UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PUBLIC SERVICE COMPANY OF COLORADO

DOCKET NO. 50-267

FORT ST. VRAIN NUCLEAR GENERATING STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59
License No. DPR-34

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Public Service Company of Colorado (the licensee) dated December 23, 1986 as supplemented December 17, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. DPR-34 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 59 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

José A. Calvo

Jose A. Calvo, Director
Project Directorate - IV
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 7, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 59

FACILITY OPERATING LICENSE NO. DPR-34

DOCKET NO. 50-267

Replace the following pages of the Appendix A Technical Specifications with the attached pages as indicated. the revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	Insert
4.3-7	4.3-7
4.3-8	4.3-8
4.3-9 to 4.3-16	-
5.3-9	5.3-9
5.3-10	5.3-10
5.3-11	5.3-11
5.3-15	5.3-15

Specification LCO 4.3.10 - Shock Suppressors (Snubbers) - Limiting Condition for Operation

a) The reactor shall not be operated at power unless all shock suppressors (snubbers) supporting Class I piping systems are operable except as noted in b) through d) of this LCO.

b) With one or more snubbers removed from service in a Class I system or subsystem, within 72 hours replace or restore the snubber(s) to operable status. If one or more snubber(s) is found inoperable, within 72 hours replace or restore the inoperable snubber(s) and perform an engineering evaluation per Specification SR 5.3.8.c) on the supported component or declare the supported system inoperable and follow the appropriate action statement for that system.

c) If the requirements of a) and b) of this LCO cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a low power condition within 36 hours.

d) If a shock suppressor is determined to be inoperable while the reactor is in the shutdown or refueling mode, the suppressor shall be made operable or replaced prior to reactor operation at power.

Basis for Specification LCO 4.3.10

Shock suppressors (snubbers) are designed to prevent unrestrained pipe motion under dynamic loads, as might occur during an earthquake, while allowing normal thermal motion during startup and shutdown. The consequence of an inoperable snubber is an increase in the probability of structural damage to piping resulting from the dynamic loads produced by a seismic event. It is therefore necessary that all snubbers required to protect the Class I systems, subsystems, or components be operable during reactor power operation.

Because snubber protection is required only during relatively low probability events, a period of 72 hours is allowed for repair or replacement. In case a shutdown is required, the allowance of 36 hours to reach a low power condition will permit an orderly power reduction consistent with standard operating procedures. Since reactor operation at power should not be conducted with defective safety-related equipment, reactor power operation is prohibited with inoperable snubbers, except as stated above.

activity level decreases to less than 10% of the limit of LCO 4.3.8, at which time weekly sampling may be resumed.

Basis for Specification SR 5.3.7

The specification surveillance interval is adequate to monitor the activity of the secondary coolant.

Specification SR 5.3.8 - Shock Suppressors (Snubbers) Surveillance

The following surveillance requirements apply to all Class I piping system shock suppressors (snubbers):

a) Visual Inspections

The first in-service visual inspection of snubbers shall be performed within six months from issuance of this Technical Specification (Amendment 39). For the purpose of entering the schedule described in this section, it shall be assumed that the facility had been on a six-month inspection interval.

The first in-service visual inspection of snubbers shall include all Class I piping system snubbers. If less than two snubbers are found inoperable during the first in-service visual inspection, the second in-service visual inspection shall be performed 12 months plus or minus 25% from the date of the first inspection. Otherwise, subsequent visual inspections shall be performed in accordance with the following schedule:

Number of Inoperable Snubbers per Inspection Period	Subsequent Visual Inspection Period*
0	18 Months plus or minus 25%
1	12 Months plus or minus 25%
2	6 Months plus or minus 25%
3, 4	124 Days plus or minus 25%
5, 6, 7	62 Days plus or minus 25%
8 or more	31 Days plus or minus 25%

*The inspection interval shall not be lengthened more than one step at a time.

b) Visual Inspection Acceptance Criteria

Visual inspections shall verify (1) that there are no visible indications of damage or impaired operability, (2) attachments to the foundation or supporting structure are secure, and (3) in those locations where snubber movement can be manually induced without disconnecting the snubber, that the snubber has freedom of movement and is not seized. Snubbers which appear inoperable as a result of visual

inspections may be determined operable for the purpose of establishing the next visual inspection interval, providing that (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers that may be generically susceptible, and (2) the affected snubber is functionally tested in the as-found condition and determined operable per Sections 5.3.8.d) and 5.3.8.e). However, when the fluid port of a hydraulic snubber is found to be uncovered, the snubber shall be determined inoperable and cannot be determined OPERABLE via functional testing for the purpose of establishing the next visual inspection interval. All snubbers connected to an inoperable common hydraulic fluid reservoir shall be counted as inoperable snubbers.

c) Functional Tests

At least once per 18 months (not to exceed 22 1/2 months), a representative sample of each type of snubber shall be tested using one of the following sample plans: The sample plan(s) shall be selected prior to the test period and cannot be changed during the test period. The NRC Regional Administrator shall be notified in writing of the sample plan(s) selected for each type of snubber prior to the test period or the sample plan(s) used in the prior test period shall be implemented:

1) At least 10% of the total of the type of snubber shall be functionally tested either in-place or in a bench test. For each snubber of that type that does not meet the functional test acceptance criteria of Specification SR 5.3.8.d) or SR 5.3.8.e), an additional 10% of that type of

f) Exemption From Visual Inspection or Functional Tests

Permanent or other exemptions from the surveillance program for individual snubbers may be granted by the Commission if a justifiable basis for exemption is presented.

g) Record Keeping

Record keeping shall consist of:

1) A historical record for each snubber shall be maintained.

2) Concurrent with the first in-service visual inspection and at least once per refueling cycle thereafter, the historical records for each snubber shall be reviewed to determine any trends that may adversely affect service life.

3) The maximum expected service life for the various seals, seal materials, and applications shall be determined and established based on engineering information and the seals shall be replaced so that the maximum expected service life will not be exceeded during a period when the snubber is required to be OPERABLE. This monitoring program shall be fully implemented within 22 1/2 months from the issuance of this Technical Specification (Amendment 39).