

OCT 27 1972

Docket No. 50-29

Yankee Atomic Electric Company  
ATTN: L. E. Minnick  
Vice President  
20 Turnpike Road  
Westboro, Massachusetts 01581

Gentlemen:

Change No. 102  
License No. DPR-3

By letter dated October 12, 1972, and supplemental information contained in your teletype dated October 25, 1972, you requested authorization to replace four removable fuel rods in one Core X fuel assembly with four new fuel rods.

The proposed change, designated Change No. 102, would permit you to reinstall a fuel assembly with four new fuel rods into Core location J-7 during the present outage for replacing control rods. The new fuel rods will be similar in all respects to the fuel rods in the fuel assembly. Thus, the nuclear characteristics of the new fuel rods approximate those of the irradiated fuel rods that will be replaced. You intend to carry out a detailed inspection on the four irradiated fuel rods (two prepressurized and two unprepressurized) as part of Yankee's continuing program of fuel performance evaluation. You will promptly report the inspection findings to the Commission.

The Technical Specifications appended to License No. DPR-3 for the Yankee reactor include a description of the zircaloy clad fuel in the present Core X. Specifically, Section 101 in the Technical Specifications allows a total of 96 removable rods in eight fuel assemblies to facilitate post-irradiation inspection.

We have reviewed the information you have submitted on your evaluation of the safety considerations associated with the replacement of four fuel rods in the fuel assembly from core location J-7. Your nuclear reanalysis has confirmed that the proposed replacement of four fuel rods would have a negligible effect on the nuclear characteristics determined previously by Yankee in the Core X analysis; hence, the margins of safety and the consequences of accidents determined previously for operating the reactor in the Core X configuration remain unchanged.

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We have concluded that the proposed change does not involve significant hazards considerations not described or implicit in the Final Safety Analysis Report and that the health and safety of the public will not be endangered. Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, Change No. 102 is hereby authorized.

Sincerely,

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Donald J. Skovholt  
Assistant Director for  
Operating Reactors  
Directorate of Licensing

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