TENNESSEE VALLEY AUTHORITY

1101 Market Street, Chattanooga, Tennessee 37402

CNL-21-071

August 13, 2021

10 CFR 50.54(q)
10 CFR 72.44(f)

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555-001

Browns Ferry Nuclear Plant, Units 1, 2, and 3
Renewed Facility Operating License Nos. DPR-33, DPR-52, and DPR-68
NRC Docket Nos. 50-259, 50-260, 50-296, and 72-052

Sequoyah Nuclear Plant, Units 1 and 2
Renewed Facility Operating License Nos. DPR-77 and DPR-79
NRC Docket Nos. 50-327, 50-328, and 72-034

Watts Bar Nuclear Plant, Units 1 and 2
Facility Operating License Nos. NPF-90 and NPF-96
NRC Docket Nos. 50-390, 50-391, and 72-1048

Subject: **TENNESSEE VALLEY AUTHORITY – RADIOLOGICAL EMERGENCY PLAN AND CENTRAL EMERGENCY CONTROL CENTER EMERGENCY PLAN IMPLEMENTING PROCEDURE REVISIONS**

In accordance with the requirements of Title 10 of the *Code of Federal Regulations* (10 CFR) 50.54(q) and 10 CFR 72.44(f), Tennessee Valley Authority (TVA) is submitting a description of changes to the TVA Radiological Emergency Plan (REP). The affected documents are the Generic REP and Central Emergency Control Center (CECC) Emergency Plan Implementing Procedures (EPIP) listed below.

Document	Revision	Title	Effective Date
REP-Generic	113	Radiological Emergency Plan (Generic Part)	7/19/2021
CECC EPIP-1	70	Central Emergency Control Center (CECC) Operations	7/19/2021

Document	Revision	Title	Effective Date
CECC EPIP-2	51	Operations Duty Specialist Procedure for Notification of Unusual Event	7/19/2021
CECC EPIP-3	52	Operations Duty Specialist Procedure for Alert, Site Area Emergency, or General Emergency	7/19/2021
CECC EPIP-6	45	CECC Plant Assessment Staff Procedure for Alert, Site Area Emergency, and General Emergency	7/19/2021
CECC EPIP-7	42	CECC Radiological Assessment Staff Procedure for Alert, Site Area Emergency, and General Emergency	7/19/2021
CECC EPIP-8	52	Dose Assessment Staff Activities During Nuclear Plant Radiological Emergencies	7/28/2021
CECC EPIP-19	25	Post Accident Fuel Damage Assessment	7/22/2021

Description of Changes and Summary of Analysis

REP-Generic

TVA revised this REP as follows:

- Figure 3-1, Offsite Emergency Organization, removed the Browns Ferry Nuclear Plant (BFN) Site Vice President. This position is only required at Watts Bar Nuclear Plant (WBN).
- Removed all references to the Emergency Paging System (EPS), which is no longer used or maintained by Emergency Preparedness (EP). Sections 6.0, 6.2, 6.5, and 6.6 were affected by the change. Section 6.5 was removed entirely and subsequent sections were renumbered accordingly.
- Figure 10-1, Initial Protective Action Recommendation (PAR) Flowchart, reformatted Notes 4 & 5 for rapidly progressing severe accident (RPSA) to align with the format found in Operations procedures. Also, updated the wording in Note 4 for BFN to make it clear that primary containment and drywell radiation are synonymous.

CECC EPIP-1

TVA revised this EPIP as follows:

- Section 3.1, removed the EPS as a means of activation and notification.
- Appendix A, removed item 8, requiring the Emergency Duty Officer to call Facilities to have lights turned on during non-business hours. The lights are now on motion sensors. Subsequent items were renumbered accordingly.
- Appendices C, I, K, and R, updated the location of Alabama Radiation Control from Montgomery to Prattville.
- Appendix D, added step to contact the State Governor's Authorized Representative if Item 3 of Appendix B is complete.
- Appendix H, Initial PAR Flowchart, reformatted Notes 4 & 5 for RPSA to align with the format found in Operations procedures. Also, updated the wording in Note 4 for BFN to make it clear that primary containment and drywell radiation are synonymous.

CECC EPIP-2

TVA revised this EPIP as follows:

- Section 3.1, Step [2] moved to Step [4] to make it clear that the user should continue with the section after completing Appendix B, if requested.
- Appendix A, updated Notification of Unusual Event form to be consistent with updates being made to the site notification forms concerning radiological releases, which consists of three options for radiological releases attributable to the event.
- Appendix B, removed instructions for activating the EPS if TVA Enterprise Emergency Notification System (TEENS) fails to activate, and replaced them with instructions to contact OnSolve/MIR3 directly to have the vendor activate TEENS. The EPS is no longer used or maintained by EP.

CECC EPIP-3

TVA revised this EPIP as follows:

- Section 3.0, Step [9], corrected the step number referenced within the step.
- Section 3.0, removed instructions for activating the EPS if TEENS fails to activate, and replaced them with instructions to contact OnSolve/MIR3 directly to have the vendor activate TEENS. The EPS is no longer used or maintained by EP.
- Section 4.2[1], added initials and time lines to record when Emergency Response Data System is activated.

CECC EPIP-6

TVA revised this EPIP as follows:

- Appendix C, Initial PAR Flowchart, reformatted Notes 4 & 5 for RPSA to align with the format found in Operations procedures. Also, updated the wording in Note 4 for BFN to make it clear that primary containment and drywell radiation are synonymous.

CECC EPIP-7

TVA revised this EPIP as follows:

- Attachment 1, Initial PAR Flowchart, reformatted Notes 4 & 5 for RPSA to align with the format found in Operations procedures. Also, updated the wording in Note 4 for BFN to make it clear that primary containment and drywell radiation are synonymous.

CECC EPIP-8

TVA revised this EPIP as follows:

- Attachment 19, updated WBN steam generator blowdown values for radiation monitors 1-RE-90-120, 121 in liquid effluent cases 1, 2, and 3.

CECC EPIP-19

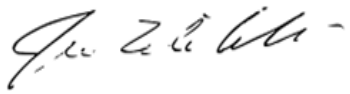
TVA revised this EPIP as follows:

- Attachment 1, updated checklist to add a note concerning monitoring for tritium in a WBN beyond design basis event where core exit thermocouples exceed 1600°F or it is suspected that Tritium-Producing Burnable Absorber Rod failure(s) have occurred.
- Attachments 9, 10, and 11, added notes to reflect that plant engineering data system readings for the 273/274 monitors at both Sequoyah Nuclear Plant and WBN are unreliable below 1.0E+00 R/hr, because that is the lower limit for the instruments.

The above changes were evaluated in accordance with 10 CFR 50.54(q)(3) and 10 CFR 72.44(f). TVA determined that the changes did not reduce the effectiveness of the TVA REP. The TVA REP, as revised, continues to meet the requirements in Appendix E to 10 CFR 50 and the planning standards of 10 CFR 50.47(b).

There are no new regulatory commitments associated with this submittal. If you have any questions regarding this information, please contact Kimberly D. Hulvey, Senior Manager, Fleet Licensing at (423) 751-3275.

Respectfully,



James T. Polickoski
Director, Nuclear Regulatory Affairs

cc: (See Page 5)

cc:

NRC Regional Administrator – Region II
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NRC Senior Resident Inspector – Sequoyah Nuclear Plant
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