NORTHEAST UTILITIES
THE CONNECTICUT LIGHT AND POWER COMPANY
THE HARTFORD ELECTRIC LIGHT COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

P.O. BOX 270
HARTFORD, CONNECTICUT 06101
(203) 666-6911

October 26, 1978

Docket No. 50-336

Director of Nuclear Reactor Regulation
Attn: Mr. R. Reid, Chief
Operating Reactors Branch #4
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

References: (1) G. Lear letter to D. C. Switzer dated April 26, 1976.
(2) G. Lear letter to D. C. Switzer dated November 22, 1976.
(3) G. Lear letter to D. C. Switzer dated January 5, 1978.

Gentlemen:

Millstone Nuclear Power Station, Unit No. 2
Proposed Revisions to Technical Specifications

In Reference (1), Northeast Nuclear Energy Company (NNECO) was informed of revisions to 10CFR50.55a which require periodic revisions to Technical Specifications, concerning inservice inspection and testing programs. References (2) and (3) provided additional guidance on preparing inservice inspection program submittals. For Millstone Unit No. 2, the start of the next (second) 40-month inspection period as defined in ASME Section XI is April 26, 1979. Accordingly, the following proposed revisions to Technical Specifications are being submitted six months prior to the start of the next inspection period as required by 10CFR50.55a.

Pursuant to 10CFR50.90, NNECO hereby proposes to amend its operating license, DPR-65, by incorporating the following proposed revisions into the Millstone Unit No. 2 Technical Specifications:

On Pages 3/4 4-22, 4-23, B3/4 4-11 and 4-12, revise the requirements of the inservice inspection program of the Reactor Coolant System and related Bases as shown in Attachment 1.

These proposed changes to the inservice inspection requirements conform to the recommendations of the Staff as provided in Reference (1). The Bases have also been changed to conform to the latest revision of Combustion Engineering Standard Technical Specifications (STS).

The above proposed changes have been reviewed pursuant to 10CFR50.59, and have not been found to constitute an unreviewed safety question.

The Millstone Unit No. 2 Nuclear Review Board has reviewed and approved the above proposed changes, and concurred in the above determination.

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NNECO has reviewed the above proposed changes pursuant to the requirements of 10CFR170, and has determined that no fee is applicable in this instance. The revision to Section 4.4.10 uses language very similar to the sample Technical Specification language forwarded in Reference (1). The related revision to the Bases section conforms to the latest revision of Combustion Engineering STS. It is not expected that any further Staff review will be required to approve these changes. These proposed Technical Specifications were necessarily determined to be acceptable prior to the date of Reference (1), April 26, 1976.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

W. G. Counsil

W. G. Counsil
Vice President

Attachment

STATE OF CONNECTICUT)
) ss. Berlin Oct. 26, 1978
COUNTY OF HARTFORD)

Then personally appeared before me W. G. Counsil, who being duly sworn, did state that he is Vice President of Northeast Nuclear Energy Company, a Licensee herein, that he is authorized to execute and file the foregoing information in the name and on behalf of the Licensees herein and that the statements contained in said information are true and correct to the best of his knowledge and belief.

Sheila M. Oates
Notary Public

My Commission Expires March 31, 1981

DOCKET NO. 50-336

ATTACHMENT 1

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

PROPOSED REVISIONS TO TECHNICAL SPECIFICATIONS

OCTOBER, 1978

REACTOR COOLANT SYSTEM

STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.4.10 The structural integrity of components (except steam generator tubes) identified in Section 1.2.14 of the FSAR as Safety Class 1 components and of the steam generator secondary side circumferential shell welds shall be maintained at a level consistent with the acceptance criteria in Specification 4.4.10.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With the structural integrity of any of the above components not conforming to the above requirements and $T_{avg} > 200°F$, either immediately isolate the affected component or be in COLD SHUTDOWN within the next 36 hours.

b. With the structural integrity of any of the above components not conforming to the above requirements and the unit in COLD SHUTDOWN, restore the structural integrity of the affected component to within its limits prior to increasing the Reactor Coolant System temperature above the minimum temperature required by NDT considerations.

SURVEILLANCE REQUIREMENTS

4.4.10 The following inspection program shall be performed:

a. Inservice inspection of ASME Code Class 1, Class 2, and Class 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55(a)(g), except where specific written relief has been granted by the NRC pursuant to 10CFR50, Section 50.55a(g)(6)(i). These exceptions and alternate examinations are included in the Inservice Inspection Program.

SURVEILLANCE REQUIREMENTS (Continued)

b. Inservice testing of ASME Code Class 1, Class 2, and Class 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10CFR50 Section 50.55a(g)(6)(i). These exceptions and alternate examinations are included in the Inservice Inspection Program.

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for piping, pumps and valves. Below this temperature, the system pressure must be limited to a maximum of 20% of the system's hydrostatic test pressure of 3125 psia.

The number of reactor vessel irradiation surveillance specimens and the frequencies for removing and testing these specimens are provided in Table 4.4-3 to assure compliance with the requirements of Appendix H to 10 CFR Part 50.

The limitations imposed on the pressurizer heatup and cooldown rates and spray water temperature differential are provided to assure that the pressurizer is operated within the design criteria assumed for the fatigue analysis performed in accordance with the ASME Code requirements.

3/4.4.10 STRUCTURAL INTEGRITY

The inspection programs for the ASME Code Class 1, 2, and 3 components ensure that the structural integrity of these components will be maintained at an acceptable level throughout the life of the plant. To the extent applicable, the inspection program for these components is in compliance with Section XI of the ASME Boiler and Pressure Vessel Code.

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3/4.4.11 CORE BARREL MOVEMENT

This specification is provided to ensure early detection of excessive core barrel movement if it should occur. Neutron noise levels are used to continually monitor core support barrel (CSB) motion. Change in motion is manifested as changes in the four excore neutron detector signals. Baseline core barrel movement Alert Levels and Action Levels at nominal THERMAL POWER levels of 20%, 50%, 80% and 100% of RATED THERMAL POWER will be determined during the reactor startup test program. 26

Data from these detectors is to be reduced in two forms. RMS values are computed from the Amplitude Probability Density (APD) of the signal amplitude. These RMS magnitudes include variations due both to various neutronic effects and internals motion. Consequently, these signals alone can only provide a gross measure of CSB motion. A more accurate assessment of CSB motion is obtained from the Auto and Cross Power Spectral Densities (PSD, XPSD), phase (ø) and coherence (COH) of these signals. These data result from a Spectral Analysis (SA) of the excore detector signals. 26

A modification to the required monitoring program may be justified by an analysis of the data obtained and by an examination of the affected parts during the plant shutdown at the end of the first fuel cycle.