



South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

March 11, 2021  
NOC-AE-20003775  
10 CFR 50.90  
STI: 35104969

ATTN: Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

South Texas Project  
Units 1 & 2

Docket Nos. STN 50-498, STN 50-499

License Amendment Request to Revise Technical Specification 3.6.3 to Append a Note and  
Remove the Index from Technical Specifications

In accordance with the provisions of 10 CFR 50.90, STP Nuclear Operating Company (STPNOC) is submitting a request for an amendment to the Technical Specifications for South Texas Project (STP), Units 1 and 2.

The proposed amendment would modify Technical Specifications by adding a note to Limiting Condition for Operation (LCO) 3.6.3 allowing for penetration flow paths to be unisolated intermittently under administrative controls. Additionally, the proposed change will remove the Index from the Technical Specifications and place them under licensee control. The proposed amendment does not involve a significant hazards consideration under the standards set forth in 10 CFR 50.9(c).

The Enclosure to this letter provides a description of the proposed change. Marked-up and retyped (clean) Technical Specification pages are provided in Enclosure Attachments 1 and 2, respectively. A marked-up copy of the Technical Specification Bases is provided for information only in Enclosure Attachment 3.

STPNOC requests approval of the proposed license amendment by April 01, 2022. STPNOC will implement the amendment within 90 days of the NRC approval date.

In accordance with 10 CFR 50.91(b), "State Consultation," STPNOC is notifying the State of Texas of this license amendment request by transmitting a copy of this letter and Enclosure to the designated State Official. The proposed amendment has been reviewed and approved by the STPNOC Plant Operations Review Committee and has undergone an independent organizational unit review. There are no regulatory commitments in this amendment request.

If there are any questions or if additional information is needed, please contact Nic Boehmisch at (361) 972-8172 or me at (361) 972-7344.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on March 11, 2021

James Connolly  
Executive VP and CNO

Enclosure: Evaluation of the Proposed Change

cc:

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**ENCLOSURE**

**Evaluation of the Proposed Change**

Subject: License Amendment Request to Revise Technical Specifications by adding a note to Limiting Condition for Operation (LCO) 3.6.3 allowing for penetration flow paths to be unisolated intermittently under administrative controls and to remove the Index from Technical Specifications

- 1 SUMMARY DESCRIPTION
- 2 DETAILED DESCRIPTION
- 3 TECHNICAL EVALUATION
- 4 REGULATORY EVALUATION
- 5 ENVIRONMENTAL CONSIDERATION
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**ATTACHMENTS:**

1. Technical Specification Pages Markup
2. Retyped Technical Specification Pages
3. Technical Specification Bases Pages Markup (Information Only)

## **1 SUMMARY DESCRIPTION**

The proposed amendment would modify Technical Specifications by adding a note to Limiting Condition for Operation (LCO) 3.6.3 allowing for penetration flow paths to be unisolated intermittently under administrative controls. The addition of this note will allow flexibility in testing and maintenance activities without any significant decrease in safety.

Additionally, the proposed change will remove the Index from the Technical Specifications and place them under licensee control.

## **2 DETAILED DESCRIPTION**

Technical Specification LCO 3.6.3 Containment Isolation Valves will be modified by the following note "NOTE: Penetration flow paths (except for the Containment Purge flow path) may be unisolated intermittently under administrative controls." The note will be applicable to LCO 3.6.3. This note will be numbered 1. The page already contains two notes designated by \* and \*\*. The designation of these notes will be changed to 2 and 3 respectively as an administrative enhancement to improve the readability of the Technical Specifications.

A discussion of acceptable administrative controls for this note will be added to the Bases and include (1) stationing an operator, who is in constant communication with the control room, at the valve controls, (2) instructing this operator to close the valve in an accident situation, and (3) assuring that environmental conditions will not preclude access to close the valve and that this action will prevent the release of radioactivity outside the containment.

The proposed change will also remove the Index from the Technical Specifications and place it under licensee control.

The Index for the Technical Specifications is not being eliminated. Following approval of this license amendment request, the responsibility for maintenance and issuance of updates to the Technical Specifications Index will transfer from the NRC to STP Nuclear Operating Company (STPNOC). The Index will no longer be included in the Technical Specifications and as such will no longer be part of the Technical Specifications (Appendix A to the Operating License). An index for the Technical Specifications will be maintained under STPNOC (licensee) control. The index will be issued by STPNOC in conjunction with the implementation of NRC-approved Technical Specification amendments.

## **3 TECHNICAL EVALUATION**

The proposal adds to Technical Specification LCO 3.6.3 a footnote that states, "Penetration flow paths (except for the Containment Purge flow path) may be unisolated intermittently under administrative controls." In addition, the Technical Specification Bases section will be changed to state the considerations provided in Generic Letter (GL) 91-08 that constitute acceptable administrative controls. These considerations include (1) stationing an operator, who is in constant communication with control room, at the valve controls, (2) instructing this operator to close the valve in an accident situation, and (3) assuring that environmental conditions will not preclude access to close the valve and that this action will prevent the release of radioactivity outside the containment.

The addition of this footnote will allow flexibility in testing and maintenance activities without any significant decrease in safety. The administrative controls, as described in the Bases, will ensure that the penetration can be rapidly isolated when a need for containment isolation is indicated. Due to the size of the Containment Purge line penetrations and the fact that those



penetrations exhaust directly from the containment atmosphere to the environment, these valves may not be opened under administrative control.

The Westinghouse Standard Technical Specifications (NUREG-1431), Reference 6.1, allows penetration flow paths to be unisolated intermittently under administrative controls. Therefore, this proposed change to the LCO is consistent with the guidance in GL 91-08 and the Standard Technical Specifications.

The proposed removal of the Index from Technical Specifications is an administrative non-technical change. The Index does not meet the criteria specified in 10 CFR 50.36 requiring its inclusion within a plant's Technical Specifications. The Index references where specific Technical Specification sections can be found throughout the Technical Specifications but does not contain technical information required by 10 CFR 50.36. Since the Index does not include information required by 10 CFR 50.36 to be reviewed by the NRC staff, inclusion of an Index within the Technical Specifications is optional and is not required by the regulation. Removal of the Index from the Technical Specifications constitutes an administrative change and is therefore acceptable.

Additionally, the "Writer's Guide for Plant-Specific Improved Technical Specifications (ITS)" (Reference 2) was reviewed for guidance. The Writer's Guide refers to the index as "Technical Specification Front Matter," which also includes the Title Page and List of Effective Pages. The writer's guide describes the Front Matter as "not part of the Technical Specifications." Since the removal of the Index from Technical Specifications is an administrative non-technical change, a technical evaluation is not necessary.

#### **4 REGULATORY EVALUATION**

##### **4.1 Applicable Regulatory Requirements/Criteria**

Penetration flow paths, except for the Containment Purge flow path, may be opened on an intermittent basis under administrative controls. Per GL 91-08, acceptable administrative controls for opening a containment isolation valve include (1) stationing an operator, who is in constant communication with the control room, at the valve controls, (2) instructing this operator to close the valve in an accident situation, and (3) assuring that environmental conditions will not preclude access to close the valve and that this action will prevent the release of radioactivity outside the containment.

10 CFR 50.36(a) states that "Each applicant for a license ... shall include in his application proposed technical specifications in accordance with the requirements of this section. A summary statement of the bases or reasons for such specifications, other than those covering administrative controls, shall also be included in the application, but shall not become part of the technical specifications." Similar to the Technical Specification Bases, the Index does not meet the criteria specified in 10 CFR 50.36 for inclusion within a plant's Technical Specifications.

STPNOC has evaluated this change against the applicable regulatory requirements as described above. Based on this, there is reasonable assurance that the health and safety of the public, following approval of this change, will remain unaffected.

##### **4.2 Precedent**

By letter dated February 23, 1993, as supplemented May 4, 1993, Duke Power submitted a license amendment request to add a note allowing penetration flow paths to be unisolated intermittently under administrative controls. This amendment was issued by the NRC by letter dated September 16, 1993, Reference 6.4.

By letter dated December 12, 2018, South Carolina Electric & Gas Company submitted a license amendment request to remove the index from Technical Specifications. This amendment was issued by the NRC by letter dated April 10, 2019, Reference 6.6.

#### 4.3 No Significant Hazards Consideration Determination

South Texas Project (STP) has evaluated whether a significant hazards consideration is involved with the proposed amendment by focusing on the three standards set forth in 10 CFR 50.92, "Issuance of amendment," as discussed below:

1) Does the proposed amendment involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The proposed change to Technical Specification Limiting Condition for Operation (LCO) 3.6.3 would allow penetration flow paths (except for the Containment Purge flow path) to be unisolated intermittently under administrative controls. This does not involve a significant increase in the probability or consequences of an accident, because: (1) this situation is expected to occur for only very short durations, and the probability of an accident occurring during the exact period of time that a penetration flow path is unisolated is remote, and (2) the use of administrative controls can reasonably be expected to prevent a significant increase in consequences during any accident that may occur during this period of time. In addition, this provision is consistent with the provisions in the Standard Technical Specifications for containment isolation valves.

The removal of the Index is an administrative non-technical change. The proposed change does not adversely affect accident initiators or precursors nor alter the design assumptions, conditions, or configurations of the facility. The proposed change does not alter or prevent the ability of structures, systems, and components (SSCs) to perform their intended function to mitigate the consequences of an initiating event within the assumed acceptance limits.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2) Does the proposed amendment create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

Operation of STP in accordance with the proposed Technical Specifications will not create any failure modes not bounded by previously evaluated accidents. Consequently, this change will not create the possibility of a new or different kind of accident from any accident previously evaluated. In addition, this provision is consistent with the provisions in the Standard Technical Specifications for containment isolation valves

Removal of the Index will not alter the design requirements of any SSC or its function during accident conditions. No new or different accidents result from the proposed changes. The changes do not involve a physical alteration of the plant or any changes in methods governing normal plant operation. The changes do not alter assumptions made in the safety analysis.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3) Does the proposed amendment involve a significant reduction in a margin of safety?

Response: No.

The proposed Technical Specifications will continue to require operation within the same safety limits as the existing Technical Specifications. The proposed administrative requirements will maintain adequate safety margin during the short duration the valves are unisolated. The proposed change is consistent with the guidance provided in Generic Letter (GL) 91-08 and the Standard Technical Specifications. Therefore, existing margins of safety are maintained.

Removing the Index does not alter the manner in which safety limits, limiting safety system settings or limiting conditions for operation are determined. The safety analysis acceptance criteria are not affected by these changes. The proposed changes will not result in plant operation in a configuration outside the design basis. The proposed changes do not adversely affect systems that respond to safely shutdown the plant and to maintain the plant in a safe shutdown condition.

Therefore, the proposed change does not involve a significant reduction in a margin of safety.

Based on the above, STP concludes that the proposed amendment does not involve a significant hazards consideration under the standards set forth in 10 CFR 50.92(c), and, accordingly, a finding of "no significant hazards consideration" is justified.

#### 4.4 Conclusions

In conclusion, based on the considerations discussed above, (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

### 5 ENVIRONMENTAL CONSIDERATION

A review has determined that the proposed amendment would change a requirement with respect to installation or use of a facility component located within the restricted area, as defined in 10 CFR 20, or would change an inspection or surveillance requirement. However, the proposed amendment does not involve (i) a significant hazards consideration, (ii) a significant change in the types or a significant increase in the amounts of any effluents that may be released offsite, or (iii) a significant increase in individual or cumulative occupational radiation exposure. Accordingly, the proposed amendment meets the eligibility criterion for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the proposed amendment.

### 6 REFERENCES

- 6.1 NUREG-1431, Volume 1, Revision 4.0, "Standard Technical Specifications Westinghouse Plants Revision 4.0 Volume 1, Specifications," Dated April 2012.
- 6.2 TSTF-GG-05-01 Revision 1, "Writer's Guide for Plant-Specific Improved Technical Specifications," Dated August 2010, ML12046A089.

- 6.3 Letter, J.W. Hampton to NRC, "Proposed Revision to Technical Specifications Remove List of Containment Penetrations," Dated February 23, 1993, ML16266A158 and ML16131A429.
- 6.4 Letter, NRC to J.W. Hampton, "Issuance of Amendments – Oconee Nuclear Station, Units 1, 2, and 3 (Tac Nos. M85934, M85935, M85936)," Dated September 16, 1993, ML012040008.
- 6.5 Letter, G. Lippard to NRC, "Request to Remove the Expired One-Time Extension to Surveillance Frequency 4.3.3.6 of the Core Exit Temperature Instrumentation and to Remove the Index from Technical Specifications," Dated December 12, 2018, ML18346A595.
- 6.6 Letter, NRC to G. Lippard, "Issuance of Amendment RE: Removes the Expired One-Time Extension to Surveillance Requirement 4.3.3.6 and Removes the Index from Technical Specifications (EPID L-2018-LLA-0568)," Dated April 10, 2019, ML19074A222.

Enclosure  
NOC-AE-20003775  
Attachment 1

**Attachment 1**

**Technical Specification Pages Markup**

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## CONTAINMENT SYSTEMS

### 3/4.6.3 CONTAINMENT ISOLATION VALVES

#### LIMITING CONDITION FOR OPERATION

3.6.3 The containment isolation valves shall be OPERABLE with isolation times less than or equal to the required isolation times.

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With one or more of the isolation valve(s) inoperable, maintain at least one isolation barrier<sup>(2)</sup> OPERABLE in each affected penetration that is open and within 24 hours:

- Restore the inoperable valve(s) to OPERABLE status, or
- Isolate each affected penetration by use of at least one deactivated automatic valve secured in the isolation position, or check valve with flow through the valve secured<sup>(1) (3)</sup>, or
- Isolate each affected penetration by use of at least one closed manual valve or blind flange<sup>(1)</sup>, or
- Apply the requirements of the CRMP.

Otherwise be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

4.6.3.1 The isolation valves shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair or replacement work is performed on the valve or its associated actuator, control or power circuit by performance of a cycling test, and verification of isolation time.

4.6.3.2 Each isolation valve shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at a frequency in accordance with the Surveillance Frequency Control Program by:

- Verifying that on a Phase "A" Isolation test signal, each Phase "A" isolation valve actuates to its

**<sup>(1)</sup> Penetration flow paths (except for Containment Purge flow paths) may be unisolated intermittently under administrative controls.**

purge and exhaust valve

- Verifying that on a Phase "B" Isolation test signal, each Phase "B" isolation valve actuates to its isolation position.
- Verifying that on a Phase "A" Isolation test signal, coincident with a low charging header pressure signal, that each seal injection valve actuates to its isolation position.

4.6.3.3 The isolation time of each power-operated or automatic valve shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

<sup>(2)</sup>An isolation barrier may either be a closed system (i.e., General Design Criteria 57 penetrations) or an isolation valve.

<sup>(3)</sup>A check valve may not be used to isolate an affected penetration flow path in which more than one isolation valve is inoperable or in which the isolation barrier is a closed system with a single isolation valve (i.e., General Design Criteria 57 penetration)

Enclosure  
NOC-AE-20003775  
Attachment 2

**Attachment 2**

**Retyped Technical Specification Pages**

## CONTAINMENT SYSTEMS

### 3/4.6.3 CONTAINMENT ISOLATION VALVES

#### LIMITING CONDITION FOR OPERATION

---

3.6.3 The containment isolation valves shall be OPERABLE with isolation times less than or equal to the required isolation times.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With one or more of the isolation valve(s) inoperable, maintain at least one isolation barrier<sup>(2)</sup> OPERABLE in each affected penetration that is open and within 24 hours:

- a. Restore the inoperable valve(s) to OPERABLE status, or
- b. Isolate each affected penetration by use of at least one deactivated automatic valve secured in the isolation position, or check valve with flow through the valve secured<sup>(1) (3)</sup>, or
- c. Isolate each affected penetration by use of at least one closed manual valve or blind flange<sup>(1)</sup>, or
- d. Apply the requirements of the CRMP.

Otherwise be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

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4.6.3.1 The isolation valves shall be demonstrated OPERABLE prior to returning the valve to service after maintenance, repair or replacement work is performed on the valve or its associated actuator, control or power circuit by performance of a cycling test, and verification of isolation time.

4.6.3.2 Each isolation valve shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at a frequency in accordance with the Surveillance Frequency Control Program by:

- a. Verifying that on a Phase "A" Isolation test signal, each Phase "A" isolation valve actuates to its isolation position;
- b. Verifying that on a Containment Ventilation Isolation test signal, each purge and exhaust valve actuates to its isolation position; and
- c. Verifying that on a Phase "B" Isolation test signal, each Phase "B" isolation valve actuates to its isolation position.
- d. Verifying that on a Phase "A" Isolation test signal, coincident with a low charging header pressure signal, that each seal injection valve actuates to its isolation position.

4.6.3.3 The isolation time of each power-operated or automatic valve shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

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<sup>(1)</sup> Penetration flow paths (except for Containment Purge flow paths) may be unisolated intermittently under administrative controls.

<sup>(2)</sup> An isolation barrier may either be a closed system (i.e., General Design Criteria 57 penetrations) or an isolation valve.

<sup>(3)</sup> A check valve may not be used to isolate an affected penetration flow path in which more than one isolation valve is inoperable or in which the isolation barrier is a closed system with a single isolation valve (i.e., General Design Criteria 57 penetration)

Enclosure  
NOC-AE-20003775  
Attachment 3

**Attachment 3**

**Technical Specification Bases Pages Markup  
(Information Only)**

## CONTAINMENT SYSTEMS

### BASES

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#### 3/4.6.2.3 CONTAINMENT COOLING SYSTEM (continued)

STPEGS has three groups of Reactor Containment Fan Coolers (RCFCs) with two fans in each group (total of six fans). Five fans are adequate to satisfy the safety requirements including single failure. If only one RCFC, out of six available, is inoperable, then there are no restrictions applied on the diesel generators by the RCFC condition and Action statement 3.8.1.1(d) (1) can be met. The fan cooler units are designed to remove heat from the containment during both normal operation and accident conditions. In the event of an accident, all fan cooler units are automatically placed into operation on receipt of a safety injection signal. During normal operation, cooling water flow to the fan cooler units is supplied by the non-safety grade chilled water system. Following an accident, cooling water flow to the fan coolers is supplied by the safety grade component cooling water system. The chilled water system supplies water at a lower temperature than that of the component cooling water system and therefore requires a lower flow rate to achieve a similar heat removal rate.

#### 3/4.6.3 CONTAINMENT ISOLATION VALVES

The OPERABILITY of the containment isolation valves ensures that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment and is consistent with the requirements of General Design Criteria 54 through 57 of Appendix A to 10 CFR Part 50. Containment isolation within the time limits specified for those isolation valves designed to close automatically ensures that the release of radioactive material to the environment will be consistent with the assumptions used in the analyses for a LOCA.

In the event one containment isolation valve in one or more penetrations is inoperable, and the inoperable valve(s) cannot be restored to OPERABLE status within 24 hours, the affected penetration(s) must be isolated. The method of isolation must include the use of at least one isolation barrier that cannot be adversely affected by a single active failure. Isolation barriers that meet this criterion are a closed and deactivated automatic isolation valve, a closed manual valve, a blind flange, or a check valve with flow through the valve secured (a check valve may not be used to isolate an affected penetration flow path in which more than one isolation valve is inoperable or in which the isolation barrier is a closed system with a single isolation valve). For a penetration flow path isolated in accordance with Action b or c, the device used to isolate the penetration should be the closest available one to containment and does not have to be a General Design Criterion containment isolation valve.

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In cases where multiple isolation valves use the same pipe going through the penetration and with one or more isolation valves inoperable, as long as the inoperable valve(s) is deactivated/manually isolated in its isolation position and the interconnecting isolation valves are operable, the appropriate Action statement is met. In these cases, the Action statement "Isolate each affected penetration..." means "Isolate each affected penetration flow path". (CR 97-908-1)

The TS 3.6.3 Limiting Condition for Operation and associated Actions are applicable to all Containment Isolation Valves (CIVs), including check valves and CIV penetrations. However, Surveillance Requirements (SRs) 4.6.3.1 and 4.6.3.2 are only applicable to power-operated CIVs. The isolation time verification required by SR 4.6.3.1 is only required for those valves where meeting the isolation time is necessary for supporting the safety analysis. The valves requiring isolation time verification are those valves with an isolation time specified in Table 16.1-1 of the Updated Final Safety Analysis Report. Containment Isolation Check Valve operability is verified in accordance with the requirements of SR 4.0.5 (Inservice Inspection and Testing) and SR 4.6.1.2 (Containment Leakage). Furthermore, as permitted by SR 4.0.5 and the Containment Leakage Rate Testing Program, specific containment isolation check valves (such as the Containment Spray Header Check Valves) have been exempted from certain Appendix J and Inservice Testing Requirements, and only require the satisfactory performance of an operational leak check to be returned to service. (CR 05-13492)

#### 3/4.6.4 NOT USED

## **INSERT**

Penetration flow paths, except for the Containment Purge flow paths, may be unisolated on an intermittent basis under administrative controls. Acceptable administrative controls for opening a containment isolation valve include (1) stationing an operator, who is in constant communication with the control room, at the valve controls, (2) instructing this operator to close the valve in an accident situation, and (3) assuring that environmental conditions will not preclude access to close the valve and that this action will prevent the release of radioactivity outside the containment.