



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**

REGION IV  
1600 EAST LAMAR BOULEVARD  
ARLINGTON, TEXAS 76011-4511

February 3, 2021

Mr. John Ferrick  
Site Vice President  
Entergy Operations, Inc.  
17265 River Road  
Killona, LA 70057

SUBJECT: WATERFORD STEAM ELECTRIC STATION, UNIT 3 – INTEGRATED  
INSPECTION REPORT 05000382/2020004

Dear Mr. Ferrick:

On December 31, 2020, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Waterford Steam Electric Station, Unit 3. On January 14, 2021, the NRC inspectors discussed the results of this inspection with you and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

John L. Dixon, Jr., Chief  
Reactor Projects Branch D  
Division of Reactor Projects

Docket No. 05000382  
License No. NPF-38

Enclosure:  
As stated

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WATERFORD STEAM ELECTRIC STATION, UNIT 3 – INTEGRATED INSPECTION REPORT  
05000382/2020004 – DATED FEBRUARY 3, 2021

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**U.S. NUCLEAR REGULATORY COMMISSION**  
**Inspection Report**

Docket Number: 05000382

License Number: NPF-38

Report Number: 05000382/2020004

Enterprise Identifier: I-2020-004-0002

Licensee: Entergy Operations, Inc.

Facility: Waterford Steam Electric Station, Unit 3

Location: Killona, LA 70057

Inspection Dates: October 1, 2020 to December 31, 2020

Inspectors: R. Alexander, Senior Emergency Preparedness Inspector  
D. Antonangeli, Health Physicist  
D. Childs, Resident Inspector  
J. Drake, Senior Reactor Inspector  
L. Flores, Reactor Inspector  
N. Greene, Senior Health Physicist  
R. Kopriva, Senior Reactor Inspector  
A. Patz, Senior Resident Inspector  
F. Ramirez Munoz, Senior Reactor Inspector

Approved By: John L. Dixon, Jr., Chief  
Reactor Projects Branch D  
Division of Reactor Projects

Enclosure

## **SUMMARY**

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Waterford Steam Electric Station, Unit 3, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

### **List of Findings and Violations**

No findings or violations of more than minor significance were identified.

### **Additional Tracking Items**

None.

## PLANT STATUS

Waterford, Unit 3, began the inspection period in a refueling outage. On October 31, 2020, Unit 3 was restarted and inadvertently tripped on November 2, 2020, due to an issue with the control element drive mechanism control system. Unit 3 was restarted on November 4, 2020, and achieved full power on November 15, 2020. On November 27, 2020, the licensee performed an emergent downpower to approximately 90 percent power to address governor valve oscillations and returned to full power on December 1, 2020. On December 11, 2020, the licensee initiated a rapid downpower into hot standby due to a steam leak near the turbine governor valves. On December 14, 2020, Unit 3 was restarted and remained at full power until the end of the inspection period.

## INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors performed plant status activities described in IMC 2515, Appendix D, "Plant Status," and conducted routine reviews using IP 71152, "Problem Identification and Resolution." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the Coronavirus Disease 2019 (COVID-19), resident inspectors were directed to begin telework and to remotely access licensee information using available technology. During this time, the resident inspectors performed periodic site visits each week; conducted plant status activities as described in IMC 2515, Appendix D, "Plant Status"; observed risk-significant activities; and completed on-site portions of IPs. In addition, resident and regional baseline inspections were evaluated to determine if all or portions of the objectives and requirements stated in the IP could be performed remotely. If the inspections could be performed remotely, they were conducted per the applicable IP. In some cases, portions of an IP were completed remotely and on-site. The inspections documented below met the objectives and requirements for completion of the IP.

## REACTOR SAFETY

### 71111.04 - Equipment Alignment

#### Partial Walkdown Sample (IP Section 03.01) (1 Sample)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) Emergency diesel generator A with emergency diesel generator B out of service for maintenance, on October 19, 2020

#### Complete Walkdown Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated system configurations during a complete walkdown of the shutdown cooling system, on October 20, 2020

#### 71111.05 - Fire Protection

#### Fire Area Walkdown and Inspection Sample (IP Section 03.01) (4 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) Fire Area RCB-001, reactor containment building general area, on October 15, 2020
- (2) Fire Area RAB 13-001, battery rooms, on October 30, 2020
- (3) Fire Area ROOF E-001, main steam isolation valve B, on November 3, 2020
- (4) Fire Area ROOF W-001, main steam isolation valve A, on November 3, 2020

#### 71111.08P - Inservice Inspection Activities (PWR)

#### PWR Inservice Inspection Activities Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors verified that the reactor coolant system boundary, steam generator tubes, reactor vessel internals, risk-significant piping system boundaries, and containment boundary are appropriately monitored for degradation and that repairs and replacements were appropriately fabricated, examined and accepted by reviewing the following activities from October 8, to November 20, 2020:

##### 03.01.a - Nondestructive Examination and Welding Activities.

- Eddy Current Examination, RCP 1A Motor Flywheel (Entire Flywheel), Report Number: ISI-ET-20-001
- Dye Penetrant Examination, RAB-Safeguards Pump Room B Containment Spray Anchor Attachment Weld, Report Number: ISI-PT-20-001
- Magnetic Particle Examination, Wet MSIV Area Main Steam Welded Support to 40-inch Pipe, Report Number: ISI-MT-20-004
- Visual Examination VT-3, RCB+46 West D-Ring Safety Injection Bolted Connection SG MSG001, Report Number: ISI-VT-20-048
- Visual Examination VT-3, Cooling Tower Area A Component Cooling Water Rigid Restraint, Report Number: ISI-VT-20-113
- Visual Examination VT-1, Cooling Tower Area A Component Cooling Water Rigid Restraint, Report Number: ISI-VT-30-114
- Visual Examination VT-3, West MSIC Area Main Steam Rigid Restraint, Report Number: ISI-VT-20-096
- Ultrasonic Examination, RCB-Loop 2 D-Ring S/G #2 Main Steam Tube sheet to Lower Shell Weld, Report Number: ISI-UT-20-005
- Ultrasonic Examination, RAB-East Wing Area Safety Injection Elbow to 8-inch Pipe Weld, Report Number: ISI-UT-20-004
- Ultrasonic Examination, Loop 1A Safety Injection 12-inch Pipe to 45 Degree Elbow Weld, Report Number: ISI-UT-20-006

- Ultrasonic Examination, Loop 1A Safety Injection 45 Degree Elbow to 12-inch Pipe Weld, Report Number: ISI-UT-20-007
- Ultrasonic Examination, Loop 1A Safety Injection 12-inch Pipe to 45 Degree Elbow Weld, Report Number: ISI-UT-20-008
- Ultrasonic Examination, RCP 1A Motor Flywheel (Entire Flywheel), Report Number: ISI-UT-20-013
- Welding: Work Order 520048-005, Weld Map No. 520048-05-01, Reference Drawing No. E3029LW3RC25, Flange to Pipe Weld SW 12 (Fillet Weld)
- Welding: Work Order 520048-97, Weld Map No. 520048-07-01, Reference Drawing No. E3029LW3RC25 -- A193-134, Support - Field Welds FW1 and FW2 03.01.b - Pressurized-Water Reactor Vessel Upper Head Penetration Examination Activities.
- Waterford 3 started Refueling Outage RF23 in October 2020. Waterford 3 installed a new reactor head with A690 welds during Refueling Outage RF 18, spring of 2013. 10CFR50.55a requires implementation of Code Case N-729-6. Inspection of replacement head is: Bare Metal Visual for Item No. B4.30 every 3rd Refuel or 5 years. The next scheduled reactor head inspection utilizing Code Case N-729-6 is during Refueling Outage RF24.

Between Refueling outage RF20 and RF21, the licensee experienced a component cooling water leak on the reactor head cooling water system, which ended up depositing molybdates on the reactor head. The reactor head was cleaned, but some residual material remained. During Refueling Outage RF22 more cleaning of the reactor head was performed. In August of 2020, the licensee experienced a leak in two incore instrumentation tubes, which sprayed boric acid over a large area, above the reactor head, in the insulation package. The licensee cleaned up most of the boric acid residue but did not perform any cleaning on the boric acid that had run across the insulation package and down the control rod drive mechanism housings and onto the reactor head. During the current Refueling Outage RF23, the licensee identified all the areas on the reactor head that had boric acid residue. The licensee took samples and ran chemical analyses on them to confirm that the radionuclides found in the samples were from the August time frame and not recent leakage of boric acid. The licensee also used the inspection crawler (utilized during required Code Case N-729-6 visual examination) to inspect all the reactor head penetrations. Upon completion of the reactor head examination, and when the licensee confirmed that there were no active leaks from any reactor head penetration, the reactor head was cleaned using dry ice particles.

#### 03.01.c – Pressurized-Water Reactor Boric Acid Corrosion Control Activities.

- Boric Acid Evaluation: 19-WF3-0003, Condition Report CR-WF3-18-6844
- Boric Acid Evaluation: 19-WF3-0004, Condition Report CR-WF3-19-0140
- Boric Acid Evaluation: 19-WF3-0012, Condition Report CR-WF3-19-1063
- Boric Acid Evaluation: 19-WF3-0017, Condition Report CR-WF3-19-2089
- Boric Acid Evaluation: 19-WF3-0021, Condition Report CR-WF3-19-3302
- Boric Acid Evaluation: 19-WF3-0023, Condition Report CR-WF3-19-4894
- Boric Acid Evaluation: 19-WF3-0025, Condition Report CR-WF3-19-6358
- Boric Acid Evaluation: 19-WF3-0026, Condition Report CR-WF3-19-6379
- Boric Acid Evaluation: 19-WF3-0028, Condition Report CR-WF3-19-5755
- Boric Acid Evaluation: 19-WF3-0034, Condition Report CR-WF3-19-6603

- Boric Acid Evaluation: 20-WF3-0001, Condition Report CR-WF3-20-1131
- Boric Acid Evaluation: 20-WF3-0004, Condition Report CR-WF3-20-3933
- Boric Acid Evaluation: 20-WF3-0013, Condition Report CR-WF3-20-4592

03.01.d – Pressurized-Water Reactor Steam Generator Tube Examination Activities

- No Steam Generator inspections were required or scheduled during Refueling Outage RF23.

71111.11Q - Licensed Operator Regualification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (1 Sample)

- (1) The inspectors observed and evaluated licensed operator performance in the control room during drain down activities to lower reactor coolant system inventory for reinstallation of the reactor vessel head on October 21, 2020.

Licensed Operator Regualification Training/Examinations (IP Section 03.02) (1 Sample)

- (1) The inspectors observed and evaluated a simulator scenario during a licensed operator examination on November 16, 2020.

71111.12 - Maintenance Effectiveness

Quality Control (IP Section 03.02) (1 Sample)

The inspectors evaluated the effectiveness of maintenance and quality control activities to ensure the following SSC remains capable of performing its intended function:

- (1) Essential chilled water system, on December 18, 2020

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

- (1) Emergent orange shutdown risk due to the failure of startup channel 2, on October 6, 2020
- (2) Emergent failure of high-pressure safety injection pump B during surveillance testing, on October 14, 2020
- (3) Emergent work on feedwater heater 2C level control valve following its failure and impact on reactor power, on November 12, 2020



## 71111.15 - Operability Determinations and Functionality Assessments

### Operability Determination or Functionality Assessment (IP Section 03.01) (3 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) Operability determination of reactor coolant loop 2 shutdown cooling upstream suction isolation valve following identification of a magnesium rotor in the motor, on November 16, 2020
- (2) Operability determination of high-pressure safety injection train A due to pump low recirculation flow, on November 17, 2020
- (3) Operability determination of the control room emergency filtration system following failure of two dampers for the B unit, on December 15, 2020

## 71111.19 - Post-Maintenance Testing

### Post-Maintenance Test Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated the following post-maintenance test activities to verify system operability and functionality:

- (1) Emergency diesel generator B following planned maintenance, on October 19, 2020
- (2) Feedwater isolation valve A following planned maintenance, on October 22, 2020
- (3) Atmospheric dump valve B following planned maintenance, on October 22, 2020
- (4) Reactor coolant loop 1 shutdown cooling upstream suction isolation valve following failure to stroke, on November 4, 2020
- (5) Chemical volume and control pump AB following planned maintenance, on November 25, 2020

## 71111.20 - Refueling and Other Outage Activities

### Refueling/Other Outage Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated refueling outage 23 activities from September 25, to November 1, 2020, which completes the partial sample documented in Waterford Inspection Report 05000382/2020003, Section 71111.20.

## 71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

### Surveillance Tests (other) (IP Section 03.01) (3 Samples)

- (1) Containment sump trisodium phosphate storage basket inspection, on November 17, 2020
- (2) Turbine trip testing, on December 14, 2020
- (3) Control room emergency filtration unit B testing, on December 16, 2020

Containment Isolation Valve Testing (IP Section 03.01) (1 Sample)

- (1) Containment atmospheric release exhaust header A inlet and upstream isolation valves, on October 19, 2020

71114.02 - Alert and Notification System Testing

Inspection Review (IP Section 02.01-02.04) (1 Sample)

- (1) The inspectors evaluated the maintenance and testing of the alert and notification system between September 1, 2018 and November 30, 2020.

71114.03 - Emergency Response Organization Staffing and Augmentation System

Inspection Review (IP Section 02.01-02.02) (1 Sample)

- (1) The inspectors evaluated the readiness of the Emergency Preparedness Organization between September 1, 2018 and November 30, 2020.

71114.04 - Emergency Action Level and Emergency Plan Changes

Inspection Review (IP Section 02.01-02.03) (1 Sample)

- (1) The inspectors evaluated the 10 CFR 50.54(q) emergency plan change process and practices between September 1, 2018 and November 30, 2020. This involved review of licensee screening and evaluation documentation. The reviews of the change process documentation do not constitute NRC approval.

71114.05 - Maintenance of Emergency Preparedness

Inspection Review (IP Section 02.01 - 02.11) (1 Sample)

- (1) The inspectors evaluated the maintenance of the emergency preparedness program between September 1, 2018 and November 30, 2020.

71114.06 - Drill Evaluation

Drill/Training Evolution Observation (IP Section 03.02) (1 Sample)

The inspectors evaluated:

- (1) Emergency preparedness training evolution in the control room simulator, on November 18, 2020

**OTHER ACTIVITIES – BASELINE**

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

EP01: Drill/Exercise Performance (IP Section 02.12) (1 Sample)

- (1) October 1, 2019, through September 30, 2020

EP02: ERO Drill Participation (IP Section 02.13) (1 Sample)

- (1) October 1, 2019, through September 30, 2020

EP03: Alert & Notification System Reliability (IP Section 02.14) (1 Sample)

- (1) October 1, 2019, through September 30, 2020

BI01: Reactor Coolant System (RCS) Specific Activity Sample (IP Section 02.10) (1 Sample)

- (1) October 1, 2019, through September 30, 2020

BI02: RCS Leak Rate Sample (IP Section 02.11) (1 Sample)

- (1) October 1, 2019, through September 30, 2020

OR01: Occupational Exposure Control Effectiveness Sample (IP Section 02.15) (1 Sample)

- (1) January 1, 2019, through September 30, 2020

PR01: Radiological Effluent Technical Specifications/Offsite Dose Calculation Manual  
Radiological Effluent Occurrences (RETS/ODCM) Radiological Effluent Occurrences Sample  
(IP Section 02.16) (1 Sample)

- (1) January 1, 2019, through September 30, 2020

71152 - Problem Identification and Resolution

Semiannual Trend Review (IP Section 02.02) (1 Sample)

- (1) The inspectors reviewed the licensee's corrective action program for potential adverse trends in the fire protection program that might be indicative of a more significant safety issue.

Annual Follow-up of Selected Issues (IP Section 02.03) (1 Sample)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issues:

- (1) High pressure safety injection pump B breaker issues, on October 27, 2020

71153 – Follow-up of Events and Notices of Enforcement Discretion

Personnel Performance (IP Section 03.03) (1 Sample)

- (1) The inspectors evaluated the licensee's response to an automatic reactor trip due to control element drive mechanism control system timer failure on November 2, 2020.

## **INSPECTION RESULTS**

No findings were identified.

## **EXIT MEETINGS AND DEBRIEFS**

The inspectors verified no proprietary information was retained or documented in this report.

- On December 11, 2020, the inspectors presented the emergency preparedness program inspection results to Mr. J. Ferrick, Site Vice President, and other members of the licensee staff.
- On January 7, 2021, the inspectors presented the inservice inspection results to Mr. M. Lewis, General Manager of Plant Operations, and other members of the licensee staff.
- On January 14, 2021, the inspectors presented the integrated inspection results to Mr. J. Ferrick, Site Vice President, and other members of the licensee staff.

## DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.04	Procedures	OP-009-002	Emergency Diesel Generator	354
		OP-009-005	Shutdown Cooling	044
71111.05	Corrective Action Documents	CR-WF3-YYYY-NNNN	2020-06250	
	Miscellaneous	RAB 13-001	Waterford S.E.S Prefire Strategy Battery Room AB	007
		RCB-001	Waterford S.E.S Prefire Strategy RCB General Area	011
		ROOF E-001	Waterford S.E.S Prefire Strategy Elevation +46 Roof E MSIV B	005
		ROOF W-001	Waterford S.E.S Prefire Strategy Elevation +46 ROOF W MSIV A	002
71111.08P	Corrective Action Documents	CR-WF3-YYYY-NNNN	2019-0617, 2019-0762, 2020-1078, 2019-4461, 2019-4223, 2019-1758, 2019-5652, 2019-8604, 2020-0180, 2020-0375, 2019-3685, 2020-2987, 2019-0931, 2019-6456, 2020-3718, 2019-0416, 2020-3327, 2019-00719, 2019-03679, 2019-03685, 2019-04683, 2019-04694, 2020-04843, 2020-04845, CR-HQN-2019, CR-HQN-2019-1125, CR-HQN-2019-2482, CR-HQN-2019-0957, CR-HQN-2019-2716, CR-HQN-2019-1060	
	Drawings	5817-10936	ICI Adapter Assembly	0
		5817-11178	Type "B" HJTC Cable Assembly (RDP to RAS)	0
		5817-11772	ICI & HJTC Quickloc Assembly, Sheet 1	2
		E-9270-164-005	Nozzle Requirements Closure Head	9
		LDRAC No. 5817-13747, Sheet 1	Closure Head Assembly	0
	Engineering Changes	EC# 0000081552	Disposition of Active Leak Found on Reactor	0
	Engineering Evaluations	Evaluation #: 19-WF3-0025	Boric Acid Walkdown, Reactor Head, ICI #5	08/02/2019
		Evaluation #: 19-WF3-0026	Boric Acid Walkdown, Reactor Head, ICI #8	08/03/2019

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Miscellaneous	17-WF3-0128	Boric Acid Leakage Identification. Component ID#: PSLMVAAA204, Location: RAB	12/28/2017
		EPID L-2018-LLR-0125	Waterford Steam Electric Station, Unit 3-Authorization of Proposed Alternative to ASME Code Case N-770-2 (EPID L-2018-LLR-0125)	06/12/2019
		EPID L-2019-LLR-0003	Waterford Steam Electric Station, Unit 3 – Authorization of Proposed Alternative to ASME Code, Section XI, IWA-4000, "Repair/Replacement Activities."	08/27/2019
		PQR-024	Procedure Qualification Record, Manual Gas tungsten & Shielded Metal Arc Welding (GTAW & SMAW), Base Metal: P-No.1, SA-106 Gr. B	1
		PQR-107	Procedure Qualification Record, Manual Gas Tungsten & Shielded Metal Arc Welding (GTAW & SMAW), Base Metal: P-No.8, SA-312 Type 304	1
		PQR-170	Manual Gas Tungsten & Shielded Metal Arc Welding (GTAW & SMAW), Base Metal: P-No.8, SA-240 Type 304	1
		PQR-330	Manual Gas Tungsten & Shielded Metal Arc Welding (GTAW & SMAW), Base Metal: P-No.1, Group 1 to 2, SA-516 Gr. 60 to SA-516 Gr. 60 to SA-516 Gr. 70	1
		PQR-331	Manual Gas Tungsten & Shielded Metal Arc Welding (GTAW & SMAW), Base Metal: P-N0.1, Group 1 to 2, SA-516 Gr. 60 to SA-516 Gr. 70	1
		W3F1-2008-0060	Request for Alternative W3-iSI-006, Proposed Alternative to Extend the Second 10-Year Inservice, Inspection Interval for Reactor Vessel Internal Weld Examinations, Waterford Steam Electric Station, Unit 3, Docket No. 50-382, License No. NPF-38	09/18/2008
		W3F1-2009-0006	RAI Response to Request for Alternative W3-ISI-006 To Extend the Second, 10 Year AMSE Code ISI and License Amendment Request, NPF-38-280 To Support Request for Alternative W3-ISI-006, Waterford Steam Electric Station, Unit 3 (Waterford 3), Docket No. 50-382, License No. NPF-38	03/19/2009
		W3F1-2013-0044	Waterford 3 Request for Alternative W3-ISI-023, ASME	09/26/2013

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Code Case N-770-1, Successive Examinations, Waterford Steam Electric Station, Unit 3, Docket No. 50-382, License No. NPF-38	
		W3F1-2013-0044	Waterford 3 Request for Alternative W3-ISI-023, ASME Code Case N-770-1, Successive Examinations, Waterford Steam Electric Station, Unit 3, Docket No. 50-382 License No. NPF-38	09/26/2013
		W3F1-2018-0008	Request for NRC Alternative to ASME IWA-5211 Regarding Charging Pipe, Visual Inspection, Relief Request W3-ISI-030, Waterford Steam Electric Station, Unit 3 (Waterford 3) Docket No. 50-382, License No. NPF-38	02/20/2018
		W3F1-2018-0067	Response to NRC Request for Additional Information Regarding Request for, Alternative to ASME Code Case N-770-2, Successive Examinations, Relief, Request W3-ISI-031	11/19/2018
		W3F1-2019-0009	Proposed Inservice Inspection Program Alternative WF3-RR-19-1 for Application of Dissimilar Metal Weld Full Structural Weld Overlay – Reactor Coolant System Cold Leg Drain Nozzles	01/28/2019
		W3F1-2019-0017	Closure of Commitment Associated with Inservice Inspection Program, Alternative WF3-RR-19-1 for Application of Dissimilar Metal Weld Full, Structural Weld Overlay – Reactor Coolant System Cold Leg Drain Nozzles	02/14/2019
		W3F1-2019-0049	Commitment Change Notification and Closure of Commitment Associated with Inservice Inspection Program Alternative WF3-RR-19-1 for Application of Dissimilar Metal Weld Full Structural Weld Overlay – Reactor Coolant System Cold Leg Drain Nozzles	07/16/2019
		W3F1-2020-0055	10 CFR Part 21 Report: Defect of Masoneilan 8012N-3C Electropneumatic Positioner. Waterford Steam Electric Station, Unit 3 (Waterford 3), NRC Docket No. 50-382 Renewed Facility Operating License No. NPF-38	09/01/2020

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		WPS-CS-1/1-C	Shielded Metal Arc Welding (SMAW) of P-No.1 carbon steels, in all positions, for all joint types, fillets and repairs using A-No.1 carbon steel filler metal, with or without Postweld Heat Treatment (PWHT).	0
		WPS-SS-8/8-B	Manual Gas Tungsten Arc Welding (GTAW) of P-No.8 stainless steels, in all positions, for all joint types, fillets and repairs using A-N0.8 stainless steel filler metal, without Postweld Heat Treatment (PWHT).	0
	NDE Reports	BOP-VE-19-002	RCS Loop 1A Lower Cold Leg, Nozzle to Safe End Circumferential Weld, Component ID: 07-009-WOL	02/06/2019
		BOP-VT-19-005	Reactor Cavity, RVCH CEDM Nozzles.	02/01/2019
		ISI-VE-19-009	RCS Loop 1A Lower Cold Leg, Nozzle to Safe-End Circumferential Weld. Component ID: 07-009	01/16/2019
		ISI-VE-19-013	RCS Loop 2A Lower Cold Leg, Nozzle to Safe-End Circumferential Weld. Component ID: 00-007	01/24/2019
		ISI-VT-19-038	Cooling Tower Area B, Rigid Restraint. Component ID: CCRR-0393	01/14/2019
		ISI-VT-19-063	Loop 2A, Rigid Restraint, Component ID: RCRR-0094	01/19/2019
		ISI-VT-19-064	Loop 2A, Rigid Restraint. Component ID: RCRR-0182	01/19/2019
		ISI-VT-20-008	RAB - 4 Wing Area, Sampling - RAB, Component ID: 13.3.3	02/12/2020
		ISI-VT-20-029	RAV-MSIV Area, N2 Accumulator VI Subsystem, Component ID: 13.3.26	05/26/2020
	Procedures	CEP-BAC-001	Boric Acid Corrosion Control (BACC) Program Plan, Entergy Nuclear Engineering Programs.	2
		CEP-ISI-100	ASME Section XI, Division 1, Fleet Administrative Controls for Inservice Inspection Program, Entergy Nuclear Engineering Programs	5
		CEP-NDE-0400	Ultrasonic Examination, Entergy Nuclear Engineering Programs	8
		CEP-NDE-0404	Manual Ultrasonic Examination of Ferritic Piping Welds (ASME XI), Entergy Nuclear Engineering Programs	9
		CEP-NDE-0423	Manual Ultrasonic Examination of Austenitic Piping Welds (ASME XI), Entergy Nuclear Engineering Programs	9



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		CEP-NDE-0493	Manual Ultrasonic Examination of Reactor Coolant Pump Flywheel, Entergy Nuclear Engineering Programs	4
		CEP-NDE-0497	Manual Ultrasonic Examination of Weids in Vessels (Non-App. VIII), Entergy Nuclear Engineering Programs	7
		CEP-NDE-0497	Manual Ultrasonic Examination of Welds in Vessels (Non-App. VIII), Entergy Nuclear Engineering Programs	7
		CEP-NDE-0730	Non-Section XI Magnetic Particle Examination (MT), Entergy Nuclear Engineering Programs	7
		CEP-NDE-0731	Magnetic Particle Examination (MT) for ASME Section XI, Entergy Nuclear Engineering Programs	6
		CEP-NDE-0901	VT-1 Examination, Entergy Nuclear Engineering Programs	6
		CEP-NDE-0903	VT-3 Examination, Entergy Nuclear Engineering Programs	6
		EIP-PT-001	ASME Section XI Pressure Test (PT) Program, Entergy Nuclear Engineering Programs.	310
		EN-DC-319	Boric Acid Corrosion Control Program (BACCP)	12
		EN-DC-328	Entergy Nuclear Welding Program	6
		RF-001-013	Incore Instrument Flanges	315
		SEP-BAC-WF3-001	Waterford 3 Boric Acid Corrosion Control Program (BACCP) Program Section SEP-BAC-WF3-001	2
		SEP-CISI-104	Program Section for ASME Section XI, Division 1, WF3 Containment Inservice Inspection Program, Entergy Nuclear Engineering Programs	4
		SEP-ISI-104	Program Section for ASME Section XI, Division 1, WF3 Inservice Inspection program, Entergy Nuclear Engineering Programs	9
		SEP-ISI-104	Program Section for ASME Section XI Division 1 WF3 Inservice Inspection Program, Entergy Nuclear Engineering Programs	9
	Self-Assessments	LO-WLO-2019-00031	Pre-NRC RF23 ISI Activities Self-Assessment Report	06/29/2020
	Work Orders	Work Orders	527322-01	
71111.12	Corrective Action	CR-WF3-YYYY-	2019-08796, 2019-08801, 2019-08806, 2019-08826,	

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	Documents	NNNN	2019-08839, 2020-03991, 2020-05725, 2020-06388	
71111.13	Corrective Action Documents	CR-WF3-YYYY-NNNN	2020-05855	
	Procedures	EN-WM-104	On-Line Risk Assessment	022
		OI-037-000	Operations Risk Assessment Guideline	315
71111.15	Miscellaneous	W3-DBD-038	Safety-Related HVAC - Control Room Design Basis Document	301
71111.19	Corrective Action Documents	CR-WF3-YYYY-NNNN	2020-05885	
71111.22	Corrective Action Documents	CR-WF3-YYYY-NNNN	2020-07126	12/17/2020
	Miscellaneous		Process Applicability Determination Form For OP-903-115, Revision 052	09/22/2020
		FSAR Section 8.3.1	AC Power Systems	
		Technical Specification 3/4.8.1	A.C. Sources	
	Procedures	ME-007-002	Molded Case Circuit Breakers	28
		OP-903-027	Inspection of Containment	306
		OP-903-115	Train A Integrated Emergency Diesel Generator/Engineering Safety Features Test	
		OP-903-123	Control Room Envelope Pressure Test	311
		STA-001-004	Surveillance Procedure Local Rate Test	318
	Work Orders	157704	Perform Maintenance on Breaker Referencing ME-007-002	09/03/2013
71114.02	Corrective Action Documents	CR-WF3-YYYY-NNNN	2018-05551, 2018-05591, 2018-07075, 2019-04362, 2019-06326, 2019-06327, 2019-08824, 2019-08930, 2020-03581, 2020-04021, 2020-04022, 2020-04023, 2020-04507, 2020-06251	
	Miscellaneous		Updated Alert/Notification System Design Report (December 2014)	8 (Addendum 0, Revision 1)
		KLD TR-825	Waterford 3 Steam Electric Station: 2015 - Population by Siren Contour Analysis Update	02/19/2016

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	Procedures	EPP-422	Siren and Helicopter Warning System Maintenance	013
		EPP-424	Siren Testing and Siren System Administrative Controls	020
71114.03	Corrective Action Documents	CR-WF3-YYYY-NNNN	2018-05402, 2018-07228, 2019-04576, 2019-06270, 2019-07314, 2019-07397, 2019-07817, 2020-01181, 2020-02280, 2020-02886	
	Miscellaneous		ERO Callout Test Analysis Results (Multiple Quarterly and Monthly Reports)	3Q/2018 - 3Q/2020
			Waterford 3 Station On-Shift Staffing Analysis Final Report (April 4, 2017)	1
	Procedures	EN-EP-310	Emergency Response Organization Notification System	009
		EP-002-015	Emergency Responder Activation	304
		EPP-462	Evaluation of ERO Callout Test	5
71114.04	Corrective Action Documents	CR-WF3-YYYY-NNNN	2019-08175, 2019-08488, 2020-00857	
	Miscellaneous		10 CFR 50.54(Q)(3) Screening & Evaluation: EP-002-010, Rev 316	04/24/2019
			10 CFR 50.54(Q)(3) Screening & Evaluation: EN-EP-313, Rev. 003	10/25/2019
			10 CFR 50.54(Q)(2) Review: Plant Entrance and Exit Points (for COVID-19 Screening)	03/24/2020
			10 CFR 50.54(Q)(2) Review: EN-EP-609, EOF Operations, Rev 6	02/17/2020
			10 CFR 50.54(Q)(2) Review: EP-001-010, Unusual Event, Rev 307	02/04/2019
			10 CFR 50.54(Q)(2) Review: EP-001-020, Alert, Rev 310	02/04/2019
			10 CFR 50.54(Q)(2) Review: EP-001-030, Site Area Emergency, Rev 309	02/08/2019
			10 CFR 50.54(Q)(2) Review: EP-001-040, General Emergency, Rev 310	02/08/2019
			10 CFR 50.54(Q)(2) Review: EP-002-052, Protective Action Guidelines, Rev 026	04/24/2019
	Procedures	EN-EP-305	Emergency Planning 10CFR50.54(q) Review Program	7 & 8
71114.05	Corrective Action Documents	CR-WF3-YYYY-NNNN	2018-06295, 2018-07047, 2019-00361, 2019-00399, 2019-00401, 2019-03651, 2019-04353, 2019-06431,	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			2019-06444, 2019-07149, 2019-07467, 2019-07477, 2019-07547, 2019-07722, 2019-08611, 2019-08638, 2019-08640, 2019-08711, 2020-00228, 2020-01114, 2020-01180, 2020-01319, 2020-01325, 2020-01992, 2020-06193	
	Corrective Action Documents Resulting from Inspection		CR-WF3-2020-07046, WTWF3-2020-00381	
	Miscellaneous		ERO Red Team Drill (3/4/2020) Report	04/03/2020
			ERP Blue Team Drill (4/3/2019) Report	04/15/2019
			ERO Red Team Site Drill (4/30/2019) Report	05/30/2019
			TSC Limited Scope Focused Drill 2019-07 Report	05/13/2019
			ERO Orange Team NRC Dress Rehearsal Drill (9/25/2019) Report	10/23/2019
			TSC Operations Coordinator Focused Drill 2019-05	03/11/2019
			ERO Blue Team Site Drill (12/6/2018) Report	01/20/2019
			HP Drill Report (12/09/2019)	12/17/2019
			HP Drill Report (06/20/2019 - #2019-08)	10/01/2019
			Onsite Medical Drill Report (10/30/2019)	12/03/2019
			ERO Orange Team NRC Exercise (11/20/2019) Report	12/17/2019
			Waterford Emergency Plan - Appendix C (Letters of Agreement with Hahnville Volunteer Fire Department, U.S. Coast Guard, St. Charles Parish Sheriffs Office, & Panther Helicopter)	Various(Updated 2019-2020)
			Waterford Emergency Plan - Appendix I (Emergency Medical Assistance Program, Letters of Agreement with St. Charles Parish Hospital and West Jefferson Medical Center)	Various (Updated 2019 - 2020)
			Waterford 3 Steam Electric Station Emergency Plan	50, 51
	Emergency Preparedness Focused Tabletop #2017-		Focused Drill for Site Accountability	12/28/2018 & 11/27/2019

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		17		
		KLD TR-1017	Waterford-3 Steam Electric Station 2018 Population Update Analysis	09/08/2018
		KLD TR-1092	Waterford-3 Steam Electric Station 2019 Population Update Analysis	09/07/2019
		KLD TR-1172	Waterford-3 Steam Electric Station 2020 Population Update Analysis	09/05/2020
		KLD TR-502	Waterford-3 Steam Electric Station Development of Evacuation Time Estimates	11/2012
		QA-7-2019-W3-1	Audit Report: Emergency Preparedness (10 CFR 50.54(t) Program Review)	05/13/2019
		QA-7-2020-W3-1	Audit Report: Emergency Preparedness (10 CFR 50.54(t) Program Review)	05/18/2020
		Trimester 19-02	Nuclear Independent Oversight Functional Area Performance Report	10/20/2019
		Trimester 20-01	Nuclear Independent Oversight Functional Area Performance Report	03/17/2020
		Trimester 20-02	Nuclear Independent Oversight Functional Area Performance Report	06/29/2020
		Trimester 20-03	Nuclear Independent Oversight Functional Area Performance Report	10/23/2020
		W3D3-2019-0001	Summary Drill Report: Control Room Emergency Breathing Air Drill (Scenario #2019-06)	05/08/2019
		W3D3-2019-0002	Summary Drill Report: Control Room Emergency Breathing Air Drill (Scenario #2019-06)	12/06/2019
		W3D3-2020-01	Summary Drill Report: Control Room Emergency Breathing Air Drill (Scenario #2020-08)	04/30/2020
		W3D3-2020-02	Summary Drill Report Control Room Emergency Breathing Air Drill (Scenario #2020-10)	08/24/2020
	Procedures	EN-LI-102	Corrective Action Program	41
		EP-001-001	Recognition and Classification of Emergency Conditions	33
		EP-002-052	Protective Action Guidelines	026
		EP-002-071	Site Protective Measures	304
		EP-003-040	Emergency Equipment Inventory	310

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		EPP-428	Equipment Important to Emergency Response	307
	Self-Assessments	LO-WLO-2019-00024	Pre-NRC Evaluated Exercise Assessment	09/09/2019
		LO-WLO-2020-00041	2020 Pre-NRC Emergency Planning Program Inspection Assessment	08/24/2020
	Work Orders	WO-WF3-NNNN	00333437, 00489973, 00495967, 00511576, 0052930321, 0052930321, 0052933267	
71151	Corrective Action Documents Resulting from Inspection	CR-WF3-YYYY-NNNN	2020-07050	
	Miscellaneous		WF3 Key ERO Member Participation Reports & ERO Rosters	4Q/2019-3Q/2020
			WF3 Drill and Exercise Performance Indicator Records (from License Operator Exam/Requalification, Team Drills, and other ERO Activities)	4Q/2019 - 3Q/2020
			WF3 ANS (Fixed Siren) Testing Records	4Q/2019 - 3Q/2020
	Procedures	EN-EP-801	Emergency Response Organization	17
		EN-FAP-EP-005	Emergency Preparedness Performance Indicators	14
		EN-LI-117	Regulatory Performance Indicator Process	17
71152	Corrective Action Documents Resulting from Inspection	CR-WF3-YYYY-NNNN	2020-03400	
	Procedures	MM-007-010	Fire Extinguisher Inspection and Extinguisher Replacement	325