



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION III  
799 ROOSEVELT ROAD  
GLEN ELLYN, ILLINOIS 60137

MAR 17 1988

Docket No. 50-456

Docket No. 50-457

Commonwealth Edison Company  
ATTN: Mr. Cordell Reed  
Senior Vice President  
Post Office Box 767  
Chicago, IL 60690

Gentlemen:

Enclosed for your review, prior to our scheduled meeting of April 5, 1988, is the SALP 7 Board Report for the Braidwood Nuclear Plant covering the period December 1, 1986 through December 31, 1987. This period encompassed precriticality testing, initial criticality, and startup testing for Unit 1, and construction completion, precriticality testing, and initial fuel loading for Unit 2.

In accordance with NRC policy, I have reviewed the SALP Board assessment and concur with the assigned ratings. It is my view that Commonwealth Edison Company's conduct of nuclear activities in connection with the Braidwood Plant shows an appropriate concern for nuclear safety. Highlights of the report are set forth below:

1. The Braidwood Plant has earned two Category 1 ratings and eleven Category 2 ratings. These ratings reflect adequate performance in all functional areas. The Category 1 ratings were in the areas of emergency preparedness and training and qualification effectiveness. Each of these areas improved from a Category 2 rating during the previous assessment period.
2. Performance declined from Category 1 to Category 2 in two areas: quality programs and administrative controls affecting quality and preoperational and startup testing. The decline in the former reflects the overall adequacy of programs and controls affecting the quality of operational activities, as opposed to the aggressive resolution of construction issues during the previous assessment period. The decline in the latter is largely due to the increase in violations in that area.
3. A concern emphasized in this report involves the number of personnel errors compiled during this assessment period. These errors indicate that the level of alertness and attention to detail at the station can be improved. While the problems encountered to date have not greatly affected preoperational activities, they could result in more serious consequences during future operation (i.e., reactor trips, actuations of engineered safety features, and safety injections with the reactors at power). The NRC will closely follow your progress in minimizing problems of this type.

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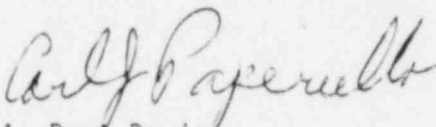
4. It is emphasized that this report assesses your performance during a transitional stage of facility activities; the assessment period covers the completion of construction, extensive preoperational testing of both units, and power ascension testing of Unit 1. You have adequately handled these activities to date, but have yet to operate the Braidwood Station through a sustained period of power generation. The challenge remains for you to make the transition from startup and licensing efforts to strong operational performance at Braidwood.

While you will have sufficient opportunity to present your comments at the meeting on April 5, 1988, we also solicit written comments within 30 days after the meeting to enable us to thoroughly evaluate your comments and to provide you with our conclusion relative to them.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and the SALP Report will be placed in the NRC's Public Document Room.

No reply to this letter is required at this time; however, should you have any questions concerning the SALP Report, we would be pleased to discuss them with you.

Sincerely,

*for*   
A. Bert Davis  
Regional Administrator

Enclosure: SALP 7 Board Reports  
No. 50-456/88001(DRP);  
No. 50-457/88001(DRP)

cc w/enclosure:

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