

March 14, 1988

Docket No. 50-277/278

Mr. Edward G. Bauer, Jr.
Vice President and General Counsel
Philadelphia Electric Company
2301 Market Street
Philadelphia, Pennsylvania 19101

Dear Mr. Bauer:

SUBJECT: GENERIC LETTER 83-28, ITEM 2.1.1.

RE: Peach Bottom Atomic Power Station, Units 2 and 3

You have responded to the staff position of Generic Letter 83-28, Item 2.1.1., "Equipment Classification Program" by letters dated November 4, 1983, April 23, 1984 and May 29, 1985. The staff's review of these submittals discloses the need for additional information as described in the Enclosure.

/S/

Robert E. Martin, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II

Enclosure:
As stated

cc: See Next Page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Docket No. 50-277/278

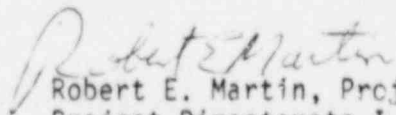
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Robert E. Martin, Project Manager
Project Directorate I-2
Division of Reactor Projects I/II

Enclosure:
As stated

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Mr. E. G. Bauer, Jr.
Philadelphia Electric Company

Peach Bottom Atomic Power Station,
Units 2 and 3

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Peach Bottom Atomic Power Station Units 2, 3

Docket Nos. 50-277/278

Request for Additional Information

For Generic Letter 83-28, Item 2.1 (Part 1)

Introduction:

We have reviewed the responses to Generic Letter 83-28, Item 2.1 (Part 1) dated November 4, 1983, April 23, 1984 and May 29, 1985 provided by Philadelphia Electric Company, the licensee for Peach Bottom Atomic Power Station, Units 2, 3, and find them unacceptable as indicated below. The licensee is requested to provide additional information for these items as described below.

Evaluation and Specific Information Request:

Evaluation:

The licensee's responses state that all systems that contribute to the reactor trip function have been identified as safety-related in the current "Q" list and that all components of safety-related systems are safety-related unless specifically excluded by safety evaluation. The "Q" list is used to identify the applicable codes, standards and procedures to be used for activities relating to the safety-related components.

Each item or service to be procured is reviewed to determine if it is safety-related. The review is performed by a cognizant member of the plant staff or the Engineering and Research Department.

Conclusion:

Item 2.1 (Part 1) requires licensees to confirm that all components whose functioning is required to trip the reactor are identified as safety-related on documents, procedures, and information handling systems used in the plant to control safety-related activities. We find that the licensee's response confirms only that the systems which contribute to reactor trip are identified as safety-related. Components are not confirmed to be so identified on the relevant plant documentation and in information handling systems as requested for this item.

The licensee should confirm that a program of equipment classification has been developed and implemented which does identify components whose functioning is required to trip the reactor as safety-related on documentation and in information handling systems that are used in the plant to control activities which may affect these components.