

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Millstone Nuclear Power Station Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 4 2 3										PAGE (3) 1 CF 0 4	
TITLE (4) Missed Containment Leakage Detection System Surveillances Due to Defective Procedure Due to Personnel Error																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)						
0 2	2 2	8 8	8 8	0 1 1	0 0	0 3	2 3	8 8							0 5 0 0 0						
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																			
1		20.402(b)				20.405(a)				50.73(a)(2)(ix)				73.71(b)							
POWER LEVEL (10)		20.405(a)(1)(i)				50.38(a)(1)				50.73(a)(2)(ix)				73.71(a)							
0 9 9		20.405(a)(1)(ii)				50.38(a)(2)				50.73(a)(2)(ix)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
		20.405(a)(1)(iii)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(A)											
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)											
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																					
NAME Vere R. Joseph, Associate Engineer X 5571										TELEPHONE NUMBER AREA CODE 2 0 3 4 4 7 - 1 7 9 1											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS											
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH		DAY		YEAR					
YES (If yes, complete EXPECTED SUBMISSION DATE)										X NO											

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On February 22, 1988 at 0330 hours while at 99% power (Mode 1), 585 degrees F and 2250 psia, a control room operator discovered that the surveillance frequency for the Containment Atmosphere Gas and Particulate Radiation Monitors had been changed from once per shift to daily on the "Daily and Shiftly Control Room Rounds". On February 29, 1988 at 0130 hours while at 99% power (Mode 1), 585 degrees F and 2250 psia, a similar incident was identified for the Containment Drain Sump and Level Monitoring System, during a review of the Technical Specifications frequency requirements versus the frequencies specified in the revised surveillance.

The cause of the incident was a defective surveillance. The incorrect surveillance frequencies were erroneously transferred during the revision process of the surveillance. In both instances, a temporary log was immediately initiated to conform to the Technical Specifications.

A comparison of the old and new revisions of the surveillance, as well as other surveillances which underwent major revisions did not identify any other problems. A memo has been distributed to unit supervisory personnel emphasizing the need for detailed reviews when revising procedures. Operations Department administrative procedures have been revised to require independent review of procedure revisions.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/86

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		YEAR 8 8	SEQUENTIAL NUMBER 0 1 1	REVISION NUMBER 0 0	0 2	OF 0 4	

TEXT (if more space is required, use additional NRC Form 366A's) (17)

I. Description of Event

On February 22, 1988 at 0330 hours while at 99% power (Mode 1), 585 degrees F and 2250 psia, a control room operator observed that the (daily) surveillance frequency specified in the latest revision of the "Daily and Shiftly Control Room Rounds", SP 3670.1-1, for the Containment Atmosphere Gas and Particulate Radiation Monitors, 3CMS*RE22A and B was less conservative than the (at least once per 12 hours) surveillance frequency established by the plant's Technical Specifications for the same radiation monitors. The observation was made while reviewing the Technical Specifications requirements for a plant component unrelated to the incident.

The control room operator immediately informed the Shift Supervisor of the procedural deficiency and a temporary log was initiated to comply with the Technical Specifications surveillance frequency requirements for 3CMS*RE22A and B.

After further investigation of the incident, it was discovered that the surveillance requirement outlined in the Technical Specifications was not satisfied since January 15, 1988 when SP 3670.1-1 underwent a major revision.

On February 29, 1988 at 0130 hours, while at 99% power (Mode 1), 585 degrees F and 2250 psia, another control room operator discovered that the (daily) surveillance frequency specified in SP 3670.1-1 for the Containment Drain Sump Level and Pumped Capacity Monitoring System did not fulfill the (at least once per 12 hours) surveillance frequency requirements established by the Technical Specifications. The discovery was made while reviewing the Technical Specifications surveillance frequency requirements versus the frequency specified in the revised version of SP 3670.1-1. The Shift Supervisor was immediately notified and a temporary log, in compliance with the Technical Specifications frequency requirements, was initiated.

II. Cause of Event

The cause of the incidents was a defective surveillance. The cause of the deficient surveillance was personnel error. The surveillance frequencies specified in the old revision of SP 3670.1-1 had been incorrectly transferred to the new revision of the surveillance. The revised version of SP 3670.1-1 was initially checked for content against the previous version of the surveillance. However, this review failed to identify the error in the surveillance frequencies.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

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Millstone Nuclear Power Station Unit 3	05000423	88	011	00	03	OF	04

TEXT (If more space is required, use additional NRC Form 365A's) (17)

III. Analysis of Event

These events are being reported pursuant to the requirements of 10CFR50.73 (a)(2)(i), in that conditions existed which were prohibited by the plant's Technical Specifications.

Plant Technical Specifications require that: a) the Containment Atmosphere Gas Radiation Monitor, b) the Containment Atmosphere Particulate Radiation Monitors and c) the Containment Drain Sump Level or Pumped Capacity Monitoring System be monitored at least once every 12 hours as part of the Reactor Coolant System (RCS) Leakage Detection System. As limiting conditions of operation at least two of these Leakage Detection Systems are required to be operable and unidentified RCS leakage is limited to 1 gpm. The basis for these requirements is that the RCS is designed to leak before it breaks. Early detection of small scale leakage increases the ability of plant personnel and systems to address and mitigate leakage which could potentially escalate into a loss of coolant accident.

From the issuance of SP 3670.1-1 (on January 15, 1988) until the first incident date, all three RCS Leakage Detection Systems were not monitored in accordance with the frequency requirements established by Technical Specifications. However alarm indication of possible unidentified leakage for the Containment Atmosphere Gas and Particulate Radiation Monitors, 3CMS*RE22A and B was available as well as other viable means of leakage detection. A review of the leakage trends for this period showed no indication of unidentified leakage in excess of the Technical Specifications requirement. Any significant unidentified leakage during this time period would have been detected by a higher than normal RCS makeup rate to the Chemical and Volume Control System's Volume Control Tank (VCT). The VCT is continuously trended during normal operations and is sensitive to changes in RCS inventory.

No adverse safety consequences to the plant and to the health and safety of the public occurred as a result of the incident.

IV. Corrective Action

Immediate corrective action for both incidents was to initiate a temporary log to conform to the requirements of the Technical Specifications. A line by line review of the "Daily and Shiftly Control Room Rounds", as well as all other surveillances which underwent major revisions, was performed. No other procedural shortcomings were identified.

A memo has been distributed to all supervisory personnel emphasizing the need for detailed reviews when performing procedure revisions. Operations Department administrative procedures have been revised to require independent review of surveillance revisions and changes against the original to ensure all changes are intentional.

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TEXT (If more space is required, use additional NRC Form 385A's) (17)

V. Additional Information

The event described in LER 87-016-00 is similar in that a defective procedure, due to an inadvertent omission made during the revision process, caused the incident.

EIIS CodesSystemComponents

Leak Monitoring System - LJ
Radiation Monitoring System - IL

Monitor - MON

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

General Offices • Seligen Street, Berlin, Connecticut

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March 23, 1988
MP-11653
Re: 10CFR50.73(a)(2)(i)

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. NPF-49
Docket No. 50-423
Licensee Event Report 88-011-00

Gentlemen:

This letter forwards Licensee Event Report 88-011-00 required to be submitted within thirty (30) days pursuant to 10CFR50.73(a)(2)(i), any operation or condition prohibited by the Plant's Technical Specifications.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

Stephen E. Scace
Stephen E. Scace
Station Superintendent
Millstone Nuclear Power Station

SES/VRJ:mo

Attachment: LER 88-011-00

cc: W. T. Russell, Region I
W. J. Raymond, Senior Resident Inspector

*Cert No.
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