

SEP 10 1976

Docket No. 50-29

Yankee Atomic Electric Company
ATTN: Mr. Robert H. Groce
Licensing Engineer
20 Turnpike Road
Westboro, Massachusetts 01581

Gentlemen:

RE: YANKEE-ROWE

General Design Criteria (GDC) 30 and 32 of Appendix A to 10 CFR Part 50 require that components that are part of the reactor coolant pressure boundary receive periodic inspection and testing of critical areas to assess their structural and leaktight integrity. The steam generator tubes constitute over 50% of the area of the total primary system pressure-retaining boundary in pressurized water reactors. Our letter dated July 18, 1974, requested you to propose suitable technical specifications for the surveillance of the steam generator tubes consistent with the requirements of the GDC and Regulatory Guide 1.83 titled "Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes." Since your proposal was submitted, R.G. 1.83 has been revised; therefore, we request that you revise your submittal to be consistent with the enclosed model technical specifications which are based on Revision 1 of R.G. 1.83, dated July 1975.

Both the original and revised R.G. 1.83 refer to a tube plugging limit and plugging criteria, but neither provides guidance on how these are to be quantified. The plugging limit in the enclosed model technical specifications is based on the minimum tube wall thickness needed to maintain tube integrity during a LOCA combined with a safe shutdown earthquake and an operational allowance to account for time intervals between inspections. We have concluded that plugging limits that are based on these considerations satisfy the requirements of the GDC and are acceptable to us.

Steam generator tube leakage can be an indication of tube degradation; therefore, we have concluded that a leakage limit and a limiting condition for operation should be included in the technical specifications to ensure timely action in the event of steam generator tube degradation. A leakage limit such as the one in the enclosed model technical specifications is acceptable to us.

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Certain steam generators have completed the first cycle of operation with tube inspection results that have indicated little discernible degradation. These early-in-life inspection results may not be indicative of the total 40 year design life of steam generator tubes in terms of tube integrity. Therefore, we have concluded that surveillance of steam generator tube integrity should be covered by technical specifications for all facilities including those where early inspection cycles have not indicated any discernible or increasing degradation. The enclosed model technical specifications gives credit for favorable early inspection results by extending subsequent inspection intervals and keeping at a minimum the number of tubes to be probed.

We request that within 45 days of receipt of this letter, you revise or resubmit your request for a license amendment using the enclosed model technical specifications for guidance. The model technical specifications should be appropriately adapted to your specific format and to the number of steam generators in your plant.

Sincerely,

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosure:
Model Technical Specifications

cc w/enclosure: See next page

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Yankee Atomic Electric Company

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September 10, 1976

cc: Mr. Donald G. Allen, President
Yankee Atomic Electric Company
20 Turnpike Road
Westboro, Massachusetts 01581

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