



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

April 21, 2021

Mr. Michael Strobe
Site Vice President
NextEra Energy Point Beach, LLC
6610 Nuclear Road
Two Rivers, WI 54241-9516

SUBJECT: POINT BEACH NUCLEAR PLANT – INTEGRATED INSPECTION REPORT
05000266/2021001 AND 05000301/2021001

Dear Mr. Strobe:

On March 31, 2021, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Point Beach Nuclear Plant. On April 13, 2021, the NRC inspectors discussed the results of this inspection with you and other members of your staff. The results of this inspection are documented in the enclosed report.

One finding of very low safety significance (Green) is documented in this report. This finding involved a violation of NRC requirements. We are treating this violation as a non-cited violation (NCV) consistent with Section 2.3.2 of the Enforcement Policy.

If you contest the violation, or the significance or severity of the violation documented in this inspection report, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region III; the Director, Office of Enforcement; and the NRC Resident Inspector at Point Beach Nuclear Plant.

If you disagree with a cross-cutting aspect assignment in this report, you should provide a response within 30 days of the date of this inspection report, with the basis for your disagreement, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region III; and the NRC Resident Inspector at Point Beach Nuclear Plant.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room, in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

Néstor J. Félix Adorno, Chief
Branch 4
Division of Reactor Projects

Docket Nos. 05000266 and 05000301
License Nos. DPR-24 and DPR-27

Enclosure:
As stated

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Letter to Michael Strope from Néstor Félix Adorno dated April 21, 2021.

SUBJECT: POINT BEACH NUCLEAR PLANT – INTEGRATED INSPECTION REPORT
05000266/2021001 AND 05000301/2021001

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U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Numbers: 05000266 and 05000301

License Numbers: DPR-24 and DPR-27

Report Numbers: 05000266/2021001 and 05000301/2021001

Enterprise Identifier: I-2021-001-0047

Licensee: NextEra Energy Point Beach, LLC

Facility: Point Beach Nuclear Plant

Location: Two Rivers, WI

Inspection Dates: January 01, 2021 to March 31, 2021

Inspectors: T. Hartman, Senior Resident Inspector
R. Ng, Project Engineer
V. Petrella, Resident Inspector
C. Zoia, Senior Operations Engineer

Approved By: Néstor J. Félix Adorno, Chief
Branch 4
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Point Beach Nuclear Plant, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

Improper Field Change Resulted in Reactor Coolant Pressure Boundary Leakage			
Cornerstone	Significance	Cross-Cutting Aspect	Report Section
Initiating Events	Green NCV 05000266/2021001-01 Open/Closed	[H.4] - Teamwork	71153
A self-revealed Green finding and associated Non-cited Violation (NCV) of Title 10 CFR Part 50, Appendix B, Criteria III, "Design Control," and Technical Specification (TS) 3.4.13, "RCS [Reactor Coolant System] Operational Leakage," was identified when the licensee failed to subject field changes to design control measures that were commensurate with those applied to the original design. Specifically, the licensee made a field change when installing a pipe to the Unit 1 B steam generator and did not subject it to the applicable design control measures. This resulted in RCS pressure boundary leakage.			

Additional Tracking Items

Type	Issue Number	Title	Report Section	Status
LER	05000266/2020-001-00	LER 2020-001-00 for Point Beach Nuclear Plant, Unit 1 Reactor Coolant System Pressure Boundary Leak on Steam Generator Bowl Drain Line Results in Operation Prohibited by Technical Specifications	71153	Closed

PLANT STATUS

Unit 1 began the inspection period at rated thermal power. On February 3, 2021, Unit 1 lowered power to 98.5 percent rated thermal power to support leading edge flow meter maintenance. On February 6, 2021, Unit 1 returned to rated thermal power and remained at or near full power throughout the remainder of the inspection period.

Unit 2 began the inspection period at rated thermal power. On February 3, 2021, Unit 2 lowered power to 98.5 percent rated thermal power to support leading edge flow meter maintenance. On February 6, 2021, Unit 2 returned to rated thermal power and remained at or near full power throughout the remainder of the inspection period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the Coronavirus Disease 2019 (COVID-19), resident inspectors were directed to begin telework and to remotely access licensee information using available technology. During this time, the resident inspectors performed periodic site visits each week; conducted plant status activities, as described in IMC 2515, Appendix D, "Plant Status," observed risk-significant activities; and completed on-site portions of IPs. In addition, resident and regional baseline inspections were evaluated to determine if all, or portions, of the objectives and requirements stated in the IP could be performed remotely. If the inspections could be performed remotely, they were conducted per the applicable IP. In some cases, portions of an IP were completed remotely and on-site. The inspections documented below met the objectives and requirements for completion of the IP.

REACTOR SAFETY

71111.01 - Adverse Weather Protection

Impending Severe Weather Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated the adequacy of the overall preparations to protect risk-significant systems from impending severe weather for high winds, cold temperatures, and snow on January 26, 2021.

71111.04 - Equipment Alignment

Partial Walkdown Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) Unit 1 train A component cooling water system on February 18, 2021
- (2) G-04 emergency diesel generator on March 2, 2021
- (3) Unit 2 train A component cooling water system on March 10, 2021

71111.05 - Fire Protection

Fire Area Walkdown and Inspection Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) fire zone 187 on March 8, 2021
- (2) fire zone 318 on March 9, 2021
- (3) fire zones 138, 139, and 142A on March 8, 2021
- (4) fire zones 142, 150, and 151 on March 8, 2021
- (5) fire zones 156 and 156A on March 8, 2021

71111.11A - Licensed Operator Regualification Program and Licensed Operator Performance

Regualification Examination Results (IP Section 03.03) (1 Sample)

- (1) The inspectors reviewed and evaluated the licensed operator written and operating test examination failure rates for the regualification exams administered for 2020, in accordance with the COVID-19 extension granted through March 31, 2021. The licensee developed remediation plans, as appropriate, for operators who failed portions of the exam.

71111.11Q - Licensed Operator Regualification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (1 Sample)

- (1) The inspectors observed and evaluated licensed operator performance in the Control Room during power ascension on February 11, 2021.

Licensed Operator Regualification Training/Examinations (IP Section 03.02) (1 Sample)

- (1) The inspectors observed and evaluated a licensed operator simulator exam on February 9, 2021.

71111.12 - Maintenance Effectiveness

Maintenance Effectiveness (IP Section 03.01) (2 Samples)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

- (1) emergency diesel generator starting air system
- (2) spent fuel pool level indications

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (4 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

- (1) red grid condition while G-02 emergency diesel and Unit 1 train A component cooling pump out of service on February 18, 2021
- (2) emergent work on Unit 2 component cooling water system due to potential system leak on March 4, 2021
- (3) emergent work on Unit 2 leading edge flow meter equipment due to instrument spiking on March 16 and March 23, 2017
- (4) dry fuel campaign dry runs on March 16, 2021

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (5 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) FLEX diesel Z-2000B failed to start
- (2) seal leak on Unit 1 train B component cooling water pump
- (3) lowering pressure on Unit 2 main feed isolation valve accumulators
- (4) potential through wall leak on Unit 2 component cooling water system
- (5) component cooling water piping analysis not updated when modification removed piping penetration

71111.18 - Plant Modifications

Temporary Modifications and/or Permanent Modifications (IP Section 03.01 and/or 03.02) (1 Sample)

The inspectors evaluated the following temporary or permanent modifications:

- (1) leading edge flow meter (LEFM) cabinet replacement/upgrade

71111.19 - Post-Maintenance Testing

Post-Maintenance Test Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated the following post-maintenance test activities to verify system operability and functionality:

- (1) TS 82, Emergency Diesel Generator G-02 Monthly Testing, after maintenance on G-02 emergency diesel generator on February 24, 2021
- (2) TS 83, Emergency Diesel Generator G-03 Monthly Testing, after maintenance on G-03 emergency diesel generator on March 2, 2021
- (3) 1-SOP-AF-001, Auxiliary Feedwater System Operation - Motor Driven, after maintenance on Unit 1 motor-driven auxiliary feedwater pump on March 4, 2021
- (4) partial TS 9, Control Room Heating and Ventilation System Checks, after maintenance on recirculation fans W-13B1 and 2 and flow switch FS-04133C on March 23, 2021
- (5) IT-21, Charging Pumps and Check Valve Test (Quarterly) Unit 1, after replacement of relief valve on March 26, 2021

71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

Surveillance Tests (other) (IP Section 03.01) (3 Samples)

- (1) 1ICP 02.003A, Reactor Protection Logic Train A 31 Day Surveillance Test, on January 14, 2021
- (2) 1ICP 02.005A, Engineered Safety Features System Logic Train A 31 Day Staggered Actuation Logic Test, on January 14, 2021
- (3) TS 4, Main Turbine Stop and Governor Valves with Turbine Trip Test (Biannual) Unit 2, on February 11, 2021

Inservice Testing (IP Section 03.01) (1 Sample)

- (1) IT 05 Train B, Train B Containment Spray Pump and Valves Unit 1, on March 3, 2021

71114.06 - Drill Evaluation

Drill/Training Evolution Observation (IP Section 03.02) (1 Sample)

The inspectors evaluated:

- (1) a crew simulator evaluation with Drill Exercise Performance (DEP) on February 9, 2021

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

IE01: Unplanned Scrams per 7000 Critical Hours Sample (IP Section 03.01) (2 Samples)

- (1) Unit 1 January 1 – December 31, 2020
- (2) Unit 2 January 1 – December 31, 2020

IE03: Unplanned Power Changes per 7000 Critical Hours Sample (IP Section 03.02) (2 Samples)

- (1) Unit 1 January 1 – December 31, 2020
- (2) Unit 2 January 1 – December 31, 2020

IE04: Unplanned Scrams with Complications (USwC) Sample (IP Section 03.03) (2 Samples)

- (1) Unit 1 January 1 – December 31, 2020
- (2) Unit 2 January 1 – December 31, 2020

71152 - Problem Identification and Resolution

Annual Follow-up of Selected Issues (IP Section 02.03) (1 Sample)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issues:

- (1) Unit 1 reactor coolant system pressure boundary leak

71153 – Follow-up of Events and Notices of Enforcement Discretion

Event Report (IP Section 03.02) (1 Sample)

The inspectors evaluated the following licensee event reports (LERs):

- (1) LER 05000266/2020-001, Reactor Coolant System Pressure Boundary Leak on Steam Generator Bowl Drain Line Results in Operation Prohibited by Technical Specifications (ADAMS Accession No. ML20330A283). The inspection conclusions associated with this LER are documented in this report under Inspection Results Section 71153 Follow-Up of Events and Notices of Enforcement Discretion.

INSPECTION RESULTS

Improper Field Change Resulted in Reactor Coolant Pressure Boundary Leakage			
Cornerstone	Significance	Cross-Cutting Aspect	Report Section
Initiating Events	Green NCV 05000266/2021001-01 Open/Closed	[H.4] - Teamwork	71153

<p>A self-revealed Green finding and associated Non-cited Violation (NCV) of Title 10 CFR Part 50, Appendix B, Criteria III, "Design Control," and Technical Specification (TS) 3.4.13, "RCS [Reactor Coolant System] Operational Leakage," was identified when the licensee failed to subject field changes to design control measures that were commensurate with those applied to the original design. Specifically, the licensee made a field change when installing a pipe to the Unit 1 B steam generator and did not subject it to the applicable design control measures. This resulted in RCS pressure boundary leakage.</p>			
<p><u>Description:</u></p> <p>On February 20, 2020, the licensee noted an increasing trend on the Unit 1 containment air particulate monitor. After chemical analysis, the licensee found isotopes that were not normally present in containment, which indicated a possible primary leak. During a Unit 1 containment entry on April 5, 2020, an active leak was identified on the B steam generator loop platform near the bottom of the steam generator. The exact source of the leak could not be identified due to significant dose rates on the platform. The licensee made several reasonable attempts to confirm the source of the leakage. However, the licensee was unsuccessful because the leak location was inaccessible while operating the unit. The licensee reasonably assumed the source was packing leakage from valve 1RC-526B, "HX-1B SG Channel Head Drain," because the valve was in the general area of the leak and had a history of packing leakage. The unidentified leakage rose to a maximum of 0.143 gallons per minute which was less than the licensee's adverse condition monitoring plan of 0.5 gallons per minute and Technical Specification 3.4.13.b requirement of 1 gallon per minute for unidentified leakage.</p> <p>However, on October 3, 2020, after shutting down Unit 1 for a refueling outage, the licensee conducted an inspection of the B steam generator and determined that the leakage was pressure boundary leakage from a weld on the steam generator bowl drain piping. Point Beach Unit 1 TS 3.4.13, "RCS Operational Leakage," requires that pressure boundary leakage be limited to zero while the unit is in Modes 1, 2, 3, or 4. When pressure boundary leakage exists, Condition B requires the unit to be in Mode 3 within 6 hours and Mode 5 within 36 hours. The licensee did not meet this requirement because they believed that the leakage was packing leakage when operating the unit.</p> <p>The licensee performed a failure investigation and determined that valve 1RC-526B was close to the harmonics range of the reactor coolant pump, resulting in increased vibrations of the pipe and valve. After further investigation, the licensee determined that the length of pipe that attached 1RC-526B to the steam generator was not in conformance with the design. Specifically, on October 20, 2017, the pipe was installed with a length of 5.095 inches while design change drawing 2D31917 required the length of pipe to be 4.5 +0.12/-0.00. The licensee also found that, in work order 00390455-01, the installer made a note stating that the length of the pipe was 5.22 inches.</p> <p>Procedure EN-AA-100, Revision 6, "Design Control Program," stated that the Design Control Program requirements applied to field changes. It also stated that field change requests must be promptly followed by an engineering change revision to incorporate the change. This program requirement was not followed.</p> <p>Corrective Actions: The licensee's immediate corrective actions were to remove the valve and install a pipe cap in place. The licensee also did a resonant bump test of the newly installed pipe cap to test the frequency of the pipe. The licensee grounded out indications on</p>			

the bowl drain welds and on the steam generator surface. At the time of this inspection, the licensee also planned to revise the Field Activity Monitoring Plan procedure (i.e., PR-AA-100-1004) and form. The licensee requested the vendor to initiate an action request in the vendor's corrective action program, stressing the importance of meeting the applicable design criteria and of obtaining responsible engineer concurrence for any deviations, and to provide the results of their investigation to the Point Beach Engineering managers. The licensee also initiated a work order to obtain operating vibration data and incorporate it into an engineering model.

Corrective Action References: AR 02345306, AR 02346204, AR 02360109, AR 02370320, AR 02370198

Performance Assessment:

Performance Deficiency: The inspectors determined that the failure to subject field changes to design control measures that were commensurate with those applied to the original design was a violation of Title 10 CFR Part 50, Appendix B, Criteria III, and was a performance deficiency, which resulted in pressure boundary leakage. Specifically, the licensee made a field change when installing a pipe to the Unit 1 B Steam Generator and did not subject it to the applicable design control measures.

Screening: The inspectors determined the performance deficiency was more than minor because it was associated with the Design Control attribute of the Initiating Events cornerstone and adversely affected the cornerstone objective to limit the likelihood of events that upset plant stability and challenge critical safety functions during shutdown, as well as power operations. Specifically, the licensee's failure to subject the field change to design control measures resulted in RCS pressure boundary leakage from a weld failure.

Significance: The inspectors assessed the significance of the finding using Appendix A, "The Significance Determination Process (SDP) for Findings At-Power." Specifically, the inspectors determined the finding was of very low safety significance (Green) because the finding did not result in exceeding the RCS leak rate for a small break loss of coolant accident (LOCA), or likely affect other systems used to mitigate a (LOCA).

Cross-Cutting Aspect: H.4 - Teamwork: Individuals and work groups communicate and coordinate their activities within and across organizational boundaries to ensure nuclear safety is maintained. The finding had a cross-cutting aspect of Teamwork because the individuals installing the pipe did not communicate and coordinate the field change with the organization responsible for controlling the design of the pipe.

Enforcement:

Violation: Title 10 CFR Part 50, Appendix B, Criteria III, "Design Control," requires, in part, that design changes, including field changes, shall be subject to design control measures commensurate with those applied to the original design. It also requires these measures to include provisions to assure that appropriate quality standards are specified and included in design documents, and that deviations from such standards are controlled.

When the unit is in Modes 1, 2, 3, or 4, Point Beach Unit 1 TS 3.4.13, "RCS Operational Leakage," requires, in part, "No pressure boundary leakage." When pressure boundary leakage exists, the associated Condition B requires the unit to be in Mode 3 within 6 hours and Mode 5 within 36 hours.

Contrary to the above, on October 20, 2017, the licensee failed to subject a field change to design control measures that were commensurate with those applied to the original design and control deviations from quality standards specified in design documents. Specifically, the licensee made a field change when installing a pipe to the Unit 1 'B' steam generator with a length that differed from the standard specified in design change drawing 2D31917 and failed to control this deviation. As a result, the valve attached to this pipe (i.e., 1RC-526B) was close to the harmonics range of the reactor coolant pump, which led to increased vibrations, crack formation, and an unrecognized pressure boundary leakage. The pressure boundary leakage also resulted in a TS 3.4.13 violation from approximately February 20 to October 3, 2020.

Enforcement Action: This violation is being treated as a non-cited violation, consistent with Section 2.3.2 of the Enforcement Policy.

EXIT MEETINGS AND DEBRIEFS

The inspectors confirmed that proprietary information was controlled to protect from public disclosure.

- On April 5, 2021, the inspectors presented the 71111.11A review of results of the 2020 written and operating exam inspection results to Mr. A. Moore, Operations Training Supervisor, and other members of the licensee staff.
- On April 13, 2021, the inspectors presented the integrated inspection results to Mr. M. Strobe, Site Vice President, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.01	Procedures	AOP 13C	Severe Weather Conditions	50
71111.04	Drawings	110E018 Sheet 3	P&ID Auxiliary Coolant System	45
		110E029 Sheet 3	P&ID Auxiliary Coolant System	47
		M-209 Sheet 15	P&ID Starting Air System Diesel Generator Building	13
		M-219 Sheet 3	P&ID Fuel Oil System Diesel Generator Building	17
	Miscellaneous	1-CL-CC-001	Component Cooling Unit 1	20
		2-CL-CC-001	Component Cooling Unit 2	21
		CL 11A G-04	G-04 Diesel Generator Checklist	12
71111.05	Fire Plans	PFP-0-CB	Pre-Fire Plan Control Building Elev 8 ft, 26 ft, 44 ft and 66 ft	3
		PFP-0-PAB 26	Pre-Fire Plan Unit 1 & Unit 2 Auxiliary Building 26 ft	1
		PFP-0-PAB-8	Primary Auxiliary Building Elevations 8' and Below	2
	Miscellaneous		Hourly Fire Rounds on 3/8/2021 for the PAB	
			Every Four Hours Fire Rounds on 3/8/2021 for the PAB	
			Hourly Fire Rounds on 3/9/2021 for the Turbine Hall and Miscellaneous Areas	
			Every Four Hours Fire Rounds on 3/9/2021 for the Turbine Hall and Miscellaneous Areas	
	Procedures	AOP-40	Response to Fire	5
71111.11Q	Miscellaneous		Point Beach Nuclear Plant Event Notification Worksheet	02/09/2021
			Nuclear Accident Reporting Form (NARS)	02/09/2021
		PBN LOC 000 017E	NRC Annual Simulator Exam	6
	Procedures	OP 1C	Startup to Power Operation Unit 2	48
71111.12	Corrective Action Documents	AR 2343854	K-4C Diesel Driven Air Compressor for G-03 Failed to Start	02/06/2020
		AR 2350438	K-5C G-03 Electric Air Compressor Interstage Relief is Lifting	03/30/2020
		AR 2351769	K-5C G-03 Electric Air Compressor Interstage Relief is Lifting	04/08/2020
		AR 2352127	Bearing Chatter	04/10/2020
		AR 2358472	PS-6353C Pressure Switch As-Found OOT, Adjusted As-Left SAT	06/02/2020

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		AR 2358475	PS-6353B As-Found OOT, Adjusted and As-Left Within Limits	06/02/2020
		AR 2371249	G-02 EDG Air Bank Pressure Drop	10/11/2020
		AR 2383886	K-4D-D Failed to Start	02/11/2021
		AR 2384706	PS-3058A Is Not Holding its Calibration	02/19/2021
		AR 2384708	PS-3058B Is Not Holding its Calibration	02/19/2021
		AR 2387232	External OE Eval Needed - RS/RG2 EAL Level Validation	03/18/2021
	Drawings	M-209 Sheet 12	P&ID Emergency Diesel Air Starting System	25
		M-209 Sheet 14	P&ID Starting & Service Air System Diesel Generator Building	13
		M-209 Sheet 15	P&ID Starting Air System Diesel Generator Building	13
	Engineering Changes	EC 276803	NRC Order Fukushima Strategy - Spent Fuel Pool Instrumentation Upgrade	12/16/2014
	Miscellaneous		Adverse Condition Monitoring and Contingency Plan - G-03 Air Bank Pressure	0
			Adverse Condition Monitoring and Contingency Plan - G-03 Air Bank Pressure	1
			Operators Logs	3/29/2020 - 4/25/2020
			G-03 Diesel Air Pressure Trend	03/28/2020 - 04/26/2020
	Procedures	ICP 03.017	Calibration of Spent Fuel Pool Level Instrumentation Systems	2
		WNA-TP-04709-GEN	Spent Fuel Pool Instrumentation System Calibration Procedure	4
	Work Orders	WO 40690209	ICP 3.17 (Fall Occurrence) - Spent Fuel Pool Level	09/12/2020
71111.13	Corrective Action Documents Resulting from Inspection	AR 2384503	Work Light Attached to U2 SI Piping	02/18/2021
		AR 2384545	Improper Wheel Chock Usage	02/18/2021
	Miscellaneous		PBNP Shutdown Safety Assessment and Fire Inspection Checklist	02/18/2021
			PBNP Shutdown Safety Assessment and Fire Inspection Checklist	03/04/2021

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			PBNP Shutdown Safety Assessment and Fire Inspection Checklist	03/17/2021
			Point Beach Elevated Risk Focus Activities for the 2021 Dry Fuel Storage Load Campaign / Spent Fuel Pool Fuel Assemblies Inspections / Fuel Handling and Fuel Moves in SFP, Inspect Fuel Cell Inserts	
	Procedures	NP 10.3.7	On-Line Safety Assessment	43
71111.15	Corrective Action Documents	AR 2380568	Z-2000B FLEX Generator Failed to Start During Quarterly Run	01/11/2021
		AR 2384245	Change in Seal Leakage Flow on 1P-11B CC Pump	02/16/2021
		AR 2385174	Pressure Trending Low on 2T-233A and B (MFIV Accum's)	02/25/2021
		AR 2385693	Potential Thru Wall Leak on CCW Identified	03/02/2021
		AR 2386499	CCW Piping Analysis Not Updated for Removed Pipe Penetration	03/10/2021
71111.19	Corrective Action Documents	AR 2385869	1P-53 MDAFW Pump Recirc Flow Below Admin Limit During IT 400	03/04/2021
	Work Orders	WO 40717267-03	G-03-VARM; Ops PMT / RTS	03/02/2021
		WO 40729869	FS-4133C Perform Visual Inspection of Flow Switch	03/23/2021
		WO 40730832-02	G-02, Post Maint Testing per RMP 9043-27A	02/25/2021
		WO 40730832-03	G-02, Ops PMT/RTS (TS-82)	02/25/2021
		WO 40731573-03	Ops PMT/RTS	03/05/2021
		WO 40750226	W-013B1 Inspect Fan and Lube Bearings	03/26/2021
71111.22	Procedures	1ICP 02.003A	Reactor Protection Logic Train A 31 Day Surveillance Test	14
		1ICP 02.005A	Engineered Safety Features System Logic Train A 31 Day Staggered Actuation Logic Test	15
		IT 05 Train B	Train B Containment Spray Pump and Valves Unit 1	7
		TS 4	Main Turbine Stop and Governor Valves with Turbine Trip Test (Biannual) Unit 2	53
71114.06	Miscellaneous		Nuclear Accident Reporting Form (NARS) - Drill	02/09/2021
			Event Notification Worksheet (Drill)	02/09/2021
71151	Miscellaneous		Performance Indicators; Units 1 and 2; Unplanned Power Changes Per 7000 Critical Hours	01/01/2020 - 12/31/2020

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Performance Indicators; Units 1 and 2; Unplanned Scrams with Complications	01/01/2020 - 12/31/2020
			Performance Indicators; Units 1 and 2; Unplanned Scrams Per 7000 Critical Hours	01/01/2020 - 12/31/2020
			Point Beach PI Reporting Data; Units 1 and 2; Unplanned Power Changes Per 7,000 Critical Hours	01/01/2020 - 12/31/2020
			Point Beach PI Reporting Data; Units 1 and 2; Unplanned Scrams Per 7,000 Critical Hours	01/01/2020 - 12/31/2020
			Point Beach PI Reporting Data; Units 1 and 2; Unplanned Scrams with Complications Per 7,000 Critical Hours	01/01/2020 - 12/31/2020
71152	Corrective Action Documents	AR 2370320	1RC-526B Liquid Penetrant Indications	10/05/2020
	Miscellaneous	NUC PRJ NCC 002	Field Activity Monitoring Plan (FAMP) Training Instruction	0
		NUC PRJ NCC 003	Contract Coordinator Responsibilities & Oversight of Supplemental Personnel Using a Field Activity Monitoring Plan (FAMP)	03/30/2021
		PR-AA-100-1004-F01	Detailed Monitoring Plan	5
	Procedures	PR-AA-100-1004	Field Activity Monitoring Plan	7
71153	Corrective Action Documents	AR 02345306	Increasing Trend on 1RE-211 Particulate Monitor	02/20/2020
		AR 02346204	Unit 1 Enters Action Level 1 for RCS Unidentified Leakage	02/26/2020
		AR 02360109	Shortfall in Documentation of Considerations	06/17/2020
		AR 02370198	1RC-526B, As-Found Boric Acid Leak	10/03/2020
		AR 02370320	1RC-526B Liquid Penetrant Indications	10/04/2020
	Drawings	2D31917	Steam Generator Channel Head Drain System	5
	Engineering Changes	295378	Replace 1RC-526A and 1RC-526B With A Pipe Cap	1
	Miscellaneous	Event Notification 54930	Reactor Coolant System Pressure Boundary Leakage	10/03/2020
		Point Beach Unit 1 LER 2020-001	Reactor Coolant System Pressure Boundary Leak on Steam Generator Bowl Drain Line Results in Operation Prohibited by Technical Specifications	11/25/2020
	Procedures	EN-AA-100	Design Control Program	6

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		EN-AA-205-1100	Design Change Packages	31
		PR-AA-100-1004	Field Activity Monitoring Plan	7
	Work Orders	WO 00390455 01	1HX-001B / Nozzle Repair of SG Channel Head Drain	10/20/2017
		WO 40713753 26	Primary Leakage in U1 Containment B Loop	10/11/2020
		WO 40713753 28	Primary Leakage in U1 Containment B Loop	10/16/2020
		WO 40713753 29	Primary Leakage in U1 Containment B Loop	10/18/2020