



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION II
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200
ATLANTA, GEORGIA 30303-1200

July 23, 2020

Mr. Jim Barstow
Vice President, Nuclear Regulatory Affairs & Support Services
Tennessee Valley Authority
1101 Market Street, LP 4A-C
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH, UNITS 1 AND 2 – INTEGRATED INSPECTION REPORT
05000327/2020002 AND 05000328/2020002

Dear Mr. Barstow:

On June 30, 2020, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Sequoyah, Units 1 and 2. On July 6, 2020, the NRC inspectors discussed the results of this inspection with Mr. Matt Rasmussen and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

Thomas A. Stephen, Chief
Reactor Projects Branch 5
Division of Reactor Projects

Docket Nos. 05000327 and 05000328
License Nos. DPR-77 and DPR-79

Enclosure:
As stated

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SUBJECT: SEQUOYAH, UNITS 1 AND 2 – INTEGRATED INSPECTION REPORT
05000327/2020002 AND 05000328/2020002 DATED JULY 23, 2020

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ADAMS ACCESSION NUMBER: **ML20206K813**

| OFFICE | RII/DRP | RII/DRP | RII/DRP | RII/DRP |
|--------|-----------|-----------|------------|------------|
| NAME | N. Childs | S. Ninh | D. Hardage | T. Stephen |
| DATE | 7/21/2020 | 7/22/2020 | 7/22/2020 | 7/23/2020 |

U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Numbers: 05000327 and 05000328

License Numbers: DPR-77 and DPR-79

Report Numbers: 05000327/2020002 and 05000328/2020002

Enterprise Identifier: I-2020-002-0064

Licensee: Tennessee Valley Authority

Facility: Sequoyah, Units 1 and 2

Location: Soddy Daisy, TN 37379

Inspection Dates: April 1, 2020 to June 30, 2020

Inspectors: N. Childs, Resident Inspector
P. Cooper, Reactor Inspector
J. Diaz-Velez, Senior Health Physicist
D. Hardage, Senior Resident Inspector
J. Rivera, Health Physicist

Approved By: Thomas A. Stephen, Chief
Reactor Projects Branch 5
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Sequoyah, Units 1 and 2, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

| Type | Issue Number | Title | Report Section | Status |
|------|-------------------------------|---|----------------|--------|
| LER | 05000327,05000328/2019-004-00 | LER 2019-004-00 for Sequoyah, Unit 1, Automatic Actuation of Emergency Diesel Generators Due to Loss of Power to 6.9kV Shutdown Board | 71153 | Closed |

PLANT STATUS

Unit 1 began the inspection period at 100 percent rated thermal power (RTP). On May 13, a turbine governor control system malfunction caused lowering main steam header pressure which initiated an automatic safety injection and reactor trip. The unit was returned to 100 percent RTP on May 17 and remained at or near 100 percent RTP for the remainder of the inspection period.

Unit 2 began the inspection period at 87 percent RTP due to end of life coastdown conditions. On April 11, the unit was shutdown for refueling. The unit was returned to 100 percent RTP on May 5 and remained at or near 100 percent RTP for the remainder of the inspection period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the coronavirus (COVID-19), resident inspectors were directed to begin telework and to remotely access licensee information using available technology. During this time the resident inspectors performed periodic site visits each week and during that time conducted plant status activities as described in IMC 2515, Appendix D; observed risk significant activities; and completed on site portions of IPs. In addition, resident and regional baseline inspections were evaluated to determine if all or portion of the objectives and requirements stated in the IP could be performed remotely. If the inspections could be performed remotely, they were conducted per the applicable IP. In some cases, portions of an IP were completed remotely and on site. The inspections documented below met the objectives and requirements for completion of the IP.

REACTOR SAFETY

71111.04 - Equipment Alignment

Partial Walkdown Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) Unit 2 residual heat removal system with both trains in service for cooldown in Mode 5 with reactor coolant system level below the flange on April 13, 2020.
- (2) 2A emergency diesel generator following station blackout testing on April 20, 2020.
- (3) Unit 2 residual heat removal system after realignment for low head injection following refueling outage on May 4, 2020.

Complete Walkdown Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated system configurations during a complete walkdown of the Unit 1 and Unit 2 Motor Driven Auxiliary Feedwater (AFW) on June 18 and June 25, 2020.

71111.05 - Fire Protection

Fire Area Walkdown and Inspection Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) Emergency Raw Cooling Water Building on April 6, 2020
- (2) Diesel Generator Building, Elevation 740.5 on April 20, 2020
- (3) Unit 1 and Unit 2 Auxiliary Building, Elevation 653 on May 4, 2020
- (4) Unit 1 and Unit 2 Control Building, Elevation 706 on May 21, 2020
- (5) Unit 2 Auxiliary Building (Non-RCA), Elevation 734 on May 21, 2020

Fire Brigade Drill Performance Sample (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated fire brigade performance on June 11, 2020.

71111.08P - Inservice Inspection Activities (PWR)

PWR Inservice Inspection Activities Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors verified that the reactor coolant system boundary, steam generator tubes, reactor vessel internals, risk-significant piping system boundaries, and containment boundary are appropriately monitored for degradation and that repairs and replacements were appropriately fabricated, examined and accepted by reviewing the following activities:

03.01.a - Nondestructive Examination and Welding Activities.

- Ultrasonic Examination (UT), 3" Valve to Elbow, CVCF-213 , ASME Class 1, WO#119449412 (reviewed)
- Liquid Penetrant Examination (PT), 1" Pipe to Branch Connection, 2-FD-1609C4R0, ASME Class 2, weld package, WO#118769371 (reviewed)
- Enhanced Remote Visual (VE), RPV Closure Head and CRDM Penetration, WO#119809141. (reviewed)

03.01.c – Pressurized-Water Reactor Boric Acid Corrosion Control Activities.

- Evaluation of 2-FCV-063-0158

71111.11Q - Licensed Operator Regualification Program and Licensed Operator Performance

Licensed Operator Regualification Training/Examinations (IP Section 03.02) (1 Sample)

- (1) The inspectors observed and evaluated a simulator training scenario on June 8, 2020.

71111.12 - Maintenance Effectiveness

Maintenance Effectiveness (IP Section 03.01) (2 Samples)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

- (1) Part 21 Eval KCR-07 Battery Container Failure SQN Named (CR 1589441) .
- (2) 2-FCV-007-0003 Turbine Startup Drain Line Steam Break Failures (CR 1303568).

Quality Control (IP Section 03.02) (1 Sample)

The inspectors evaluated maintenance and quality control activities associated with the following equipment performance activity:

- (1) Quality critical maintenance to replace volume control tank (VCT) pressure transmitter 2-PT-62-122 (WO 120428331).

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed;

- (1) Unit 1 and 2, week of April 5 – April 10 including protected equipment status reviews for scheduled maintenance on Unit 2 steam driven Auxiliary Feed Water (AFW) turbine trip and throttle valve and the 1A2 component cooling system heat exchanger.
- (2) Unit 2 Yellow shutdown risk week of April 12-18 while reactor coolant system (RCS) level was below Reactor flange level and during Reactor core offload, including review of defense in depth protected equipment for U2R23.
- (3) Unit 2 Yellow shutdown risk week of April 19-24 while reactor coolant system (RCS) level was below Reactor flange level following core onload, including review of defense in depth protected equipment for U2R23.

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (5 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) MSIV 2-FCV-1-22, Steam Generator #3 Main Steam Header Isolation Valve, high friction after adjustment.

- (2) Foreign material found in Unit 2 Reactor Vessel Level Indication System (RVLIS) Line (CR 1600942).
- (3) 2-FCV-63-22 leaks by when indicating closed (CR 1603527).
- (4) Perform evaluation of 2-FSV-68-397, Reactor Head Vent Throttle Valve (CR 1605225).
- (5) Centrifugal Charging Pump (CCP) Room Cooler Potential Design Basis Heat Load Issue (CR 1609392).

71111.18 - Plant Modifications

Temporary Modifications and/or Permanent Modifications (IP Section 03.01 and/or 03.02) (1 Sample)

The inspectors evaluated the following temporary or permanent modifications:

- (1) Design Change Package 22703, Upgrade of the Turbine Driven Auxiliary Feedwater Speed Governor and Flow Controller.

71111.19 - Post-Maintenance Testing

Post-Maintenance Test Sample (IP Section 03.01) (4 Samples)

The inspectors evaluated the following post maintenance test activities to verify system operability and functionality:

- (1) WO 120164119, repair 2-FCV-62-72-A, Regen Heat Exchanger Letdown Isolation Valve, due to local leak rate test failure, on April 12, 2020.
- (2) WO 115152446, Replace Power Operated Relief Valve (PORV) SQN-2- 068-0334B, on April 29, 2020.
- (3) WO 118769371, Replace SQN-2-VLV-002-0282A, on May 3, 2020.
- (4) WO 120418929, Replace TE-68-25A-E, RCS Loop 2 Hot Leg Resistance Temperature Detector (RTD), on May 4, 2020.

71111.20 - Refueling and Other Outage Activities

Refueling/Other Outage Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated refueling outage U2R23 activities from April 10, 2020 to May 4, 2020.

71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

Surveillance Tests (other) (IP Section 03.01) (2 Samples)

- (1) 0-SI-OPS-082-026.A, Loss of Offsite Power With Safety Injection - D/G 2A-A Test on April 12, 2020.
- (2) 0-SI-SXV-068-201.0, Pressurizer PORV Operability Test performed prior to being required for LTOP on April 24, 2020.

Containment Isolation Valve Testing (IP Section 03.01) (1 Sample)

- (1) 0-SI-SLT-062-258.1, Unit 2 - Containment Isolation Valve – Local Leak Rate Test Chemical and Volume Control System, on April 12, 2020.

Ice Condenser Testing (IP Section 03.01) (1 Sample)

- (1) 0-SI-MIN-061-105.0, Ice Condenser – Ice Weighing on April 22, 2020.

RADIATION SAFETY

71124.05 April 2020 - April 2020 Radiation Monitoring Instrumentation

April 2020 Calibration and Testing Program (IP Section 03.02) (14 Samples)

The inspectors evaluated the calibration and testing of the following radiation detection instruments:

- (1) Unit 1 Containment High Range Monitor 1-RE-90-271 (Upper Containment)
- (2) Unit 1 Containment High Range Monitor 1-RE-90-274 (Lower Containment)
- (3) Whole Body Counter, TVA No. 2
- (4) Perkin-Elmer Tri Carb Liquid Scintillation Counter No. 2203
- (5) Count Room Gamma Detector No. 2
- (6) ARGOS Personnel Contamination Monitor, TVA No. 951453
- (7) CRONOS Small Article Monitor, TVA No. 951456
- (8) GEM-5 Portal Monitor, TVA No. 18pm
- (9) Mirion Telepole, TVA No. 860096
- (10) Ludlum 12-4, TVA No. 860290
- (11) Thermo Radeye GX, TVA No. 951143
- (12) Mirion AMP-100, TVA No. 951485
- (13) DMC-3000 ED, TVA No. A0B724
- (14) AMS-4 Continuous Air Monitor, TVA No. 860272

April 2020 Effluent Monitoring Calibration and Testing Program Sample (IP Sample 03.03) (3 Samples)

The inspectors evaluated the calibration and maintenance of the following radioactive effluent monitoring and measurement instrumentation:

- (1) Auxiliary Building Vent Gaseous Radiation Monitor, 0-R-90-101B
- (2) Waste Disposal System Liquid Effluent Radiation Monitor, 0-R-90-122
- (3) Auxiliary Building Gas Treatment System Filter, Train A

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

IE01: Unplanned Scrams per 7000 Critical Hours Sample (IP Section 02.01) (2 Samples)

- (1) Unit 1 (April 2019 - March 2020)
- (2) Unit 2 (April 2019 - March 2020)

IE03: Unplanned Power Changes per 7000 Critical Hours Sample (IP Section 02.02) (2 Samples)

- (1) Unit 1 (April 2019 - March 2020)
- (2) Unit 2 (April 2019 - March 2020)

IE04: Unplanned Scrams with Complications (USwC) Sample (IP Section 02.03) (2 Samples)

- (1) Unit 1 (April 2019 - March 2020)
- (2) Unit 2 (April 2019 - March 2020)

71152 - Problem Identification and Resolution

Semiannual Trend Review (IP Section 02.02) (1 Sample)

- (1) The inspectors reviewed the licensee's corrective action program for potential adverse trends in operationally impactful equipment failures that might be indicative of a more significant safety issue (CR 1573093).

Annual Follow-up of Selected Issues (IP Section 02.03) (1 Sample)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issues:

- (1) Steam leak on turbine drain line results in reactor downpower (CR 1605906).

71153 - Followup of Events and Notices of Enforcement Discretion

Event Followup (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated Unit 1 reactor trip and safety injection and licensee's response on May 13, 2020.

Event Report (IP Section 03.02) (1 Sample)

The inspectors evaluated the following licensee event reports (LERs):

- (1) LER 05000327/2019-004-00, Automatic Actuation of Emergency Diesel Generators Due to Loss of Power to 6.9kV Shutdown Board (ADAMS accession: ML20044D623). The inspectors determined that it was not reasonable to foresee or correct the cause discussed in the LER therefore no performance deficiency was identified. The inspectors also concluded that no violation of NRC requirements occurred.

INSPECTION RESULTS

No findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On July 6, 2020, the inspectors presented the integrated inspection results to Mr. Matt Rasmussen and other members of the licensee staff.
- On April 29, 2020, the inspectors presented the Informal debrief with RP and Licensing for the completion of 2 out of 3 samples in IP 71124.05 (Covid-19 remote inspection). Formal RP exit meeting with senior site management to be conducted when the last sample of the IP can be completed onsite. Inspection results were presented to Lee Baxley (Radiation Protection Manager) and Andrew McNeil (Licensing) and other members of the licensee staff.

DOCUMENTS REVIEWED

| Inspection Procedure | Type | Designation | Description or Title | Revision or Date |
|----------------------|-----------------------------|--|---|------------------|
| 71111.04 | Procedures | 0- SI-OPS-082-007.0 | Diesel Generator Operability Verification | Revision 20 |
| | | 0-SO-74-1 | Residual Heat Removal System | Revision 107 |
| | | 0-SO-82-3 | Diesel Generator 2A-A | Revision 67 |
| | | 1,2-SO-3-2 Attachment 1 | Auxiliary Feedwater System Power Checklist | |
| | | 1,2-SO-3-2 Attachment 2 | Auxiliary Feedwater System Valve Checklist | |
| 71111.05 | Corrective Action Documents | CR 1615936 | Observations from the 6/11/20 SQN Fire Drill | 06/11/2020 |
| | | CR 1617501 | Fire Drills Lessons Learned 2nd Quarter | 06/18/2020 |
| | Fire Plans | AUX-0-653-00 | Pre-Fire Plan Auxiliary Building, Elevation 653 | Revision 9 |
| | | AUX-0-734-01 | Pre-Fire Plan Auxiliary Building, Elevation 734 (Unit 1 Non-RCA Side) | Revision 8 |
| | | AUX-0-734-02 | Pre-Fire Plan Auxiliary Building, Elevation 734 (Unit 2 Non-RCA Side) | Revision 7 |
| | | CON-0-706-00 | Pre-Fire Plan Control Building, Elevation 706 | Revision 6 |
| | | DGB-0-740.5-00 | Pre-Fire Plan Diesel Generator Building, Elevation 740.5 | Revision 6 |
| | | ERCW-0-688-00 | Pre-Fire Plan Essential Raw Cooling Water Pump Station, Elevation 688 | Revision 2 |
| | | ERCW-0-704-00 | Pre-Fire Plan Essential Raw Cooling Water Pump Station, Elevation 704 | Revision 2 |
| | | ERCW-0-720-00 | Pre-Fire Plan Essential Raw Cooling Water Pump Station, Elevation 720 | Revision 3 |
| | Miscellaneous | | Fire Drill Evaluation Report - Yard Near Solar Building | 05/07/2020 |
| | Procedures | 0-PI-FPU-317-531.Y | Fire Extinguisher Annual Inspection | Revision 9 |
| 71111.11Q | Miscellaneous | SEG OPL273S2007 | Steam Generator Tube Rupture with loss of C-9 Interlock | Revision 0 |
| 71111.12 | Corrective Action Documents | CR 1303568 Equipment Failure Investigation | Unit 2 Turbine and Reactor Shutdown due main turbine drain line steam break | 08/04/2017 |

| Inspection Procedure | Type | Designation | Description or Title | Revision or Date |
|----------------------|-------------------------|----------------------------------|--|------------------|
| | Miscellaneous | CR 1585280 | Seven DG B Battery Cells Have Jar Cracks | 02/07/2020 |
| | | CR 1585746 | Extent of Condition for BFN DG Battery Jar Cracking | 02/10/2020 |
| | | CDE 2897 | June 4, 2017 Failure of Turbine Startup Drain No. 3 | Revision 1 |
| | | CDE 2986 | June 2, 2017 Failure of Turbine Startup Drain No. 3 | Revision 1 |
| | | TVA Central Lab Technical Report | Sequoyah Unit 2 Main Steam Drain Failures | 06/20/2017 |
| | Procedures | NPG-SPP-07.2.16 | Quality Critical Maintenance Identification and Oversight | Revision 6 |
| | | TVA-NQA-PLN89-A | Nuclear Quality Assurance Plan (NQAP) Quality Assurance Program Description | Revision 38 |
| 71111.13 | Procedures | 0-GO-16 | System Operability Checklist | Revision 25 |
| 71111.15 | Drawings | 2-47A595-3 | Set-Up Parameters for Air Operated Valves, Setup Box for 2-FCV-1-22 | Revision 3 |
| | Engineering Evaluations | EWR-20-DEM-001069 | U2R23 MSIV 2-FCV-1-22 High Friction after Adjustment Accept As Is Evaluation for CR 1603849 | 04/26/2020 |
| | Operability Evaluations | | Prompt Determination of Operability for CR 1609392, CCP Room Cooler Design Basis Heat Load Issue | 05/29/2020 |
| | Procedures | 2-PI-ICC-068-397 | Channel Calibration of Reactor Vessel Vent Throttle Valve Loop 2-F-68-397 and Vent Valve 2-FSV-68-397 | Revision 10 |
| | | MMTP-154 | Air Operated Valve Diagnostic Testing | Revision 0 |
| | Work Orders | WO 121314149 | 2-FSV-068-397 not operating properly | 05/20/2020 |
| 71111.19 | Procedures | 0-SI-SXV-068-201.0 | Pressurizer PORV Operability Test | Revision 2 |
| | | 2-SI-ICC-068-025.2 | Channel Calibration of Delta T/ Tave Channel II, Rack 6 Loop 2-T-68-25 | Revision 25 |
| | | 2-SI-IFT-003-519.2 | Functional Test of Environmental Allowance Monitor / Trip Time Delay Protection Set II | Revision 23 |
| | | 2-SI-IRT-099-13A.0 | Response Time Test of RTD's in Loops 2-T-68-2 and 2-T-68-25 Using Analysis and Measurement Services Techniques Cycle A | Revision 11 |
| | | 2-SI-SXI-068-201.0 | Leakage Test of the Reactor Coolant Pressure Boundary | Revision 16 |
| | Work Orders | 121274792 | Pen X-15 (Letdown) Failed As Found LLRT | 04/20/2020 |
| | | 121282200 | Troubleshoot and Increase Seating Load on 2-FCV-62-72-A | 04/20/2020 |

| Inspection Procedure | Type | Designation | Description or Title | Revision or Date |
|------------------------|-----------------------------|--|---|------------------|
| 71111.20 | Miscellaneous | | Unit 2 Cycle 23 Outage Safety Plan | Revision 0 |
| | | SQN-2-130 | Fuel Transfer Operation - U2R23 Core Offload | Revision 0 |
| | | SQN-2-134 | Fuel Transfer Operation - Core Reload | Revision 0 |
| | Procedures | 0-GO-5 | Normal Power Operation | Revision 97 |
| | | 0-GO-6 | Power Reduction from 30% Reactor Power to Hot Standby | Revision 65 |
| | | 0-GO-7 | Unit Shutdown from Hot Standby to Cold Shutdown | Revision 88 |
| | | 0-SI-SXX-068-127.0 | RCS and Pressurizer Temperature and Pressure Limits | Revision 15 |
| | | E-0 | Reactor Trip or Safety Injection | Revision 41 |
| | | ES-0.1 | Reactor Trip Response | Revision 39 |
| 71111.22 | Corrective Action Documents | CR 1600918 | Pen X-15 (Letdown) Failed as found LLRT | 04/12/2020 |
| | | CR 1605537 | U2 Ice Condenser Frozen Basket Trend | 05/03/2020 |
| | Miscellaneous | 0-SI-SLT-062-258.1 | Unit 2 - Containment Isolation Valve Local Leak Rate Test Chemical and Volume Control System, completed performance | 04/30/2017 |
| | | U2R23.OUT | ICE, Computer Program Version SQ1.1 | 04/22/2020 |
| | NDE Reports | 0-SI-SLT-0692-258.1 | Unit 2 - Containment Isolation Valve Local Leak Rate Test Chemical and Volume Control System, completed performance | 05/10/17 |
| | | | | |
| 71124.05 April 2020 | Corrective Action Documents | CR 1554447 | Upper Containment Rad Monitor drifting into alarm | 10/03/2019 |
| | | CR 1569547 | FY19 Annual and 4th Quarter RP Instrument Performance Trend Review | 11/29/2019 |
| | | CR 1580287 | 2-RE-90-130 failed to respond to source check | 01/21/2020 |
| | Miscellaneous | Sequoia Nuclear Plant | Offsite Dose Calculation Manual (ODCM) | Rev. 0063 |
| | Procedures | 0-TI-CEM-016-001.4 | Shield Building Radiation Monitor Sampling Methods | Rev. 0002 |
| | | RCDP-8 | Radiological Instrumentation/Equipment Controls | Rev. 0008 |
| | | RCI-05 | Radiation Protection Instrumentation Program | Rev. 0083 |
| | Self-Assessments | SQN-RP-SA-18-013 | Radiation Monitoring Instrumentation | 10/12/2018 |
| | | | | |
| 71152 | Corrective Action Documents | CR 1605906 Level 2 Evaluation Report | Steam Leak on Turbine Drain Line Results in Reactor Downpower | 06/04/2020 |

| Inspection Procedure | Type | Designation | Description or Title | Revision or Date |
|----------------------|-----------------------------|-------------|---|------------------|
| 71153 | Corrective Action Documents | CR 1115176 | GR5 Relays from ABB have incorrect components | 02/25/2016 |
| | | CR 1142038 | Part 21 evaluation for ABB GR5 Relays | 03/03/2016 |
| | | CR 1573472 | Loss of 1B-B Shutdown Board Offsite Power | 12/16/2019 |
| | | CR 1573726 | 250 VDC Station Battery system voltage excursion | 12/16/2019 |
| | | CR 1607726 | Transient on Unit 1 Reactor Trip and Safety Injection | 05/13/2020 |