109	Unit 2 Refuel Bridge Mast Issue During Fuel Moves	03/07/2021
		4407109	Unit 2 Refuel Bridge Mast Issue During Fuel Moves	03/07/2021
		4407388	"L" SRV Drop Test Failure	03/08/2021
		4407915	"L" SRV Failed LOS-MS-R& Drop Test	03/10/2021
		4407917	"R" SRV Failed Regulated Header Drop Test	03/10/2021
		4409413	NRC Identified-NRC Questioned Weld Procedure Qualification	03/15/2021
		4417755	RM - Unit 2 Jet Pump #6 is not Within Jet Pump Curves	04/20/2021
		4422004	FAC RWCU Bottom Head Drain Lessons Learned	05/06/2021
		4423071	NRC IN 2021-01: Design Bases Lled Power Operated Valves	05/11/2021
		4426705	RM - 2A RPS M/G Set Output Breaker Tripped	05/31/2021
		4427494	2A RPS MG Set Voltage Adjustment Pot Bad	06/04/2021
		4428890	L1R19 Class 1 Small Bore Pipe Socket Welds	06/11/2021
		4433739	Tendon Grease Cap Area Unit 2 Concrete Inspections	07/07/2021
		4441741	2021 PI&R SA Gap: OBJ 3A	08/20/2021
		44447160	No Flow Thru U1 EHC Fullers Earth Filters Via Hydraulic Pump	09/17/2021
		4448905	10CFR Part 21. Ametek Equalize Relay	09/27/2021
	Corrective Action Documents Resulting from Inspection	4465190	NRC ID- 2HP54A-4" UT Inspection Results Per WO-4621240	12/07/2021
		4466247	NRC Identified Issue With L-003447 Rev. 002	12/13/2021
		4466816	NRC ID - 2DG22A-1.5" Min Wall Design Limit	12/15/2021
		4467017	NRC Observation - 2021 PI&R - CR Type AR Generation	12/16/2021
		4467039	NRC Observation - PI&R 2021 - Security Self-Assessments	12/16/2021
		4467111	NRC Observation - PI&R 2021 - Enhancement Opportunity for RCR	12/16/2021
	Engineering Changes	403264	Minimum Wall Thickness for Unit 2 Division 3 VY Cooler Piping for Extent of Condition	1
		403264	Minimum Wall Thickness for Unit 2 Division 3 VY Cooler Piping for Extent of Condition	2
		628380	L2R18 FAC Min Wall Evaluation	4

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Engineering Evaluations	628380	L2R18 FAC Min Wall Evaluation	5
		LAS-02121	Failure Analysis of Limit Switch from LaSalle Station	08/14/2018
		LAS-24193	Failure Analysis of a Namco Limit Switch from LaSalle Station	11/16/2018
		LAS-62503	Failure Analysis of a Namco limit Switch from Lasalle Station	05/03/2019
		LAS-93383	Failure Analysis of a NAMCO Limit Switch Assembly	06/13/2012
	Miscellaneous	4405141	Rad Worker Behaviors Root Cause	05/27/2021
		4407109	Root Cause Investigation Report for Unit 2 Refuel Bridge Mast Issue During Fuel Moves	06/25/2021
		IEEE STD 485-1997	IEEE Recommended Practice for Sizing Lead-Acid Batteries for Stationary Applications	09/11/2003
		IEEE STD 946-1985	IEEE Recommended Practice for the Design of Safety-Related DC Auxiliary Power Systems for Nuclear Power Generating Stations	03/21/1985
		NOSA-LAS-19-05	Engineering Design Control Audit Report	07/10/2019
		NOSA-LAS-20-05	Engineering Programs Audit Report	07/15/2020
		NOSA-LAS-21-05	LaSalle Corrective Action Program Audit Report	09/01/2021
		OPEX Evaluation 2570259-02	ICES 318466, Pin Hole Leak at Residual Heat Removal Service Water (RHRSW) Pipe Elbow	11/25/2015
		OPEX Evaluation 4056244-02	LER 2017-003, Division 2 Residual Heat Removal Service Water System (RHRSW) Inoperable Due to an Inoperable RHRSW Flow Control Valve (FCV) (Fermi 2)	11/09/2017
		OPEX Evaluation 430760	ESW Inop Due to Silt Buildup	04/29/2020
	NDE Reports	20-141	Ultrasonic Exam of 2DG22A-1 1/2	09/08/2020
	Procedures	EI-AA-101	Employee Concerns Program	11
		ER-AA-320	Maintenance Rule Implementation Per NEI 18-10	0
		NISP-PI-02	Conduct of Self-Assessments and Benchmarks	0
		NO-AA-10	Quality Assurance Topical Report	97
		PI-AA-101	Conduct of Performance Improvement	2
		PI-AA-1012	Safety Culture Monitoring	3
		PI-AA-115	Operating Experience Program	5
		PI-AA-120	Issue ID and Screening Process	11

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		PI-AA-120	Issue Identification and Screening Process	11
		PI-AA-125-1001	Root Cause Analysis Manual	6
		PI-AA-125-1003	Corrective Action Program Evaluation Manual	6
		PI-AA-126-1001	Self-Assessments	5
	Self-Assessments	4091749	RP Personnel Knowledge SA Report Assignment	07/02/2019
		4113395	NRC SY Baseline Inspect-SA Report PASSPORT Assignment	06/06/2019
		4113396	NRC EP Baseline Inspect-SA Report PASSPORT Assignment	09/05/2019
		4192133	SA Report - M&TE Inventory Control, Access, & Handling	10/08/2019
		4192133	Self-Assessment M&TE Inventory Control, Access, & Handling	11/06/2018
		4192145	SA Report Assignment - M&T Effectiveness of ACAD 17-01	10/02/2019
		4192182	SA Report Assignment - NRC SY Baseline Inspection	12/18/2019
		4257452	Self-Assessment of SA/BM NISP-PI-02 Implementation	05/14/2020
		4305470	Biennial Safety Culture Self-Assessment	02/04/2020
		4329347	LaSalle Pre-NRC Triennial Heat Sink	02/24/2021
		4329367	SA Report - Security Baseline Inspection	07/21/2021
		4329672	SA Report - RP Baseline Inspection	07/16/2021
		4329675	SA Report - EP Baseline Inspection/Performance Indicator Verification	10/01/2021
		4343239	Cyber Security Program Effectiveness Self-Assessment	05/14/2020
		4362946	LaSalle M&T Comprehensive Self-Assessment Report	06/30/2021
		4378794	2020 INPO Performance Based Evaluation Results	09/23/2021
		4430137	2021 Preparation for NRC Problem Identification and Resolution (PI&R) Inspection	08/23/2021
		LaSalle Station 01/27-31/2020	Fleet Assessment Special Assessment Report	02/24/2020
		LaSalle Station 03/23-27/2020	Fleet Assessment T-6 Assessment Report (Maintenance)	04/16/2020
		LaSalle Station 05/17-21/2021	Fleet Assessment T-18 Assessment Report (Maintenance)	06/14/2021
		LaSalle Station	Fleet Assessment Mid-Cycle Assessment Report	09/18/2019

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		08/16-28/2019	(Maintenance)	
		LaSalle Station	Fleet Assessment Midcycle Assessment Debrief	11/15/2021
	Work Orders	4866827	Unit 2 Refuel Bridge Contingency Repairs	03/07/2021