

10 CFR 50.55a

April 12, 2021

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001R. E. Ginna Nuclear Power Plant
Renewed Facility Operating License Nos. DPR-18
NRC Docket No. 50-244

Subject: One-Time Interval Extension for Supplemental Position Indication Testing

In accordance with 10 CFR 50.55a, "Codes and standards," paragraph (z)(2), Exelon Generation Company, LLC (Exelon), requests NRC approval of a proposed relief request associated with the Inservice Testing (IST) Program for the R. E. Ginna Nuclear Power Plant. This request proposes a one-time extension to the 2-year Supplemental Position Indication (SPI) testing frequency for fifteen IST program valves.

The timing for implementation was not clear; therefore, several position verification tests were performed that were not supplemented with alternate indication to demonstrate obturator position. Compliance with the requirements of ISTC-3700 will create undue hardship without a compensating increase in the level of quality or safety. The earliest surveillance testing of these valves is due to expire on April 28, 2021; therefore, Exelon requests your review and verbal approval of this request by April 27, 2021.

There are no regulatory commitments contained in this letter.

If you have any questions, please contact Tom Loomis at 610-765-5510.

Respectfully,



David T. Gudger
Senior Manager - Licensing and Regulatory Affairs
Exelon Generation Company, LLC

Attachment: Relief Request GR-03 - One-Time Interval Extension for SPI Testing

cc: Regional Administrator - NRC Region I
NRC Senior Resident Inspector - Ginna
NRC Project Manager - Ginna

ATTACHMENT

**Relief Request GR-03 - One-Time Interval Extension for Supplemental Position Indication
Testing**

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**Request for Relief GR-03
One-Time Interval Extension for Supplemental Position Indication Testing
In Accordance with 10 CFR 50.55a(z)(2)**

1. ASME Code Component(s) Affected:

Pneumatically operated and motor operated valves with remote position indications were not verified in the open and/or closed position using a supplemental indication during the Ginna Nuclear Power Plant refueling outage conducted in April 2020. This includes the following valves:

<u>Valve ID</u>	<u>Description</u>	<u>Code Class</u>	<u>Category</u>	<u>Type</u>	<u>Test Expiration</u>
310	Excess Letdown Loop A Cold to Heat Exchanger Valve	1	B	AOV	5/2/2021
371	Letdown Isolation Valve Residual Heat Removal to Non-Regenerative Heat Exchanger	2	A	AOV	5/2/2021
386	Reactor Coolant Pump A and B Seal Bypass Valve	2	B	AOV	5/2/2021
5392	Instrument Air to Containment Isolation Valve	2	A	AOV	5/2/2021
835A	Safety Injection Accumulator A Fill Valve	2	B	AOV	5/2/2021
835B	Safety Injection Accumulator B Fill Valve	2	B	AOV	5/2/2021
839A	Safety Injection Accumulator A Test Valve	2	B	AOV	5/2/2021
840A	Safety Injection Accumulator B Test Valve	2	B	AOV	5/2/2021
844A	Safety Injection Accumulator A Drain Valve	2	B	AOV	5/2/2021
844B	Safety Injection Accumulator B Drain Valve	2	B	AOV	5/2/2021
875A	Upper Containment Spray Charcoal Filter Dousing Valve	2	B	MOV	5/6/2021
875B	Upper Containment Spray Charcoal Filter Dousing Valve	2	B	MOV	5/6/2021
876A	Lower Containment Spray Charcoal Filter Dousing Valve	2	B	MOV	5/6/2021
876B	Lower Containment Spray Charcoal Filter Dousing Valve	2	B	MOV	5/6/2021
966A	Pressurizer Steam Space Sample Containment Isolation Valve	2	A	AOV	4/28/2021

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2. Applicable ASME OM Code Edition:

American Society of Mechanical Engineers (ASME) Code for Operation and Maintenance of Nuclear Power Plants (OM Code), 2012 Edition with no Addenda.

3. Applicable Code Requirements:

ISTC-3700, *Position Verification Testing*, states: "Valves with remote position indicators shall be observed locally at least once every 2 years to verify that valve operation is accurately indicated. Where practicable, this local observation should be supplemented by other indications such as use of flow meters or other suitable instrumentation to verify obturator position. These observations need not be concurrent. Where local observation is not possible, other indications shall be used for verification of valve operation."

10 CFR 50.55a(b)(3)(xi) *OM condition: Valve Position Indication* states: "When implementing paragraph ISTC-3700, "Position Verification Testing," in the ASME OM Code, 2012 Edition through the latest edition and addenda of the ASME OM Code incorporated by reference in paragraph (a)(1)(iv) of this section, licensees shall verify that valve operation is accurately indicated by supplementing valve position indicating lights with other indications, such as flow meters or other suitable instrumentation to provide assurance of proper obturator position for valves with remote position indication within the scope of Subsection ISTC including its mandatory appendices and their verification methods and frequencies."

As required by 10 CFR 50.55a(b)(3)(xi), ISTC-3700 requirements became applicable on January 1, 2020 when the R. E. Ginna Nuclear Power Plant (Ginna) Inservice Testing (IST) Program was updated to the 2012 Edition of the ASME OM Code.

4. Reason for Request:

Pursuant to 10 CFR 50.55a, *Codes and standards*, paragraph (z)(2), compliance with the requirements of ISTC-3700 will create undue hardship without a compensating increase in the level of quality or safety. Compliance will require a plant shutdown to perform a SPI for the valves listed in Section 2.

The most-limiting Supplemental Position Indication (SPI) testing interval for these valves ends on April 28, 2021, based on the 2-year interval that came into effect on January 1, 2020 and considering the use of the 6-month interval extension allowed by ASME OM Code Case OMN-20, Inservice Test Frequency. The position verification with SPI requires the valves to be exercised in the open and closed direction and the valve's position verified by other indications such as use of flow meters or other suitable instrumentation to verify obturator position. The timing for implementation was not clear; therefore, several position verification tests were performed that were not supplemented with alternate indication to demonstrate obturator position.

This relief request proposes a one-time interval extension of eight months (December 28, 2021) which will allow testing during the next refueling outage which is scheduled to begin October 4, 2021. This extension will avoid a forced plant shutdown to achieve refueling conditions to permit safely stroking the identified valves.

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5. Proposed Alternative and Basis for Use:

As a proposed alternative to the 2-year SPI testing frequency required by ISTC-3700, this relief request proposes a one-time interval extension of eight months (December 28, 2021) which will allow testing during the next refueling outage which is scheduled to begin October 4, 2021. The listing of valves provided in Section 2 have satisfactorily passed their IST examinations for the past 10 years.

Any component degradation identified prior to the next outage will be evaluated per the Corrective Action Program for operability concerns and compliance with the IST Program and Technical Specifications.

The identified valves have demonstrated exceptional performance and reliability. The subject valves have no history of stem-disc separation. This was confirmed by an internal maintenance history search and by an INPO Industry Reporting Information System (IRIS) search for each valve make and model and review of EPRI Technical Report 3002019621 (Reference 1). The historical performance provides assurance of the valves' capabilities to perform their design function(s) within the eight-month SPI interval extension. The past and current performance of each identified valve and the one-time interval extension bounded by test frequencies in existing regulations provide an acceptable level of quality and safety.

6. Duration of Proposed Alternative:

The proposed alternative is a one-time interval extension of eight months for the Ginna SPI testing of the identified valves, with an end date of December 28, 2021.

7. Precedent:

None

8. References:

1. EPRI Technical Report 3002019621, Susceptibility of Valve Applications to Failure of the Stem-to-Disk Connection