

APR 14 1972

Docket No. 50-29

Yankee Atomic Electric Company
ATTN: Mr. Donald E. Vandenburg
Vice President
20 Turnpike Road
Westboro, Massachusetts 01581

Gentlemen:

This refers to your Proposed Change No. 96 dated August 6, 1971, including four supplements dated February 4, March 10, 20 and 30, 1972, and your "Revised Loss of Coolant Analysis" dated January 15, 1972. These documents have been submitted in support of your request for certain changes to the Technical Specifications appended to License No. DPR-3 for the Yankee reactor to allow: (1) loading and operation of the reactor with zircaloy clad fuel in the Core X configuration at power levels that are authorized presently, (2) modifications to the Emergency Core Cooling System to upgrade its performance capabilities, and (3) modifications to the reactor protection system, involving the reactor coolant flow and the steam generator low level safety channels. We have designated our action on the proposed changes as Change No. 97.

We have reviewed your Proposed Change No. 96 and supplemental documents that include the information on the design, fabrication and testing of the proposed modified facility features, the safety considerations you have evaluated, the proposed changes to the Technical Specifications and the supporting revised sections of the Safety Analysis Report. We have concluded that: (a) the safety related characteristics of Core X are not significantly different from those of previous cores, (b) the proposed modifications to the Emergency Core Cooling System provide a significant improvement in the capability to cool the core in the event of a loss-of-coolant accident, (c) the performance of the upgraded Emergency Core Cooling System has been analyzed in accordance with the prescribed evaluation models and meets the criteria for Emergency Core Cooling System performance in the Commission's "Interim Policy Statement", dated June 29, 1971, and (d) the proposed modifications to the reactor protection system safety channels will enhance the safety of reactor operation.

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License No. DPR-3

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In our evaluation of Proposed Change No. 96, we also have taken into account your planned actions for improving the Emergency Core Cooling System initiation instrumentation by providing diversity of initiation signals, and for the installation of means for mixing, sampling and control of hydrogen gas that may accumulate inside the vapor container in the event of a loss-of-coolant accident. These installations will be completed at the end of the outage for refueling with Core XI.

Based on our review of the information presented in Proposed Change No. 96 and supplements, we have concluded that there are no hazards considerations not described or implicit in the Safety Analysis Report. There is reasonable assurance that the health and safety of the public will not be endangered by the proposed facility modifications, refueling with Core X and operation of the reactor up to presently authorized power levels.

Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, Change No. 97 is hereby authorized as proposed. The Technical Specifications in Appendix A to License No. DPR-3 are changed as set forth in Attachment A to this letter.

Sincerely,

/s/

Donald J. Skovholt
Assistant Director
for Reactor Operations
Division of Reactor Licensing

Enclosure:

Attachment A - Change to the
Technical Specifications

cc: C. Duane Blinn, Esquire
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Counselors at Law
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