

YANKEE ATOMIC ELECTRIC COMPANY

20 Turnpike Road Westborough, Massachusetts 01581

January 26, 1978

United States Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Office of Nuclear Reactor Regulation
Paul F. Collins, Chief
Operator Licensing Branch
Division of Project Management

Reference: (a) License No. DPR-3 (Docket No. 50-29)
(b) Letter, D. E. Moody to USNRC, dated
October 17, 1977, WYR 77-97
(c) Letter, USNRC to D. E. Moody, dated
December 23, 1977.

Dear Sir:

Subject: Yankee Rowe Operator Training Program

In Reference (b) Yankee Atomic Electric Company submitted their Operator Training Program for the Yankee Rowe plant. Additional information needed for the review of that program was requested by the NRC in Reference (c). Attached you will find, as Attachment A, the answers to the questions outlined in Reference (c). Also attached is Revision 4 to Procedure AP-0500, Yankee Rowe Operator Training Program which has been revised to include the information which appears in Attachment A to this letter.

We trust that you will find this information satisfactory; however, should you desire additional information please contact us.

Very truly yours,

YANKEE ATOMIC ELECTRIC COMPANY

Robert H. Groce

R. H. Groce
Licensing Engineer

JKT/kg
Attachment

8011070075

ATTACHMENT A

ADDITIONAL INFORMATION

QUESTION II.A.4.

The comments and evaluations contained on the completed Operator Evaluation Form plus the results of the Annual Evaluation Examination shall form the basis for an accelerated requalification program when needed.

RESPONSE II.A.4. Changed to Read

If a licensed operator for senior operator fails to achieve an overall examination grade of 70%, on either the annual examination or the walk through evaluations, he will be required to participate in an accelerated requalification program prior to resuming licensed activities.

QUESTION II.C.1.

Facility design changes, procedure changes, and facility license changes must be explicitly indicated as part of the required review.

RESPONSE II.C.1. Changed to Read

All licensed operators and senior operators will review facility design changes, applicable procedure changes and facility license changes.

QUESTION II.C.2.

As a minimum, all abnormal and emergency procedures shall be reviewed on an annual basis.

RESPONSE II.C.2. Changed to Read

The Training Coordinator will insure all licensed operators and senior operators review all abnormal and emergency procedures at least annually. The completed forms will be returned to the Training Coordinator for file in the individual's file.

QUESTION II.C.4.

Provide the method by which non-shift licensed personnel and Shift Supervisors are evaluated during actual or simulated abnormal and emergency plant conditions.

RESPONSE II.C.4. Changed to II.C.3. To Read

The performance of all licensed individuals will be evaluated during actual or simulated abnormal and emergency plant conditions. This will be done to meet the requirements for systematic observation and evaluation which appears in Paragraph 4.c of Appendix A of 10 CFR 55. This will be accomplished by oral examination in the case of simulated emergencies. These evaluations will be documented and placed in individual's file.

QUESTION II.D.1.

Appendix A of AP-0500 contains a listing of reactivity changes for which Yankee Rowe will take credit. The staff position is that only the following constitute acceptable reactivity changes.

1. Reactor startup to the point of adding heat.
2. Controlled reactor shutdown.
3. Manual control of steam generator levels during startups and shutdowns when power is less than 15%.
4. Operation of the turbine governor controls in manual during startups and for power changes equal to or greater than 10%.
5. Boration or dilution which changes boron concentration by at least 10 ppm.
6. Operation of the manipulator crane in the core during refueling.
7. Any power changes of 10% or more in manual rod control.
8. Manual rod control prior to and during generator synchronization.

RESPONSE II.D.1. Appendix A Changed to Read

1. Reactor startup to the point of adding heat.
2. Controlled reactor shutdown.
3. Manual control of steam generator levels during startups and shutdowns when power is less than 15%.
4. Operation of the turbine governor controls in manual during startups and for power changes equal to or greater than 10%.

5. Boration or dilution which changes boron concentration by at least 10 ppm.
6. Operation of the manipulator crane in the core during refueling.
7. Any power changes of 10% or more in manual rod control.
8. Manual rod control prior to and during generator synchronization.