

2018 NMP-1 NRC Written Exam Q5

Source/Parent Question

ES-401	Written Examination Question Worksheet	Form ES-401-5
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Examination Outline Cross-Reference:	Level	RO
Tier #	2	
Group #	1	
K/A #	223002 K6.04	
Importance Rating	3.3	

PCIS/Nuclear Steam Supply Shutoff

Knowledge of the effect that a loss or malfunction of the following will have on the PRIMARY CONTAINMENT ISOLATION SYSTEM/NUCLEAR STEAM SUPPLY SHUT-OFF: Nuclear boiler instrumentation

Proposed Question: #25

The plant is operating at 100% power with the following:

- Lo-Lo Rosemount Reactor Water Level Transmitters, LT-36-04B and LT-36-04D, fail high.
- Then, a loss of injection causes a Reactor scram.
- The Mode Switch is placed in SHUTDOWN.
- Reactor water level lowers to 0 inches.

Which one of the following describes the effect, if any, of these instrument failures on the required automatic isolations?

- A. All required automatic isolations will still occur.
- B. A required Vessel isolation will NOT occur, only.
- C. A required Containment isolation will NOT occur, only.
- D. Both a required Vessel and Containment isolation will NOT occur.

2018 NMP-1 NRC Written Exam Q7

Source/Parent Question

ES-401 Written Examination Question Worksheet Form ES-401-5

Examination Outline Cross-Reference: Level RO
 Tier # 2
 Group # 1
 K/A # 215004 K3.02
 Importance Rating 3.4

Source Range Monitor

Knowledge of the effect that a loss or malfunction of the SOURCE RANGE MONITOR (SRM) SYSTEM will have on the following: Reactor manual control: Plant-Specific

Proposed Question: #6

A Reactor startup is in progress with the following:

- All SRMs are fully inserted and indicate between 200 and 300 cps.
- All IRMs indicate downscale on Range 2.
- REFUEL INST TRIP BYPASS CH 11 and 12 keylock switches are in COINCIDENT.
- Then, the following malfunctions occur:
 - SRM 11 fails downscale.
 - SRM 14 fails upscale.

Which one of the following describes the impact of these malfunctions, if any, on the Reactor Protection System (RPS) and/or Reactor Manual Control System (RMCS)?

	Impact of SRM 11 Failing Downscale	Impact of SRM 14 Failing Upscale
A.	None	Rod block, but NO scram
B.	None	Rod block and a scram
C.	Rod block, but NO half scram	Rod block, but NO scram
D.	Rod block, but NO half scram	Rod block and a scram

2018 NMP-1 NRC Written Exam Q12

Source/Parent Question

SGTS

Knowledge of the effect that a loss or malfunction of the STANDBY GAS TREATMENT SYSTEM will have on following: Off-site release rate

Proposed Question: #44

The plant is operating at 100% power with the following:

- RBEVS fan 11 is tagged out for maintenance.
- An un-isolable steam leak develops from Reactor Water Cleanup into the Reactor Building.
- Reactor Building Ventilation exhaust radiation monitors are reading 30 mR/hr and slowly rising.
- Reactor Building differential pressure is -0.05 psig and stable.

Then, RBEVS fan 12 trips and CANNOT be restarted.

Which one of the following describes the consequences of the RBEVS fan 12 trip on Reactor Building differential pressure (D/P) and off-site release rate?

	<u>Reactor Building D/P</u>	<u>Off-site Release Rate</u>
A.	Maintained negative by normal Reactor Building Ventilation	Remains unchanged
B.	Maintained negative by normal Reactor Building Ventilation	Rises
C.	Becomes positive	Remains unchanged
D.	Becomes positive	Rises

2018 NMP-1 NRC Written Exam Q16

Source/Parent Question

ADS

Knowledge of the effect that a loss or malfunction of the AUTOMATIC DEPRESSURIZATION SYSTEM will have on following: Ability to rapidly depressurize the reactor

Proposed Question: #6

The plant was operating at 100% power when the following events occurred:

- An un-isolable steam leak in the Reactor Building has led to the need for an RPV Blowdown.
- Battery Board 11 is de-energized due to a sustained electrical fault.

Which one of the following describes the number of ERVs available to rapidly depressurize the Reactor, in accordance with N1-EOP-8, RPV Blowdown?

- A. Exactly the Minimum Number of ERVs Required for Emergency Depressurization is available, only.
- B. The Minimum Number of ERVs Required for Emergency Depressurization is available, plus one additional ERV.
- C. One less ERV than the Minimum Number of ERVs Required for Emergency Depressurization is available, only.
- D. Two less ERVs than the Minimum Number of ERVs Required for Emergency Depressurization are available.

2018 NMP-1 NRC Written Exam Q19

Source/Parent Question

Ability to manually operate and/or monitor in the control room: Adjustment of exciter voltage			
Proposed Question:	RO Question # 20		
The plant is operating at 100% power with EDG 102 running loaded.			
Which one of the following describes the effect of going to RAISE on the VOLTAGE ADJ RHEO GEN 102 control switch in the following conditions?			
	<table border="1"> <tr> <td>EDG 102 is operating with PB 102 disconnected from the grid</td> <td>EDG 102 is operating in parallel with the grid</td> </tr> </table>	EDG 102 is operating with PB 102 disconnected from the grid	EDG 102 is operating in parallel with the grid
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A.	<table border="1"> <tr> <td>PB 102 voltage rises KVARs remain the same</td> <td>PB 102 voltage rises KVARs remain the same</td> </tr> </table>	PB 102 voltage rises KVARs remain the same	PB 102 voltage rises KVARs remain the same
PB 102 voltage rises KVARs remain the same	PB 102 voltage rises KVARs remain the same		
B.	<table border="1"> <tr> <td>PB 102 voltage rises KVARs remain the same</td> <td>PB 102 voltage remains the same KVARs rise</td> </tr> </table>	PB 102 voltage rises KVARs remain the same	PB 102 voltage remains the same KVARs rise
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C.	<table border="1"> <tr> <td>PB 102 voltage remains the same KVARs rise</td> <td>PB 102 voltage rises KVARs remain the same</td> </tr> </table>	PB 102 voltage remains the same KVARs rise	PB 102 voltage rises KVARs remain the same
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D.	<table border="1"> <tr> <td>PB 102 voltage remains the same KVARs rise</td> <td>PB 102 voltage remains the same KVARs rise</td> </tr> </table>	PB 102 voltage remains the same KVARs rise	PB 102 voltage remains the same KVARs rise
PB 102 voltage remains the same KVARs rise	PB 102 voltage remains the same KVARs rise		
Proposed Answer: B			
Explanation (Optional):			
A.	Incorrect - When in parallel with the grid, raising EDG 102 excitation will cause the EDG to assume a higher MVAR loading, but will not change grid voltage.		

2018 NMP-1 NRC Written Exam Q21

Source/Parent Question

QUESTION 52

The plant is operating at 100% power with the following:

- Instrument Air Compressors (IAC) 11 and 13 are in service
- IAC 12 control switch is tagged in pull-to-lock for maintenance
- Instrument Air Dryer (IAD) 94-168 is in service
- Subsequently, IAC 11 trips and its control switch is placed in pull-to-lock
- The CRS directs bypass of IAD 94-168 and IAD 94-169

Which one of the following actions is required until either IAC 11 or IAC 12 is returned to service, in accordance with N1-OP-20?

- A. Blow down designated air manifolds once every 24 hours.
- B. Align the temporary service air compressor to the service air system.
- C. Align service air to the reactor building track bay roll door (D-39) inflatable seal.
- D. Align breathing air to the Service Air System after removing the valve internals from check valve 94-51.

2018 NMP-1 NRC Written Exam Q25

Source/Parent Question

SGTS

Ability to (a) predict the impacts of the following on the **STANDBY GAS TREATMENT SYSTEM**; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: High secondary containment ventilation exhaust radiation

Proposed Question: #87

The plant is operating at 100% power with the following:

- Emergency Diesel Generator (EDG) 102 is out of service for corrective maintenance.
- A small steam leak develops in the Reactor Building.
- Reactor Building Ventilation isolates and Reactor Building Emergency Ventilation (RBEVS) automatically starts.
- Reactor Building Ventilation exhaust radiation stabilizes at 10 mR/hr.
- It is desired to secure one train of RBEVS.

Which one of the following identifies the train of RBEVS that should be secured and the need for Technical Specification LCO entry when the train is secured, in accordance with N1-OP-10, Reactor Building Heating, Cooling, and Ventilating System?

Secure RBEVS train...

- A. 11. Technical Specification LCO entry is required.
- B. 11. Technical Specification LCO entry is NOT required.
- C. 12. Technical Specification LCO entry is required.
- D. 12. Technical Specification LCO entry is NOT required.

2018 NMP-1 NRC Written Exam Q27

Source/Parent Question

Partial or Complete Loss of Forced Core Flow Circulation

Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION: Nuclear boiler instrumentation system

Proposed Question: #48

The plant is operating at power when the following occur:

- Powerboard 11 de-energizes due to an electrical fault.
- Reactor Recirculation pump (RRP) flows stabilize at the following values:
 - RRP 11: 4 Mlbm/hr
 - RRP 12: 4 Mlbm/hr
 - RRP 13: 11 Mlbm/hr
 - RRP 14: 11 Mlbm/hr
 - RRP 15: 11 Mlbm/hr
- Reactor power stabilizes at 52%.
- Main Condenser vacuum is 27 inches Hg and stable.

Note: A portion of the three loop power-to-flow map is provided on the following page.

Which one of the following describes the current rod line and if continuous operation is permitted in the current region of the power-to-flow map, in accordance with N1-SOP-1.5, Unplanned Power Change?

The plant is operating...

- A. below the 100% rod line. Action is required to exit the current region.
- B. below the 100% rod line. Continuous operation is allowed in the current region.
- C. above the 100% rod line. Action is required to exit the current region.
- D. above the 100% rod line. Continuous operation is allowed in the current region.

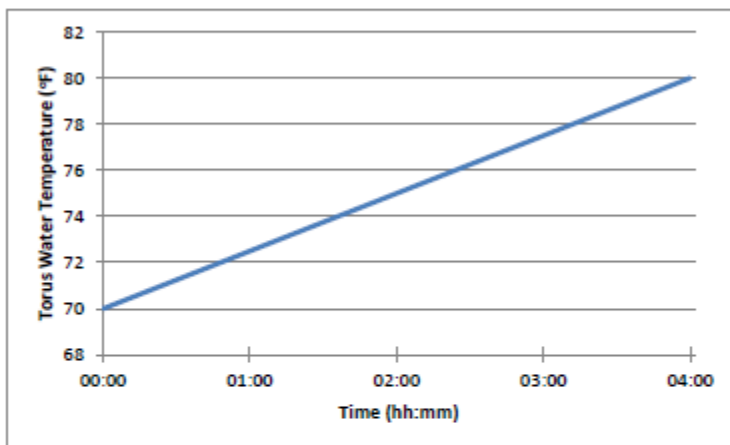
2018 NMP-1 NRC Written Exam Q28

Source/Parent Question

Proposed Question: #49

The plant is operating at 100% power with the following:

- An ERV has been leaking for the last four (4) hours.
- Reactor pressure is 1020 psig and stable.
- Torus water level is 10.7 feet and approximately stable.
- Torus water temperature has risen as follows:



Note: Assume the trend remains constant and Torus Cooling is NOT placed in service. The Heat Capacity Temperature Limit is provided on the following page.

Which one of the following identifies the approximate time N1-EOP-4, Primary Containment Control, entry will first be required and the approximate time a manual Reactor scram will first be required due to Torus water temperature, in accordance with N1-EOP-4?

	Approximate Time N1-EOP-4 Entry First Required	Approximate Time Manual Reactor Scram First Required
A.	06:00	16:00
B.	06:00	22:00
C.	10:00	16:00
D.	10:00	22:00

2018 NMP-1 NRC Written Exam Q30

Source/Parent Question

Partial or Complete Loss of AC

Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER: Systems necessary to assure safe plant shutdown

Proposed Question: #48

A Station Blackout is in progress.

Which one of the following describes steps that are taken to limit battery discharge, in accordance with EO-100-030, Unit 1 Response to Station Blackout?

Shedding of non-class 1E 250 VDC loads is required to be completed within (1) minutes from the start of the Station Blackout. HPCI and RCIC are (2).

- A. (1) 15
(2) run continuously unless erratic operation is observed.
- B. (1) 15
(2) secured as soon as NOT needed for Reactor water level control.
- C. (1) 45
(2) run continuously unless erratic operation is observed.
- D. (1) 45
(2) secured as soon as NOT needed for Reactor water level control.

2018 NMP-1 NRC Written Exam Q38

Source/Parent Question

94

ID: N1-226001-RBO-05-Q-02

Points: 1.00

The plant is operating at 90% power, with the following:

- Containment Spray Loop 111 is in Torus Cooling
- All other Containment Spray components are in normal, standby condition

THEN a loss of coolant accident results in the following conditions:

- Drywell pressure is 3.5 psig and slowly rising
- RPV water level is 0 inches and slowly lowering

Which one of the following identifies the condition of Containment Spray Loop 111, two minutes after the scram?

- A. ☐ Not spraying the drywell and tripped
- B. ☐ Not spraying the drywell and in Torus Cooling
- C. ☐ Spraying the drywell with FCV 80-118, Containment Spray Test to Torus, OPEN
- D. ☐ Spraying the drywell with FCV 80-118, Containment Spray Test to Torus, CLOSED

Answer: ☐ B

2018 NMP-1 NRC Written Exam Q39

Source/Parent Question

Importance Rating 3.8

Proposed Question: # 21

Unit 1 is operating at 100% power with the following:

- Instrument Bus 1Y226 de-energizes due to a sustained electrical fault.
- ON-YPNL-101, Loss of Instrument Bus, is being performed.
- Then, a spurious Reactor scram occurs.
- The Unit Supervisor enters EO-000-102, RPV Control.

Which one of the following describes the correct procedure implementation for ON-YPNL-001?

- A. Continue performing ON-YPNL-101. In the event of a conflict between ON-YPNL-101 and EO-000-102, EO-000-102 is the overriding procedure.
- B. Continue performing ON-YPNL-101. In the event of a conflict between ON-YPNL-101 and EO-000-102, ON-YPNL-001 is the overriding procedure.
- C. Exit ON-YPNL-101. ON-YPNL-101 is re-entered at the step in progress after exiting EO-000-102.
- D. Exit ON-YPNL-101. ON-YPNL-101 entry conditions are re-evaluated after exiting EO-000-102.

2018 NMP-1 NRC Written Exam Q48

Source/Parent Question

Knowledge of the effect that a loss or malfunction of the REACTOR WATER CLEANUP SYSTEM will have on following: Reactor water level

Proposed Question: RO Question # 29

The plant has commenced a startup, with the following conditions:

- Reactor water level is 71" and stable
- Feedwater low flow control valve (FCV) 11 is in MAN
- RWCU System is in service, with the auxiliary pump running
- RWCU System low pressure PCV is operating in AUTO
- Reject flow rate is 7×10^4 lbm/hr
- RWCU System sensed pressure fails upscale

Which one of the following describes the impact on RWCU system flow rate due to the failure and the required operator action to maintain RPV water level constant?

	<u>RWCU Flow Rate</u>	<u>Required action</u>
A.	Lowers	Lower flow on Feedwater low FCV 11
B.	Lowers	Raise flow on Feedwater low FCV 11
C.	Rises	Lower flow on Feedwater low FCV 11
D.	Rises	Raise flow on Feedwater low FCV 11

Proposed Answer: A

2018 NMP-1 NRC Written Exam Q52

Source/Parent Question

Loss of CRD Pumps

Knowledge of the reasons for the following responses as they apply to LOSS OF CRD PUMPS: Reactor SCRAM

Proposed Question: #61

The plant is at 100% power. Conditions are as follows:

- The running CRD pump has tripped.
- The standby CRD pump has failed to start.
- Two accumulator trouble alarms have been received for withdrawn control rods.
- Both control rods have accumulator pressures of 920 psig.
- The CRD pumps cannot be started.

Per N2-OP-30, Control Rod Drive Failures, which one of the following correctly completes the below statement?

A Reactor scram is required ____ (1) _____. This scram is required due to degraded ____ (2) _____.

- A. (1) immediately
(2) control rod scram capability
- B. (1) immediately
(2) cooling to the CRD mechanisms
- C. (1) 20 minutes after the loss of CRD Charging Header pressure
(2) control rod scram capability
- D. (1) 20 minutes after the loss of CRD Charging Header pressure
(2) cooling to the CRD mechanisms

2018 NMP-1 NRC Written Exam Q55

Source/Parent Question

Ability to monitor automatic operations of the RECIRCULATION FLOW CONTROL SYSTEM including: Lights and alarms

Proposed Question: RO Question # 35

The plant is operating at 85% power, with the following conditions:

- F2-2-1, REACT RECIRC M-G SET 11 annunciator alarms
- RR MG 11 SCOOP TUBE AIR FAILURE LOCK red light is ON

Which one of the following describes a possible cause of the red light and the actions required to adjust recirculation pump 11 flow?

- A. Loss of instrument air; adjust speed using the speed controller at F Panel.
- B. Loss of instrument air; station a licensed operator at the Bailey positioner to adjust speed.
- C. Loss of scoop tube oil; adjust speed using the speed controller at F Panel.
- D. Loss of scoop tube oil; station a licensed operator at the Bailey positioner to adjust speed.

Proposed Answer: B

2018 NMP-1 NRC Written Exam Q61

Source/Parent Question

PCIS/Nuclear Steam Supply Shutoff

Knowledge of the physical connections and/or cause-effect relationships between
PRIMARY CONTAINMENT ISOLATION SYSTEM/NUCLEAR STEAM SUPPLY SHUT-OFF
and the following: Containment drainage system

Proposed Question: #41

Which one of the following conditions will cause the Drywell Floor Drain Isolation Valves to close?

- A. Reactor water level at 0"
- B. Drywell pressure at 2.7 psig
- C. Drywell temperature at 160°F
- D. Drywell floor drain leakage at 20 gpm

2018 NMP-1 NRC Written Exam Q64

Source/Parent Question

SGTS

**Ability to monitor automatic operations of the STANDBY GAS TREATMENT SYSTEM including:
Fan start**

Proposed Question: #17

The plant is operating at 100% power with the following:

Time (hh:mm)	Condition
00:00	<ul style="list-style-type: none">Annunciator L1-4-3, REACT BLDG VENT RAD MONITOR OFF NORMAL, alarms.Reactor Building Exhaust radiation monitor 12 indicates downscale.
00:05	<ul style="list-style-type: none">Reactor Building Exhaust radiation monitor 11 fails upscale.

Which one of the following describes the response of Reactor Building Emergency Ventilation (RBEVS) to these conditions?

- A. Both trains of RBEVS remain in standby.
- B. Both trains of RBEVS automatically start at time 00:00.
- C. Both trains of RBEVS automatically start at time 00:05.
- D. RBEVS train 11 automatically starts at time 00:05. RBEVS train 12 remains in standby.

2018 NMP-1 NRC Written Exam Q72

Source/Parent Question

Proposed Question: #71

A transient is in progress with the following:

- Operators are required to enter a radiologically posted area in order to manually close Primary Containment Isolation Valves.
- Radiation Protection (RP) has determined that the highest dose rate in the area is 750 mRem/hr.

Which one of the following describes the radiological posting requirement for the area and the need for the Operators to be continuously escorted by RP personnel, in accordance with GAP-RPP-08, Control of High, Locked High, and Very High Radiation Areas?

This area is required to be posted as a ____ (1) _____. The Operators ____ (2) _____ required to be continuously escorted by RP personnel while in this area.

	(1)	(2)
A.	Locked High Radiation Area	are
B.	Locked High Radiation Area	are NOT
C.	High Radiation Area	are
D.	High Radiation Area	are NOT

2018 NMP-1 NRC Written Exam Q75

Source/Parent Question

Proposed Question: #100

A loss of coolant accident has resulted in the following:

- An RPV Blowdown has been performed due to exceeding the Pressure Suppression Pressure.
- Reactor pressure is 75 psig and stable.
- Reactor water level is being controlled 53 to 95 inches using Condensate.
- Core Spray injection has been secured by closing the inside isolation valves.
- Drywell pressure is 45 psig and slowly rising.
- Torus pressure is 42 psig and slowly rising.
- Torus water level is 17 feet and slowly rising.
- Chemistry reports that venting of the Primary Containment will exceed release rate limits.

Note: The Primary Containment Pressure Limit (PCPL) curve is provided on the next page.

Which one of the following describes the required control of Condensate injection and Primary Containment venting, in accordance with the Emergency Operating Procedures?

- A. Continue injection with Condensate. Direct Primary Containment venting.
- B. Continue injection with Condensate. Do NOT direct Primary Containment venting.
- C. Transition injection from Condensate to another system. Direct Primary Containment venting.
- D. Transition injection from Condensate to another system. Do NOT direct Primary Containment venting.

2018 NMP-1 NRC Written Exam Q82

Source/Parent Question

Proposed Question: #82

The plant is operating at 100% power with the following sequence of events:

<u>Time (hh:mm)</u>	<u>Event</u>
00:00	The following Main Fire Panel annunciators alarm: <ul style="list-style-type: none">• 2-1-1-1, TURB. BLDG. 261 LOCAL PNL NO. 1 FIRE• 2-2-1-2, DIESEL FIRE PUMP #1 RUNNING• 2-2-2-2, ELECTRIC FIRE PUMP #1 STARTED. <p>Fire Zone WD-8131, T1 XFMR Water Deluge System, is in alarm and indicates discharge is in progress.</p>
00:03	The Fire Brigade leader reports that there are visible flames coming from Transformer 1.

Given the following sections of EPIP-EPP-01-EAL, Emergency Action Level Matrix / Unit 1 (see next page).

Which one of the following describes the earliest time at which the fire can be considered "confirmed" and the time at which EAL 8.2.1 is met, assuming the fire is still in progress, in accordance with EPIP-EPP-28, Firefighting, and EPIP-EPP-01, Classification of Emergency Conditions at Unit 1?

	<u>Earliest Time Fire Can Be Considered "Confirmed"</u>	<u>Time EAL 8.2.1 Is Met</u>
A.	00:00	00:00
B.	00:00	00:15
C.	00:03	00:03
D.	00:03	00:18

2018 NMP-1 NRC Written Exam Q85

Source/Parent Question

QUESTION 83

The plant is operating at 80% power when a seismic event occurs. A Torus water leak results in the following:

- Annunciator H2-2-1, R BLDG FL DR SUMPS 11-16 AREA WTR LVL LEVEL HIGH, is in alarm
- Computer point F188 NE RB CORNER RM WTR LVL HIGH is in alarm
- An operator reports water level in the Northeast Corner Room is 5 feet and slowly rising

Which one of the following describes the operability of the safety-related pumps in this area?

- A. Core Spray Pumps 121 and 122 are inoperable at this time.
- B. Containment Spray Pumps 112 and 122 are inoperable at this time.
- C. Core Spray Pumps 121 and 122 remain operable until level in this area rises an additional 2 feet.
- D. Containment Spray Pumps 112 and 122 remain operable until level in this area rises an additional 2 feet.

2018 NMP-1 NRC Written Exam Q86

Source/Parent Question

Proposed Question: #77

The plant is operating at 100% power with the following:

- A spurious signal causes all Reactor Recirculation pumps to trip.
- The Reactor is manually scrammed.
- NO Reactor Recirculation pump is able to be re-started.

Five hours later, the following conditions exist:

- Reactor pressure is 90 psig and slowly lowering.
- Reactor water level is being controlled in a band of 53" to 95".
- Shutdown Cooling is being placed in service in accordance with N1-OP-43C, Plant Shutdown, and N1-OP-4, Shutdown Cooling.

Which one of the following describes (1) the required control of Reactor water level and (2) the required control of the Recirculation loops, in accordance with N1-OP-43C and N1-OP-4?

	(1) Reactor water level must be controlled...	(2) Required Control of Recirculation Loops
A.	in the current band.	The suction and discharge valves of at least two Recirculation loops must remain full open.
B.	in the current band.	All Recirculation pump suction valves, or all discharge and discharge bypass valves, must be closed.
C.	higher than the current band.	The suction and discharge valves of at least two Recirculation loops must remain full open.
D.	higher than the current band.	All Recirculation pump suction valves, or all discharge and discharge bypass valves, must be closed.

2018 NMP-1 NRC Written Exam Q88

Source/Parent Question

Proposed Question: #55

A plant startup is in progress with the following:

- The Reactor Mode Switch is in STARTUP.
- Reactor power is mid-scale on IRM range 8.
- Reactor pressure is 750 psig and rising slowly.
- The following annunciators are in alarm:
 - F3-3-6, MSIV CLOSURE SCRAM BY-PASS
 - F3-4-6, FIRST STAGE BOWL PRESSURE LOW

Which one of the following describes the status of these annunciators?

- A. Both annunciators are consistent with plant conditions.
- B. F3-3-6 is consistent with plant conditions, but F3-4-6 should NOT be in alarm.
- C. F3-4-6 is consistent with plant conditions, but F3-3-6 should NOT be in alarm.
- D. NEITHER annunciator should be in alarm.

2018 NMP-1 NRC Written Exam Q95

Source/Parent Question

Proposed Question: #94

A refueling outage is in progress when operators determine that SECONDARY CONTAINMENT is inoperable.

Which one of the following identifies an activity that may continue while SECONDARY CONTAINMENT is inoperable, in accordance with N1-OP-34, Refueling Operations?

- A. Movement of a new fuel bundle
- B. Movement of a recently irradiated fuel bundle
- C. Movement of an irradiated fuel cask in the Reactor Building
- D. An Operation with the Potential to Drain the Reactor Vessel (OPDRV)

2018 NMP-1 NRC Written Exam Q100

Source/Parent Question

Proposed Question: #99

The plant is shut down for a refueling outage with the following:

- The Reactor has been shut down for 12 hours.
- Preparations are underway for Reactor head de-tensioning.
- Reactor coolant temperature is 190°F and slowly lowering.
- Shutdown Cooling loops 11 and 13 are in service.
- Shutdown Cooling loop 12 is unavailable due to an unexpected pump trip.
- Spent Fuel Pool Cooling loop 11 is in service.

Note: A portion of OU-NM-103-101, Shutdown Safety Management Program, Attachment 1, Unit 1 Key Safety Functions, is provided on the next page.

Which one of the following describes the current shutdown safety color code for Reactor decay heat removal, in accordance with OU-NM-103-101 and ER-AA-600-1023, Paragon Model Capability?

- A. Green
- B. Yellow
- C. Orange
- D. Red