



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 25, 1997

LICENSEE: Wolf Creek Nuclear Operating Corporation
FACILITY: Wolf Creek Generating Station
SUBJECT: STATUS OF OUTSTANDING LICENSING ACTIONS AND UPCOMING REQUESTS

Background

On June 3, 1997, I met with members of the Wolf Creek Nuclear Operating Corporation's (WCNOC) staff to discuss the status of licensing actions currently under review by the NRR staff and to discuss any upcoming requests the licensee has identified. This meeting was held at the Wolf Creek site.

Summary

1. The status of the following amendment requests was discussed:

a. ECCS Accumulator Allowed Outage Time (TAC M91859)

This amendment would increase the allowed outage time (AOT) for an ECCS accumulator from 1 hour to 24 hours for conditions other than boron concentration outside the allowable band and would add a new action statement that would allow 72 hours to return the boron concentration in the accumulator to within the allowable band.

The 72-hour AOT for boron concentration is in accordance with Generic Letter 93-05. The 24-hour AOT is based on a risk argument, supplemented with administrative controls to ensure key safety systems are in service prior to removing the accumulator from service. A similar request for Combustion Engineering plants is being reviewed by the staff. The staff is reviewing the Wolf Creek request. The licensee has submitted the request for the improved standard technical specifications which assumes that this amendment request has been approved.

b. Removal of Containment Isolation Valve Tables (TAC M91899)

This amendment would remove the containment isolation valve tables from the technical specifications to a licensee controlled document. This amendment request is in accordance with Generic Letter 91-08. The request has been reviewed and the amendment package has been prepared.

c. Change in the In _____ Method of the _____ for Coolant Pump Flywheel
(TAC M97362)

This request would change the _____ method for the reactor coolant pump (RCP) flywheel to _____ rather than a surface examination of

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an ultrasonic examination and also change the frequency of the examination. For the "D" RCP a one time extension of the inspection interval has been included. The amendment will be issued during the week of June 23, 1997.

d. Use of ZIRLO Clad Fuel (TAC M98204)

This amendment would allow the use of ZIRLO clad fuel. The staff is reviewing this request. A supplement may be required to add the Westinghouse topical to the list of methodologies referenced in the technical specifications for use in determining limits in the Core Operating Limits Report. Approval has been requested before the start of the September 20, 1997, refueling outage.

c. Containment Penetration Control During Refueling (TAC M98508)

This request would allow the breathing air and service air containment penetrations to remain open, under administrative controls, during fuel movement. This amendment will be issued during the week of July 7, 1997.

d. Conversion to the Improved Standard Technical Specifications (TAC M98738)

This amendment would convert to the new improved standard technical specifications and was prepared in concert with Callaway, Diablo Canyon and Comanche Peak. This application has just been received. Staff review has not started, pending receipt of the Diablo Canyon application.

2. The status of the following licensing activities was discussed:

a. Exemption from 10 CFR 70.24, Criticality Monitors (TAC M89161)

This would exempt the licensee from the requirements to have criticality accident monitors in the new fuel storage area and conduct emergency drills in response to the alarms. Additional information has been received and the issue is currently under staff review.

b. Second Ten Year Inservice Inspection Program Plan (TAC M93381)

The response to the fourth request for additional information has been received. The staff is reviewing the submittal.

c. Use of Code Case N-508-1 (TAC M97682)

The staff has completed their review of this request. The approval to use this case is expected to be issued by the end of June 1997.

d. Inservice Test Program Relief Request 2VR-7 & 8 (TAC M97883)✓

The first relief request (2VR-7) proposes to use Code Case OMN-1, "Alternative Rules for Preservice and Inservice Testing of Certain Electric Motor-Operated Valve Assemblies in Light-Water Reactor Power Plants OM Code-1995, Subsection ISTC," for testing of motor operated valves. The second relief request (2VR-8) proposes to use ASME OMa-1996, Subsection ISTC 4.5 including the Mandatory Appendix II, "Check Valve Condition Monitoring Program," as an alternative to the current check valve test program. The staff is currently reviewing this application. A supplement will be required to provide additional information that was requested in order for the staff to complete its review.

e. Inservice Test Program Relief Request 2VR-9 (TAC M98506)✓

This request proposes an alternative test method for a check valve that is a containment isolation valve. The leak test and operational test would be done at 5-year intervals. The staff is reviewing the request.

f. Inservice Inspection Program Relief Request I2R-22 (TAC M98507)✓

This request proposes to use a later edition of the code for the inspection of buried pipe. The staff has requested additional information about the acceptable leak rates and inspection techniques available to determine the leak location.

g. Inservice Inspection Program Relief Requests I1R-46 through I1R-49 and I2R-21 (TAC M98809)✓

These relief requests provide the results of previous examination activities during Refuel VI and Refuel VII, with supplemental examinations completed during Refuel VIII. The staff has requested additional information from the licensee.

3. The licensee has identified the following items to be submitted for approval:

a. The following submittals were identified by the licensee with approval desired before the end of the next refueling outage (October 20, 1997):

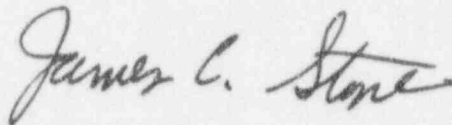
- Elimination of pressure sensor time response testing requirements as detailed in WCAP-13632-P-A, Revision 2.
- A revised definition of core alteration. The licensee is considering using the definition in the Improved Standard Technical Specifications.

- Revise Action 5.a. of Table 3.3-1 to clearly define actions to be taken on loss of a source range channel during Mode 3, 4, or 5.

The licensee was reminded that the sooner the applications were submitted, the more likely approval could be obtained by the requested date.

- b. Clarify the action statements for auxiliary feedwater system depending on the availability of different combinations of essential service water supplies to the auxiliary feedwater pumps and the availability of different steam supplies to the turbine driven pump.
- c. Change the titles of the operating crews.
- d. Replace the corrective action request (CAR) program with the performance improvement request in the QA program.
- e. Revision to the emergency plan to convert to the administrative procedure format.

Personnel in attendance at the meeting were James Stone, NRC, Terry Damashek, WCNO, and David Claridge, WCNO.



James C. Stone, Senior Project Manager
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Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-482

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Original Signed By

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