

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) PILGRIM NUCLEAR POWER STATION										DOCKET NUMBER (2) 0 5 0 0 0 2 9 3										PAGE (3) 1 OF 0 4		
TITLE (4) Missed Surveillance for "B" Diesel Generator																						
EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER (S)							
											N/A				0 5 0 0 0							
0	3	0	4	8	8	0	0	9	0	0	0	4	0	1	8	8	N/A				0 5 0 0 0	
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																			
N			20.402(b)					20.405(a)					60.73(a)(2)(iv)					73.71(b)				
POWER LEVEL (10)			0 0 0					20.405(a)(1)(i)					60.73(a)(2)(v)					73.71(a)				
			20.405(a)(1)(ii)					60.73(a)(2)(vi)					OTHER (Specify in Abstract below and in Text, NRC Form 365A)									
			20.405(a)(1)(iii)					60.73(a)(2)(vii)														
			20.405(a)(1)(iv)					60.73(a)(2)(viii)(A)														
			20.405(a)(1)(v)					60.73(a)(2)(viii)(B)														
			20.405(a)(1)(vi)					60.73(a)(2)(ix)														
LICENSEE CONTACT FOR THIS LER (12)																						
NAME										TELEPHONE NUMBER												
Paul J. Hamilton - Compliance Division Manager										AREA CODE 6 1 7 7 4 7 - 8 2 9 3												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																						
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC			
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)												
YES / / / yes, complete EXPECTED SUBMISSION DATE: X NO										MONTH DAY YEAR												

ABSTRACT // Limit to 1400 spaces (i.e., approximately fifteen single-space typewritten lines) (1A)

O: March 4, 1988 at approximately 1600 hours a Boston Edison Quality Assurance Audit determined that documentation of a surveillance test could not be located.

Technical Specification 4.9.A.1.a requires that the emergency diesel generators, Electrical Power System (EPS), be started and loaded once/cycle with the control circuits isolated from the cable spreading room. Contrary to the above, documentation retrieved from the Document Control Center could not confirm that the "B" Emergency Diesel Generator had been started and loaded with control circuits isolated from the cable spreading room during the previous refueling cycle (December 10, 1983 to December 24, 1984).

Deficiency Report (DR) 1763 and Failure and Malfunction Report (F&MR) 88-64 were written on March 4, 1988 to document this deficiency.

This deficiency was identified during an extended outage while in cold shutdown. The reactor mode selector switch was in the SHUTDOWN position, the control rods inserted, and the reactor water temperature approximately 90°F with negligible core decay heat.

Based on the redundancy of systems and procedures available, and the likely functional status of the circuitry in question this deficiency posed no threat to the health and safety of the public. This deficiency was determined to be reportable pursuant to 10CFR50.73(a)(2)(1)(B).

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S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31-88

FACILITY NAME (1): Pilgrim Nuclear Power Station	DOCKET NUMBER (2): 0 5 0 0 0 2 9 3	LER NUMBER (5):			PAGE (3):		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		88	009	00	02	OF	04

TEXT (if more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION

On March 4, 1988 at 1600 hours while BECo Quality Assurance was performing Audit 88-4, "Review of Past Surveillance Test", documentation for completion of surveillance Procedure 8.9.13, "Diesel Test from Alternate Shutdown Panel", could not be located for the "B" Diesel Generator.

Technical Specification 4.9.A.1.a and implementing Procedure 8.9.13 require that the emergency diesel generators, Electric Power System (EPS), be started and loaded once/cycle with the control circuits isolated from the cable spreading room. Contrary to the above, documentation retrieved from the Document Control Center could not confirm that the "B" Emergency Diesel Generator had been started and loaded with control circuits isolated from the cable spreading room during the previous refueling cycle (December 10, 1983 to December 24, 1984).

Available documentation proves testing of the "A" Diesel Generator per Procedure 8.9.13 was performed on December 22, 1984. Based upon discussion with personnel, Operations firmly believes that the 8.9.13 test for "B" Diesel was performed between December 15, 1984 and December 17, 1984.

Deficiency Report (DR) 1763 and Failure and Malfunction Report (F&MR) 88-64 were written on March 4, 1988 to document this deficiency. Notification of the deficiency was made to the NRC Operations center on March 4, 1988 at 1645 hours.

This deficiency was identified during an extended outage while in cold shutdown with the reactor mode selector switch in the SHUTDOWN position, the control rods inserted, and the reactor water temperature approximately 90 degrees Fahrenheit with negligible core decay heat.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED LER NO. 3150-0104
EXPIRES 3/31/88

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NOTE: If more space is required, use additional NRC Form 365A (1) (17)

CAUSE

A search of the Document Control Center records, Master Surveillance Tracking Program (MSTP) records and the Nuclear Watch Engineer (NWE) log has provided no documented evidence of having performed Procedure 8.9.13 for "B" D/G. Therefore either:

- a) The test was not performed and constitutes a missed surveillance,
or
- b) The test was performed but not documented
or
- c) The test was performed and documentation misplaced.

An exact cause of the deficiency could not be determined. The facts cannot be traced because of the lack of specific log entries. A contributing cause may be that the MSTP schedule did not distinguish between D/G "A" and "B". This lack of distinction may have inadvertently allowed both D/G "A" and "B" to be statused complete when only D/G "A" had been tested.

CORRECTIVE ACTION

The circuits which provide isolation of the diesel generator controls from the Cable Spreading Room were redesigned and modified in 1986. Temporary Procedure (TP) 86-159 tested the modification on January 17, 1987. This test, although not an exact duplicate of the 8.9.13 surveillance, demonstrated the adequacy of the circuitry in question. Test 8.9.13 has not yet been performed for this outage, but, is due prior to startup.

PREVENTIVE ACTION TO PRECLUDE RECURRENCE

The following actions address the probable cause(s) of this deficiency:

- * Log keeping - The Nuclear Operating Supervisor is required to log the initiation and completion of tests. This practice was re-emphasized in 1987. Procedure 1.3.34 requires surveillance test logging.
- " MSTP - Procedure 8.9.13 has been split on the MSTP to track the testing of the A and B Diesel Generators individually. MSTP already tracks other major ECCS pumps and valves individually.

The completed actions as outlined above are believed adequate to preclude recurrence.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 6/31/88

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TEXT (if more space is required, use additional NRC Form 306A's) (17)

SAFETY CONSEQUENCES

Based on the redundancy of systems and procedures available and functional status of the circuitry in question, this deficiency posed no threat to the health and safety of the public.

This deficiency was determined to be reportable pursuant to 10CFR50.73(a)(2)(i)(B).

SIMILARITY OF PREVIOUS EVENTS

A review was conducted of Pilgrim Station Licensee Event Reports (LER's) written since January 1984. The review focused on LER's submitted pursuant to 10CFR50.73(a)(2)(i)(B) involving missed surveillances. There were six similar events (LER 84-003, 85-016, 85-024, 85-026, 85-033, and 86-022) reported during this time.

ENERGY INDUSTRY IDENTIFICATION SYSTEM (EIIS)

The EIIS codes for this report are as follows:

COMPONENTS

Diesel Generator ('B')

CODES

DG

SYSTEMS

Emergency Onsite Power Supply System (EPS)

CK

BOSTON EDISON

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

Ralph G. Bird

Senior Vice President — Nuclear

April 1, 1988
BECo Ltr. #88-064


U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Docket No. 50-293
License No. DPR-35

Dear Sir:

The attached Licensee Event Report (LER) 88-009-00 "Missed Surveillance for
"B" Diesel Generator" is submitted in accordance with 10CFR Part 50.73.

Please do not hesitate to contact me if you have any questions regarding this
report.


R.G. Bird

RW/b1

Enclosure: LER 88-009-00

cc: Mr. William Russell
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Rd.
King of Prussia, PA 19406

Sr. Resident Inspector - Pilgrim Station

Standard BECo LER Distribution

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