



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PORTLAND GENERAL ELECTRIC COMPANY

THE CITY OF EUGENE, OREGON

PACIFIC POWER AND LIGHT COMPANY

DOCKET NO. 50-344

TROJAN NUCLEAR PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 140  
License No. NPF-1

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Portland General Electric Company, et al., (the licensee) dated January 15, 1988, as revised February 15, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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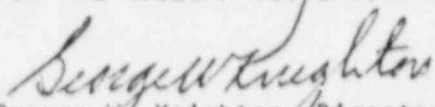
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-1 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 140, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
George W. Knighton, Director  
Project Directorate V  
Division of Reactor Projects - III,  
IV, V and Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 11, 1988

April 11, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 140

TO FACILITY OPERATING LICENSE NO. NPF-1

DOCKET NO. 50-344

Revise Appendix A as follows:

Remove Pages

6-2

6-3

6-6

Insert Pages

6-2

6-3

6-6

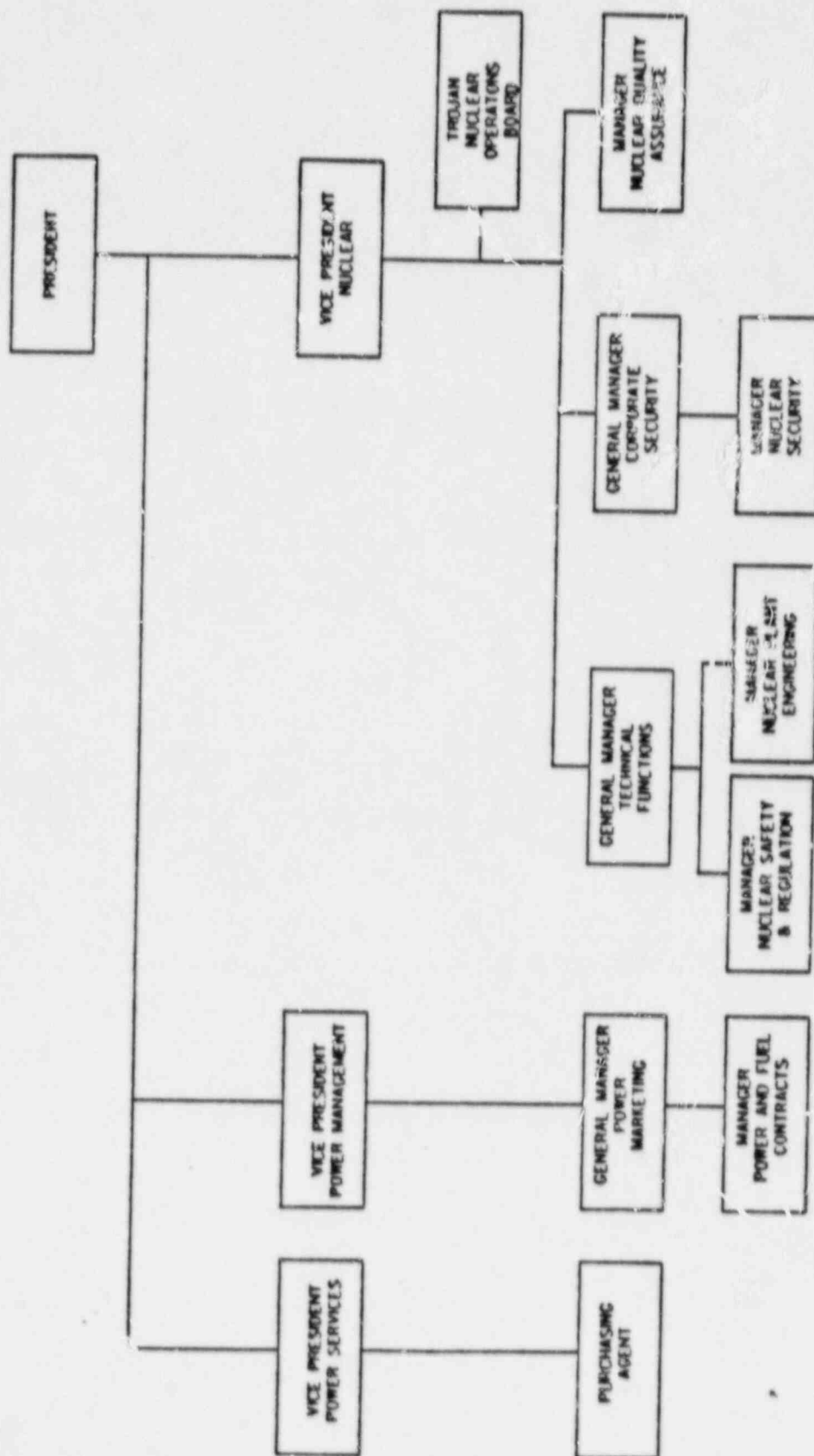


FIGURE 6.2-1 OFFSITE ORGANIZATION



## ADMINISTRATIVE CONTROLS

### COMPOSITION

6.5.1.2 The Plant Review Board shall be composed of the:

|           |   |
|-----------|---|
| Chairman: | Manager, Technical Services                 |
| Member:   | System Engineering Branch Manager           |
| Member:   | Maintenance Branch Manager                  |
| Member:   | Operations Branch Manager                   |
| Member:   | Quality Assurance Operations Branch Manager |
| Member:   | Radiation Protection Branch Manager         |

### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PRB activities at any one time.

### MEETING FREQUENCY

6.5.1.4 The PRB shall meet at least once per calendar month and as convened by the PRB Chairman or his designated alternate.

### QUORUM

6.5.1.5 A quorum of the PRB shall consist of the Chairman or his designated alternate and at least four members including alternates.

### RESPONSIBILITIES

6.5.1.6 The Plant Review Board shall be responsible for:

- a. Review of the safety evaluations for: (1) procedures, (2) change to procedures, equipment, systems or facilities, and (3) tests or experiments completed under the provision of 10 CFR 50.59 to verify that such actions did not constitute an unreviewed safety question.
- b. Review of proposed procedures and changes to procedures, equipment, systems or facilities which may involve an unreviewed safety question as defined in 10 CFR 50.59 or involves a change in Technical Specifications.
- c. Review of proposed test or experiments which may involve an unreviewed safety question as defined in 10 CFR 50.59 or requires a change in Technical Specifications.
- d. Review of proposed changes to Technical Specifications or the Operating License.