

50-321



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 7, 1997

Mr. H. L. Sumner, Jr.
Vice President
Southern Nuclear Operating
Company, Inc.
P. O. Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: SAFETY EVALUATION FOR THE THIRD TEN-YEAR INTERVAL FOR THE PUMP AND VALVE INSERVICE TESTING PROGRAM - EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2 (TAC NOS. M95727 AND M95728)

Dear Mr. Sumner:

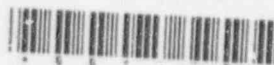
By letter dated September 15, 1995, Georgia Power Company (GPC), submitted the Third Ten-Year Interval Inservice Testing (IST) Program for the Pumps and Valves at Edwin I. Hatch Nuclear Plant, Units 1 and 2. Your submittal included several proposed relief requests, deferred test justifications, and other sections of the IST program developed according to the requirements of the American Society of Mechanical Engineers (ASME) OM Code 1990 Edition for pump and valve testing, with the exception of safety relief valve testing, which was generated in accordance with the requirements of the ASME OM Code 1995 Edition.

By letter dated April 12, 1996, the NRC transmitted a Safety Evaluation (SE) that provided the staff's review of your above IST relief requests. Two relief requests were denied and one relief request was granted on a provisional basis. In addition, the staff informed you of concerns regarding four relief requests, which were granted on an interim or provisional basis during the second 10-year interval. Consequently, you were requested to provide a response to the specific issues raised in the evaluation within 60 days of the date of the third 10-year interval SE.

By letter dated June 4, 1996, you addressed the items which were denied, required a response within 60 days, or were granted provisionally. You also provided an additional response dated July 24, 1996, in which revisions to Relief Requests RR-V-4, RR-V-8, and RR-V-9 were submitted. On December 2, 1996, you submitted revised Relief Request RR-P-10, which included an evaluation of test parameters under current operating conditions, due to a recent power uprate. In addition, you submitted a new Relief Request, RR-P-12, that was developed to address the use of control room instrumentation for high pressure coolant injection discharge pressure monitoring during performance of IST.

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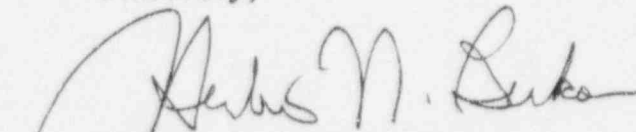
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The results of the staff's review of your responses to the staff's April 12 SE and other supplemental letters, are provided in the enclosed SE. Specifically: (1) Relief Requests RR-P-10 and RR-P-12 are authorized pursuant to 10 CFR 50.55a(a)(3)(i) based on the determination that your proposal provides an acceptable level of quality and safety; (2) Relief Requests RR-P-9 and RR-V-2 were withdrawn by GPC; (3) Revised Relief Requests RR-V-4, RR-V-8, and RR-V-9, have now satisfied the conditions placed on interim relief, for the second 10-year interval, in the staff's April 12 SE; (4) Revised Relief Requests RR-V-4, RR-V-8, and RR-V-9 are granted as impractical for the third 10-year interval pursuant to 10 CFR 50.55a(f)(6)(i); (5) Refueling Outage Justification ROJ-V-2, has satisfied the provisional conditions stated in the staff's April 12 SE, but GPC should redesignate this ROJ as a relief request--no further staff review is needed; and (6) GPC has clarified the anomaly 7.9-Pump Note 7 regarding the diesel fuel oil transfer pumps, and anomaly 7.12-Valve Note 13 on the residual heat removal minimum flow line valves.

In the April 12 letter, the staff stated its disagreement with GPC's information pertaining to the exclusion of the reactor core isolation cooling (RCIC) system from its IST program. In a separate evaluation, the staff will address anomaly 7.8-Pump Note 3, and anomaly 7.14-Valve Notes 16, 17, 19, and 20, regarding whether the RCIC system (other than its containment isolation function) may be removed from the IST program.

This task will remain open until the NRC staff completes its review of your plan to delete the RCIC system from the IST program.

Sincerely,



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

Enclosure: Safety Evaluation

cc w/encl: See next page

July 7, 1997

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Sincerely,

ORIGINAL SIGNED BY:

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

Enclosure: Safety Evaluation

cc w/encl: See next page

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