

PUBLIC NOTICE

THE U.S. NUCLEAR REGULATORY COMMISSION PROPOSES TO AMEND THE FACILITY OPERATING LICENSES FOR THE DIABLO CANYON NUCLEAR POWER PLANT, UNITS 1 AND 2

The U.S. Nuclear Regulatory Commission (NRC, the Commission) has received an application dated August 12, 2020 (available in the NRC's Agencywide Documents Access and Management System (ADAMS) at Accession No. ML20225A303), from Pacific Gas and Electric Company (PG&E, the licensee), for exigent amendments to the facility operating licenses for the Diablo Canyon Nuclear Power Plant (Diablo Canyon), Units 1 and 2, located in San Luis Obispo County, California. The proposed amendments would avoid an unnecessary plant shutdown during the expected time needed to perform potential repairs to the Unit 1 auxiliary feedwater system (AFW) piping that PG&E conservatively anticipates may be identified during Diablo Canyon, Unit 1, Cycle 22, planned upcoming inspections to the AFW system. The licensee requested NRC approval by August 25, 2020.

The proposed exigent amendments would provide a new Technical Specification (TS) 3.7.5, "Auxiliary Feedwater (AFW) System," Condition G to address a one-time planned Diablo Canyon, Unit 1, Cycle 22, AFW system alignment for which current TS 3.7.5 would require shutdown. The new alignment proposed in Condition G is for one or two AFW trains inoperable in MODE 1, 2, or 3 due to inoperable AFW piping affecting the AFW flow path(s) to one steam generator. The new Condition G includes Required Actions to isolate flow to the affected steam generator within 2 hours and to restore the AFW System to OPERABLE status within 7 days. Because Diablo Canyon TSs are combined for Units 1 and 2, the changes are requested for Diablo Canyon Units 1 and 2. However, the proposed Condition G would include a Note specifying that the Condition is applicable to Unit 1 only. TS 3.7.5 Conditions B and D are correspondingly revised to add reference to new Condition G.

The licensee requested that the proposed amendments be processed on an exigent basis in accordance with the provisions in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.91(a)(6). Under 10 CFR 50.91(a)(6)(i), where the Commission finds that exigent circumstances exist, in that a licensee and the Commission must act quickly and that time does not permit the Commission to publish a *Federal Register* notice allowing 30 days for prior public comment, and it also determines that the amendment involves no significant hazards considerations, the Commission will either (A) issue a *Federal Register* notice providing notice of an opportunity for hearing and allowing at least two weeks from the date of the notice for prior public comment or (B) use local media to provide reasonable notice to the public in the area surrounding a licensee's facility of the licensee's amendment and of its proposed determination that no significant hazards consideration is involved, consulting with the licensee on the proposed media release and on the geographical area of its coverage. Due to the timing of the proposed amendments, the NRC is providing this notice in local media pursuant to 10 CFR 50.91(a)(6)(i)(B), and has consulted with the licensee and the NRC regional office on the proposed media release.

The licensee provided the following information to explain the exigency of the proposed amendments. Because of localized corrosion identified on Diablo Canyon, Unit 2, AFW piping during a recent Diablo Canyon, Unit 2, maintenance outage, the licensee intends to perform inspections of Diablo Canyon, Unit 1, AFW piping in the near term to ensure that Diablo Canyon, Unit 1, is not similarly affected. If similar below-minimum pipe wall thicknesses are found in the Unit 1 AFW system piping and elbows that were found in Unit 2, based on the estimated time-to-repair gained from the Unit 2 repair, it is likely that the current TS 3.7.5 Required Actions B.1 or D.1 would result in the required shutdown of Unit 1. The TS 3.7.5 change would avoid an unnecessary plant shutdown during the expected time needed to perform the potential repairs, and associated post-maintenance inspections and testing to the

Unit 1 AFW system piping. The licensee states that it has assessed the potential extent of the upcoming needed Unit 1 AFW system piping repairs based on the required repairs for Unit 2 and is making its best efforts to make a timely application and has not created the exigency. Accordingly, the licensee requested approval of this license amendment request on an exigent basis.

Based on this information, the NRC staff finds that exigent circumstances exist in that the licensee and the NRC must act quickly and that time does not permit the NRC staff to publish a Federal Register notice allowing 30 days for prior public comment.

As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration using the standards in 10 CFR 50.92. Under the NRC's regulations in 10 CFR 50.92(c), this means that operation of the facility in accordance with the proposed amendments would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. The licensee and the NRC staff have evaluated the proposed change with regard to the determination of whether or not a significant hazards consideration is involved as described below.

Operation of Diablo Canyon, Unit 1 in accordance with the proposed amendments will not involve a significant increase in the probability or consequences of an accident previously evaluated. The proposed change revises the requirements in TS 3.7.5, "Auxiliary Feedwater (AFW) System," to provide a new Condition G to address a one-time planned Unit 1 Cycle 22 AFW system alignment for which one or two AFW trains are inoperable in MODE 1, 2, or 3 due to inoperable AFW piping affecting the AFW flow path to one steam generator. The AFW System is not an initiator of any design basis accident or event, and therefore the proposed change does not increase the probability of any accident previously evaluated. The AFW

System is used to respond to accidents previously evaluated. The proposed change affects only the actions taken when portions of the AFW System are inoperable and does not affect the design of the AFW System. With the change to TS 3.7.5, adequate AFW cooling flow continues to be provided for accidents previously evaluated and there is no significant impact on accident consequences. No physical changes are made to the plant. The proposed change does not significantly change how the plant would mitigate an accident previously evaluated. Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed amendments will not create the possibility of a new or different kind of accident from any previously evaluated. The proposed change does not result in a change in the manner in which the AFW System provides plant protection. The AFW System will continue to remain available to supply water to three of the four steam generators while in the proposed TS 3.7.5 Condition G to remove decay heat and other residual heat by delivering at least the minimum required flow rate to provide adequate cooling. There are no design changes associated with the proposed changes. The changes to the Conditions and Required Actions do not change any existing accident scenarios, nor create any new or different accident scenarios. The change does not involve a physical alteration of the plant (i.e., no new or different type of equipment will be installed). The required manual control of one or two AFW level control valves is not an accident initiator. Therefore, the proposed change does not create the possibility of a new or different accident from any accident previously evaluated.

The proposed amendments will not involve a significant reduction in a margin of safety. The proposed change does not alter the manner in which safety limits, limiting safety system settings, or limiting conditions for operation are determined. The safety analysis acceptance criteria are not impacted by this change. The proposed change will not result in plant operation in a configuration outside the design basis since AFW cooling flow to two steam generators can

provide adequate core cooling. Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

Following an initial review of this application, the requested amendments have been evaluated against the standards in 10 CFR 50.92 and the NRC has made a proposed determination that the requested amendments involve no significant hazards considerations. The proposed change would not significantly increase the probability or consequences of any accident previously considered, nor create the possibility of a new or different kind of accident, nor significantly decrease any margin of safety.

The NRC is seeking public comments on this proposed determination that the amendments involve no significant hazards considerations. The NRC will consider comments received through August 21, 2020. If the proposed determination that the requested amendments involve no significant hazards consideration becomes final, the NRC will issue the amendments without first offering an opportunity for a public hearing. An opportunity for a hearing will be published in the *Federal Register* at a later date and any hearing request will not delay the effective date of the amendments.

The NRC may issue the amendments prior to the expiration of the comment period, should circumstances change such that failure to act in a timely way would result, for example, in derating or shutdown of the facility. Should the NRC take action prior to the expiration of the comment period it will publish in the *Federal Register* a notice of issuance. Before any issuance of the proposed amendments, the NRC will need to make the findings required by the Atomic Energy Act of 1954, as amended, and NRC regulations.

If the NRC decides in its final determination that the amendments do involve a significant hazards consideration, a notice of opportunity for a prior hearing will be published in the *Federal Register* and, if a hearing is granted, it will be held before the amendments are issued.

Comments on the proposed determination of no significant hazards consideration may be (1) telephoned to Jennifer Dixon-Herrity, Chief, Plant Licensing Branch 4, by collect call to 301-415-2967, (2) e-mailed to Jennifer.Dixon-Herrity@nrc.gov, or (3) submitted in writing to Office of Administration, Mail Stop: TWFN-7-A60M, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, ATTN: Program Management, Announcements and Editing Staff. All comments received by 5:00 pm EDT on August 21, 2020, will be considered in reaching a final determination.

A copy of the application may be examined electronically through the NRC's ADAMS in the NRC Library at <http://www.nrc.gov/reading-rm/adams.html> under Accession No. ML20225A303. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC Public Document Room (PDR) Reference staff by telephone at 1-800-397-4209, or 301-415-4737, or by e-mail to pdr.resource@nrc.gov.