

NOTICE OF VIOLATION

United States Enrichment Corporation
Portsmouth Gaseous Diffusion Plant

Docket No. 70-7002
Certificate No. GDP-2

During an NRC inspection conducted from April 7, 1997, to May 18, 1997, four violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," NUREG-1600, the violations are listed below:

1. 10 CFR 76.93, "Quality Assurance," requires that the Corporation shall establish, maintain, and execute a quality assurance program satisfying each of the applicable requirements of American Society of Mechanical Engineers (ASME) NQA-1-1989, "Quality Assurance Program Requirements for Nuclear Facilities."

American Society of Mechanical Engineers (ASME) NQA-1-1989, "Quality Assurance Program Requirements for Nuclear Facilities," Basic Requirement 16 "Corrective Action" states that conditions adverse to quality shall be identified promptly and corrected as soon as practical. In the case of a significant condition adverse to quality, the cause of the condition shall be determined and corrective action taken to preclude recurrence.

Contrary to the above, a significant condition adverse to quality was identified in April 1997 in that the corrective actions taken did not prevent the in-line strainer for Autoclave No. 1 in the X-344 building from clogging. As a result a safety system actuation occurred on May 13, 1997.

This is a Severity Level IV violation (Supplement VI) (70-7002/97003-02).

2. Technical Safety Requirement 2.2.3.15 requires, in part, that within eight hours after entering MODE VI (shutdown), a system which contains an uranyl fluoride (UO_2F_2) deposit greater than safe mass, not in a fluorination environment and not undergoing chemical treatment shall be pressurized with nitrogen (N_2)/air to greater than or equal to 14 psia.

Contrary to the above, between April 11 and 12, 1997, with cell 29-2-2 in MODE VI, facility staff did not restore plant N_2 /air to greater than 14 psia on an UO_2F_2 deposit located on the cell bypass line following an X-joint piping replacement.

This is a Severity Level IV violation (Supplement VI) (70-7002/97003-07).

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3. Technical Safety Requirement 3.11.2 requires, in part, that all operations involving uranium enriched to 1.0 weight-percent (wt%) or higher U-235 and 15 grams (g) or more of U-235 shall be performed in accordance with a documented nuclear criticality safety approval.

Contrary to the above, on May 1, 1997, a cascade converter was surveyed and found with deposits of uranium enriched to 1.0 wt% U-235 and 15 g or more of U-235 and the storage was not performed in accordance with a nuclear criticality safety approval.

This is a Severity Level IV violation (Supplement VI) (70-7002/97003-08).

4. 10 CFR 76.60(g) requires that the Corporation comply with the applicable provisions of 10 CFR Part 71.

10 CFR 71.5(a) requires, in part, that the Corporation shall comply with Department of Transportation requirements in 49 CFR 170 through 189 for shipments of licensed material outside of plant boundaries.

10 CFR 71.12 states that a general license is issued to any licensee to transport, or deliver to a carrier for transport, licensed material in a package for which a license, certificate of compliance, or other approval is issued by the NRC. 10 CFR 71.12 requires that the licensee comply with the terms and conditions of the certificate.

Condition 11 of Certificate of Compliance No. 4909, Revision 14, for a Model GE-21PF-1 overpack, requires that each 30B cylinder be inspected, tested, maintained, assembled, and used in accordance with American National Standards Institute (ANSI) N14.1-1990.

Section 6.12.6 of ANSI N14.1-1990 required that the valve and plug threads for 30B cylinders be tinned with a thin uniform coating of ASTM B32 alloy 50A solder.

Contrary to the above, on April 10, 1997, the Corporation delivered two 30B cylinders to carriers for transport but failed to assemble and maintain the cylinders in accordance with ANSI N14.1. Specifically, tinning for the valve and plug threads was not made of alloy ASTM B32 50A solder.

This is a Severity Level IV violation (Supplement VI) (70-7002/97003-11).

Pursuant to the provisions of 10 CFR 2.201, United States Enrichment Corporation is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the Regional Administrator, Region III, and a copy to the NRC Resident Inspector at the Portsmouth Gaseous Diffusion Plant of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved.

Your response may reference or include previously docketed correspondence, if the correspondence adequately addresses the required response. If an adequate reply is not received within the time specified in this Notice, an order or a Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. If personal privacy or proprietary information is necessary to provide an acceptable response, then please provide a bracketed copy of your response that identifies the information that should be protected and a redacted copy of your response that deletes such information. If you request withholding of such material, you must specifically identify the portions of your response that you seek to have withheld and provide in detail the bases for your claim of withholding (e.g., explain why the disclosure of information will create an unwarranted invasion of personal privacy or provide the information required by 10 CFR 2.790(b) to support a request for withholding confidential commercial or financial information). If safeguards information is necessary to provide an acceptable response, please provide the level of protection described in 10 CFR 73.21.

Dated at Lisle, Illinois
this 20th day of June 1997