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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 12, 1997

Dr. Robert L. Seale, Chairman
Advisory Committee on Reactor Safeguards
U.S. Nuclear Regulatory Commission

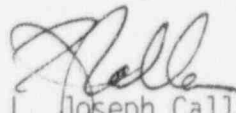
Dear Dr. Seale:

I am responding to your letter of May 7, 1997, to Chairman Jackson in which you presented the conclusions and recommendations resulting from your meeting with the NRC staff and the Nuclear Energy Institute to discuss design basis verification during the 441st meeting of the Advisory Committee on Reactor Safeguards, which was held on May 1-3, 1997. In your letter, you noted that the NRC will be relying on the results of probabilistic risk assessments (PRAs) to an ever increasing extent and that PRAs should be based on the current configuration of the plant. Therefore, you recommended that the results of the design-basis inspections currently being performed by the Office of Nuclear Reactor Regulation (NRR) should be shared with the Office of Nuclear Regulatory Research (RES) and the Office for Analysis and Evaluation of Operational Data (AEOD).

Regarding the design-basis inspection results, NRR is preparing periodic assessments of the Architect/Engineer Design Inspection Program. The purpose of the assessments is (1) to provide NRR management with a general overview of the results of the program, (2) to provide recommended actions and followup activities, and (3) to provide for trending and integration of specific inspection findings. NRR is also developing a database to help with the trending and integration of the individual inspection findings.

NRR intends to transmit the periodic assessments and the design inspection reports to AEOD and to RES for use in their reactor oversight activities and the evaluation of PRA assumptions, as appropriate.

Sincerely,


L. Joseph Callan
Executive Director
for Operations

cc: Chairman Jackson
Commissioner Rogers
Commissioner Dicus
Commissioner Diaz
Commissioner McGaffigan
SECY

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PDR ORG NE ED
PDR

OSM-7 AURS
X OSM-6 meeting

June 12, 1997

Dr. Robert L. Seale, Chairman
Advisory Committee on Reactor Safeguards
U.S. Nuclear Regulatory Commission

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Original Signed by
L. J. Callan
L. Joseph Callan
Executive Director
for Operations

cc: Chairman Jackson
Commissioner Rogers
Commissioner Dicus
Commissioner Diaz
Commissioner McGaffigan
SECY

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NAME	RGallo*	FGillespie	MSlosson	SCollins	LCallan
DATE	06/03/97	06/11/97	06/11/97	06/11/97	06/11/97

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P. Norry

J. Blaha

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R. Zimmerman

W. Travers

T. Martin

F. Gillespie

M. Slosson

S. Weiss

D. Matthews

F. Akstulewicz

J. Birmingham

R. Gallo

M. Thadani

M. Boyle, (e-mail MLB4)

S. Burns, OGC

K. Cyr, OGC

C. Paperiello, NMSS

D. Morrison, RES (w/incoming)

D. Ross, AEOD (w/incoming)

J. Mitchell, OEDO

OPA

OCA

ACRS File

B. Sweeney

Dr. Robert. L. Seale, Chairman
Advisory Committee on Reactor Safeguards
U. S. Nuclear Regulatory Commission

SUBJECT: DESIGN BASIS VERIFICATION

Dear Dr. Seale:

I am responding to your letter of May 7, 1997, to Chairman Jackson in which you presented the conclusions and recommendations resulting from your meeting with the NRC staff and the Nuclear Energy Institute to discuss design basis verification during the 441st meeting of the Advisory Committee on Reactor Safeguards, which was held on May 1-3, 1997. In your letter, you noted that the NRC will be relying on the results of probabilistic risk assessments (PRAs) to an ever increasing extent and that PRAs should be based on the current configuration of the plant. Therefore, you recommended that the results of the design-basis inspections currently being performed by the Office of Nuclear Reactor Regulation (NRR) should be shared with the Office of Nuclear Regulatory Research (RES) and the Office for Analysis and Evaluation of Operational Data (AEOD).

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Sincerely,

L. Joseph Callan
Executive Director
for Operations

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Commissioner Rogers
Commissioner Dicus
Commissioner Diaz
Commissioner McGaffigan
SECY

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Dr. Robert. L. Seale, Chairman
Advisory Committee on Reactor Safeguards
U. S. Nuclear Regulatory Commission

SUBJECT: DESIGN BASIS VERIFICATION

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Regarding the design-basis inspection results, you should know that NRR is issuing quarterly assessments of the Architect/Engineer Design Inspection Program. The purpose of the assessments is (1) to provide NRC management with a general overview of the results of the program, (2) to provide recommended actions and followup activities, and (3) to provide for trending and integration of specific inspection findings. These assessments include the findings of the inspections, a discussion of PRA, and recommended agency actions. NRR is also developing a database to help with the trending and integration of the individual inspection findings.

NRR intends to formally transmit ~~the~~ quarterly assessments to AEOD and to RES for use in their reactor oversight activities and the evaluation of PRA assumptions, as appropriate.

Sincerely,

L. Joseph Callan
Executive Director
for Operations

cc: Chairman Jackson
Commissioner Rogers
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SECY

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Dr. Robert. L. Seale, Chairman
Advisory Committee on Reactor Safeguards
U. S. Nuclear Regulatory Commission

SUBJECT: RECOMMENDATION TO SHARE RESULTS OF DESIGN-BASIS INSPECTIONS WITH
OTHER NRC OFFICES

Dear Dr. Seale:

I am responding to your letter of May 7, 1997, to Chairman Jackson in which you presented the conclusions and recommendations resulting from the 441st meeting of the Advisory Committee on Reactor Safeguards, which was held on May 1-3, 1997. In your letter, you noted that the NRC will be relying on the results of probabilistic risk assessments (PRAs) to an ever increasing extent and that PRAs should be based on the current configuration of the plant. Therefore, you recommended that the results of the design-basis inspections currently being performed by the Office of Nuclear Reactor Regulation (NRR) should be shared with the Office of Nuclear Regulatory Research (RES) and the Office for Analysis and Evaluation of Operational Data (AEOD).

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NRR intends to formally transmit the quarterly assessments to AEOD and to RES for use in their reactor oversight activities and the evaluation of PRA assumptions, as appropriate.

Sincerely,

L. Joseph Callan
Executive Director
for Operations

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Commissioner Rogers
Commissioner Dicus
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NAME	MSlosson	SCollins	LCallan	
DATE	/ /97	/ /97	/ /97	

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Matthe
ACTION

EDO Principal Correspondence Control

FROM: DUE: 06/09/97

EDO CONTROL: G970341
DOC DT: 05/07/97
FINAL REPLY:

R. L. Seale
ACRS

TO:

Chairman Jackson

FOR SIGNATURE OF : ** GRN **

EDO

DESC:

DESIGN BASIS VERIFICATION

DATE: 05/09/97

ASSIGNED TO:

NRR

CONTACT:

Collins

CRC NO:

ROUTING:

Callan
Jordan
Thompson
Norry
Blaha
Burns
Paperiello, NMSS
Morrison, RES
Ross, AEOD
Mitchell, OEDO
ACRS File

SPECIAL INSTRUCTIONS OR REMARKS:

Prepare response to ACRS for EDO signature. Add Commissioners and SECY as cc's (shown on original for reply).

NRR RECEIVED: MAY 9, 1997
NRR ACTION: DRPM:GLOSSON

NRR ROUTING: COLLINS
MIRAGLIA
MARTIN
ZIMMERMAN
TRAVERS
BOHRER

ACTION
DUE TO THE SIGNED DATE
BY June 4, '97



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

May 7, 1997

The Honorable Shirley Ann Jackson
Chairman
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555-0001

Dear Chairman Jackson:

SUBJECT: DESIGN BASIS / VERIFICATION

During the 441st meeting of the Advisory Committee on Reactor Safeguards, May 1-3, 1997, we met with representatives of the NRC staff and the Nuclear Energy Institute to discuss design basis verification. We reviewed the staff criteria for evaluating licensee responses to the NRC request for design basis information pursuant to 10 CFR 50.54(f), staff initiatives to perform design inspections, and related industry activities and initiatives. We also had the benefit of the documents referenced.

Conclusions and Recommendations

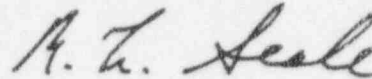
- We believe that the current four-phase approach is effective in identifying those licensees that need to take action to maintain their design basis. The staff review of licensee responses appears to have been successful in identifying and prioritizing follow-up inspection activities.
- The design inspections conducted to date have shown that, in some cases, the actual plant configuration and procedures do not correspond to the design basis upon which the plant was licensed. This illustrates the value of the design inspection program and suggests that such a program be continued in some form.
- The NRC will be relying on the results of probabilistic risk assessments (PRAs) to an ever-increasing extent as it embarks on risk-informed, performance-based regulation. PRAs should be based on the current configuration of the plant. Results of the design basis inspections should, then, be shared formally with the Office of Nuclear Regulatory Research and the Office for Analysis and Evaluation of Operational Data. Where inconsistencies between PRA assumptions and design inspection results are found, it may be of use to conduct

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sensitivity studies to establish the risk significance of these inconsistencies.

Sincerely,



R. L. Seale
Chairman

References:

1. Section (f) of 10 CFR 50.54, "Conditions of Licenses."
2. Memorandum dated March 19, 1997, from L. J. Callan, Executive Director for Operations, NRC, to the Commissioners, Subject: Update on 10 CFR 50.54(f) Response Review Efforts: Pilot Process Results.
3. Memorandum dated February 25, 1997, from L. J. Callan, Executive Director for Operations, NRC, to the Commissioners, Subject: Review of Licensees' Responses to the 10 CFR 50.54(f) Letter of October 9, 1996, on the Adequacy and Accuracy of Design Bases Information for Nuclear Power Plants.
4. Nuclear Energy Institute, NEI 96-05, draft Revision D, "Guidelines for Assessing Programs for Maintaining the Licensing Basis," dated July 25, 1996.
5. Nuclear Management and Resources Council, Inc., NUMARC 90-12, "Design Basis Program Guidelines," October 1990.