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MAY 13 1974

Docket No. 50-29

Yankee Atomic Electric Company
ATTN: Mr. L. H. Heider
Assistant Vice President
20 Turnpike Road
Westboro, Massachusetts 01581

Gentlemen:

By letter dated March 18, 1974, you submitted changes to the Technical Specifications appended to License No. DPR-3 that would allow installation of a new incore instrumentation package for the Yankee-Rowe reactor. We have reviewed your submittal and found that additional information is required to complete our review. The needed information and schedules were discussed with Mr. C. Andognini of your staff during a discussion on the telephone on April 16, 1974.

Attachment A to this letter identifies for the record the additional information that we have requested during the recent telephone discussion. In order to maintain our review schedule, we need acceptable responses not later than May 30, 1974.

Sincerely,

Robert A. Purple, Chief
Operating Reactors Branch #1
Directorate of Licensing

Enclosure:
Attachment A - Additional
Information on Yankee-Rowe

OFFICE ▶	L:ORB #1	L:ORB #1				
SURNAME ▶	ABurger:lb	RAPurple				
DATE ▶	5/13/74	5/13/74				

Form AEC-318 (Rev. 9-53) AECM 0240 GPO C43 16 81465-1 520-284

ATTACHMENT A

ADDITIONAL INFORMATION ON YANKEE-ROWE

INSTALLATION OF A NEW INCORE INSTRUMENTATION PACKAGE

DOCKET NO. 50-29

YANKEE ATOMIC ELECTRIC COMPANY

1. Provide a description of the new incore instrumentation package in sufficient detail comparable to the description in the present pages 107:1 through 107:8, including Figure 1 included in the Technical Specifications.

2. Provide a figure or figures that will depict the important details of the flux thimble sealing system at the vessel ports comparable to details shown in the present Figures 107:4A and 107:4B.

3. Describe in sufficient detail the thimble leak detection system and how you will provide for isolation of individual thimbles.

4. Provide the basis for your conclusion that the total flow and the flow distribution in the reactor vessel will not be changed significantly as a result of the installation of the modified support structure in the upper plenum.

5. Justify the use of the conoseals and swageseals in the reactor vessel head port assemblies as ANSI N18.2 Safety Class I features.

6. Provide assurance that it is unlikely that potential flow-induced vibration of the thimbles in the upper plenum will cause failure of a thimble or a thimble swageseal.

7. Assuming a thimble were to fail and would be isolated, will the thimble up to the isolation valve be protected against potential damage from missiles to preclude uncontrolled leakage of primary coolant?