

Chined States Fuerear Regulatory Commission

Protecting People and the Environment

Higher Burnup Workshop Will Begin Shortly...

- Public Meeting Slides:
 - NRC Presentations
 - 1st: ML20211L082
 - 2nd: ML20211L086
 - NEI: ML20211L080
 - EPRI: ML20211L077
 - Westinghouse: ML20211L091
 - International Perspective: ML20211089
- Please log into both webinar and bridgeline.
 Visuals will be through WebEx and audio will be through the bridgeline
 - <u>Webinar Link</u>
 - Bridgeline:
 - US: 888-378-9869
 - Int'l: +1 210-234-9856

Passcode: 9654996



United States Nuclear Regulatory Commission

Protecting People and the Environment

Higher Burnup Workshop

Overview of Higher Burnup and Increased Enrichment

July 30, 2020





- 10:00am 10:05am: Welcome/Introductions
- 10:05am 10:50am: Overview of Higher Burnup and Increased Enrichment
- 10:50am 11:30am: Presentations by NEI, EPRI, and Westinghouse
- 11:30am 11:45am: International Perspective and FIDES Program
- 11:45am 12:30pm: Public Question and Answer
- 12:30pm 1:30pm: Lunch Break
- 1:30pm 2:00pm: Schedules and Quality Submittals
- 2:00pm 2:30pm: Open Discussion
- 2:30pm 3:00pm: Public Comment Period



Welcome

Joe Donoghue – Director, Division of Safety Systems

Introductions

- Joan Olmstead, NMSS Meeting Facilitator
- Mike Orenak, NRR ATF Lead Project Manager
- Kevin Heller, NRR Higher Burnup and Increased Enrichment Technical Lead
- Marilyn Diaz, NMSS Chemical Engineer
- Ricardo Torres, NMSS Materials Engineer

Meeting Logistics



- Meeting visuals are through WebEx
- Meeting audio is through the bridgeline.
- Participants are in listen-only mode until the question and answer periods. The operator will open phone lines during the public discussion period.
- Today's meeting is a Category 2 meeting. Public participation and comments are sought during specific points during the meeting.
- No regulatory decisions will be made at today's meeting.

Purpose of Today's Meeting

- 1. Provide the general public with information about higher burnup and increased enrichment
- 2. Provide an open question and answer period on accident tolerant fuel subjects for the public.
- 3. Exchange of information with industry on higher burnup and increased enrichment, focusing on the components of a quality submittal.

What is ATF?

- After the Fukushima accident, Congress directed the US Department of Energy to fund research into fuels with enhanced accident tolerance, specifically:
 - improve performance during normal operation, anticipated operational occurrences, and loss of active cooling and other accidents
 - reducing fuel cost
 - reducing nuclear waste
- Accident tolerant fuel (ATF) is exclusively for the existing reactor fleet

ATF Technologies

- Near-term
 - Coated cladding
 - Doped pellets
 - FeCrAl cladding
- Longer term
 - Uranium nitride pellets
 - Silicon carbide cladding
 - Extruded metallic fuel
- Higher burnup (HBU) and increased enrichment (IE) are not ATF technologies, but are considered components of the ATF program

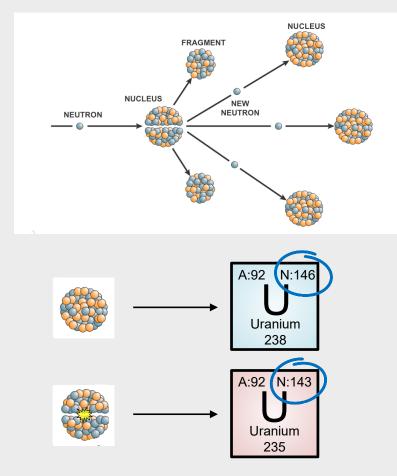
What is Increased Enrichment?

Increased Enrichment (IE) means increasing the amount of U-235 present in nuclear fuel to between 5% and 10% by weight.



But what does that mean?

What Does It Mean to Increase the Amount of Uranium-235? Nuclear fuel, fission, and uranium isotopes



What Does It Mean to Increase the Amount of Uranium-235? (cont'd)

Enrichment 0000000000000 000000000000 000000000 \frown 000000000 _____ 000000000 000000000000 0.7% "Enrichment" 0000000000 00000 (naturally occurring) 0000000000 0000000000000 5% Enrichment (current) 0000000000 00000000 000000000 \frown 00000000000 **10% Enrichment** (max)

What Does It Mean to Increase the Amount of Uranium-235? (cont'd)

- Historical enrichment levels
- Increased enrichment will have implications throughout the fuel cycle (enrichment facilities, transportation, use in-core, spent fuel storage).

Anticipated Impacts to **Enrichment** and **Fabrication Facilities**

- Facilities will require a license amendment.
 - Physical modifications to the facilities would likely be needed.
 - Focus areas:
 - Criticality Safety
 - Changes to Integrated Safety Analyses
- Licensees will need to evaluate changes following 10 CFR 70.72, "Facility Change and Change Process"

Anticipated Impacts to Transportation of Increased Enrichment Materials



- Feed material (uranium hexafluoride, UF₆)
 - Criticality safety
- Fresh fuel transport (assemblies)
 - Criticality safety
 - Configuration control
 - Normal conditions of transport
 - Hypothetical accident conditions

Anticipated Impacts to Increased Enrichment In-core

- Potential for higher power peaking
 - Additional burnable absorbers to suppress
- Increased enrichment safety when at reactor site
 - Criticality Safety
 - Decay heat

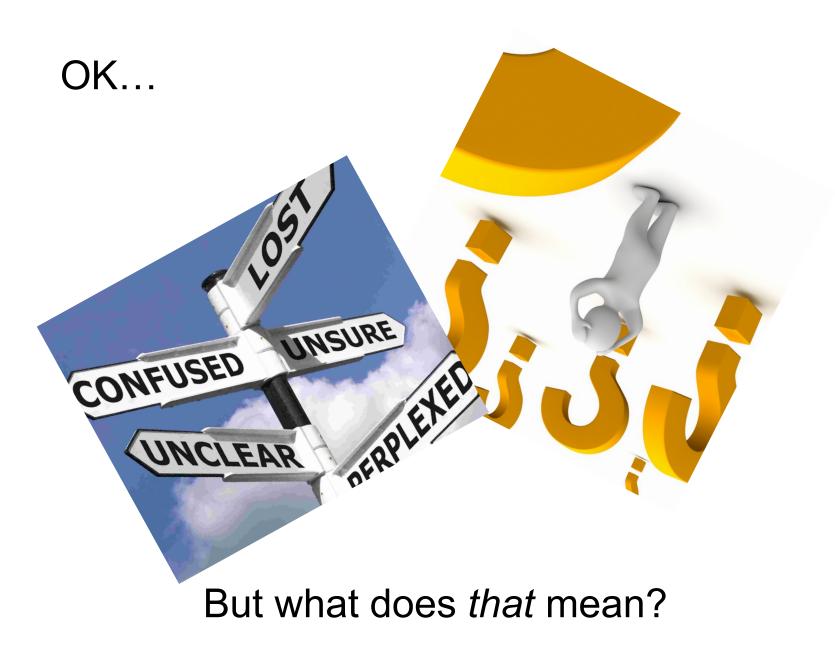
Increased Enrichment and Current Regulations



- Some regulations specify limits for material enrichments to less than 5 weight percent
 - 10 CFR 50.68(b)(7) Spent
 Fuel Pool
 - 10 CFR 71.55(g) UF₆

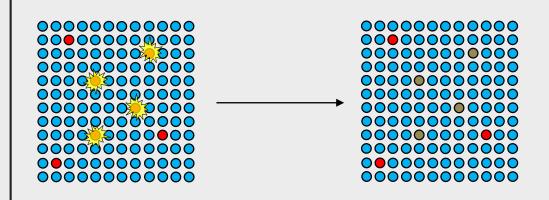
What is Higher Burnup?

Higher burnup (HBU) means increasing the amount of energy extracted from nuclear fuel to between 75 gigawatt-days per metric ton of uranium (GWd/MTU) and 80 GWd/MTU



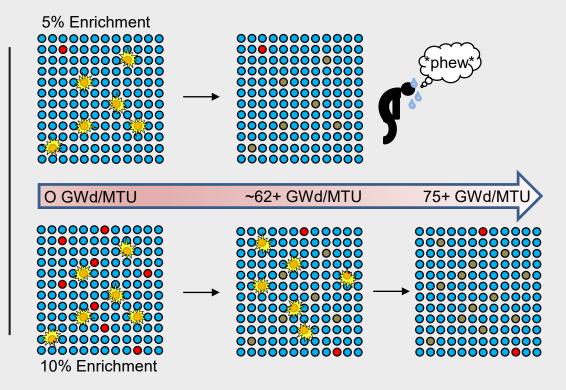
- Burnup is a measure of how much thermal energy is produced by the nuclear fuel over a given time frame divided by the initial mass of uranium
- It is also thought of as a measure of fuel depletion

 As uranium-235 fissions, energy is generated



- The total amount of uranium-235 present decreases, and the fuel begins to deplete
- Currently limited to 62 GWd/MTU

 Not enough uranium-235 present in current fuel to reach burnups much greater than ~62 GWd/MTU



 To reach higher burnups, fuel would need to start with more uranium-235

 Higher burnups would allow the fuel to be irradiated for longer periods in the reactor

 The amount of burnup affects the temperature, radioactivity and physical makeup of the irradiated fuel Anticipated Impacts to In-Reactor Performance

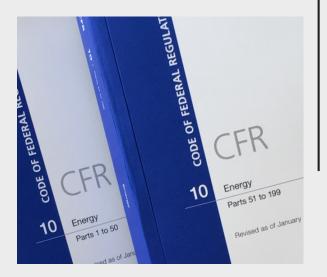
- Radioactivity of the irradiated fuel
- Heat generated by the irradiated fuel
- Characteristics of the irradiated fuel cladding and fuel pellet material
 - Fuel pellet fragmentation
 - Potential fuel relocation and dispersal in reactor
 - Fuel cladding mechanical performance and stresses

Anticipated Impacts to Spent Fuel Storage and Transportation



- Wet Storage (spent fuel pool)
 - Criticality safety
- Dry Storage
 - Criticality safety / Radiation shielding
 - Higher fuel temperatures and stresses
 - Age-related phenomena
- Transportation
 - Normal conditions of transport
 - Hypothetical accident condition

Higher Burnup and Current Regulations



• No explicit limits on burnups defined in current regulations

- Limitations placed on applicability of analysis methods
 - 10 CFR 50.34
 - Primary basis for applicability limits is range of experimental data



PROJECT PLAN TO PREPARE THE U.S. NUCLEAR REGULATORY COMMISSION FOR EFFICIENT AND EFFECTIVE LICENSING OF ACCIDENT TOLERANT FUELS

- ATF Project Plan
- Following Technological Progress
- CRAFT and ESCP

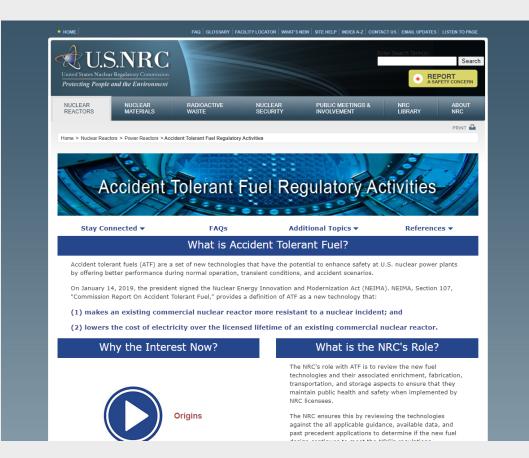
What is the

NRC Doing?

 Interaction with U.S. Department of Energy and International Counterparts



Revised ATF Website



Closing Remarks

If you have any remaining questions:

- Use the "Contact Us" on the ATF website
- Email: accident_tolerant_fuel@nrc.gov