

LICENSEE EVENT REPORT (LER)

Facility Name (1) Braidwood, Unit 1										Docket Number (2) 015000456					Page (3) 1 of 03																
Title (4) Incomplete Power Operated Relief Valve Surveillance Due to an Overly Restrictive Procedural Requirement																															
Event Date (5)					LER Number (6)					Report Date (7)					Other Facilities Involved (8)																
Month	Day	Year	Year	Sequential Number	Month	Day	Year	Month	Day	Year	Facility Names					Docket Number(s)															
1	2	2	4	8	7	8	7	---	0	1	6	1	---	0	1	0	1	1	1	8	8	NONE					015000456				
OPERATING MODE (9) 1					THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)																										
POWER LEVEL (10) 085					20.402(b)					20.405(c)					50.73(a)(2)(iv)					73.71(b)											
					20.405(a)(1)(i)					50.36(c)(1)					50.73(a)(2)(v)					73.71(c)											
					20.405(a)(1)(ii)					50.36(c)(2)					50.73(a)(2)(vii)					Other (Specify in Abstract below and in Text)											
					20.405(a)(1)(iii)					X 50.73(a)(2)(i)					50.73(a)(2)(viii)(A)																
					20.405(a)(1)(iv)					50.73(a)(2)(ii)					50.73(a)(2)(viii)(B)																
20.405(a)(1)(v)					50.73(a)(2)(iii)					50.73(a)(2)(x)																					
LICENSEE CONTACT FOR THIS LER (12)																															
Name Marvin Engen, Technical Staff Engineer										Ext. 2365										TELEPHONE NUMBER AREA CODE 815 458-2801											
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																															
CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NFRDS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NFRDS																						
SUPPLEMENTAL REPORT EXPECTED (14)										Expected Submission Date (15)																					
Yes (If yes, complete EXPECTED SUBMISSION DATE)										X NO																					
ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)																															

At 1250 on December 24, 1987, it was discovered that the Power Operated Relief Valve (PORV) and PORV Block Valve Stroke Test Surveillance was overdue. PORV 1RY456 was inoperable due to excessive seat leakage and its associated block valve closed. At 1305 both PORV block valves and the remaining operable PORV, 1RY455A, were declared inoperable due to the incomplete surveillance which was initiated on December 11, 1987. The surveillance was satisfactorily performed on both PORV block valves and 1RY455A and the valves were declared operable at 1316.

The cause was overly restrictive procedural requirements which mandated both PORV's to be operable to perform the surveillance.

The surveillance procedure will be revised to preclude this type of occurrence.

There have been no previous licensee event reports due to restrictive surveillance procedures.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)						Page (3)		
		Year	///	Sequential	///	Revision				
				Number		Number				
Braidwood, Unit 1	0 5 0 0 0 4 5 6	8 7	-	0 6 1	-	0 0	0 2	OF	0 3	
TEXT Energy Industry Identification System (EIIIS) codes are identified in the text as [xx]										

A. PLANT CONDITIONS PRIOR TO EVENT:

Unit: Braidwood 1; Event Date: December 24, 1987; Event Time: 1250
 MODE: 1 - Power Operation; Rx Power: 35%; RCS [AB] Temperature/Pressure: 585°F/2228 psig

B. DESCRIPTION OF EVENT:

Unit 1 was in power operation with Pressurizer [AB] Power Operated Relief Valve (PORV) 1RY-456 inoperable due to excessive seat leakage and its associated block valve, 1RY-8000B, closed.

At 1250 on December 24, 1987, during a review of the operating department surveillance schedule, it was discovered that surveillance procedure BwVS 0.5-2.RY.1, Pressurizer System PORV and PORV Block Valve Stroke Test, was overdue. The surveillance had a due date of November 26, 1987 and an expiration of its critical period per Braidwood Technical Specification 4.0.2 of December 17, 1987.

On December 11, 1987, the surveillance procedure was initiated but not completed.

At 1305 on December 24, 1987, both PORV block valves and the remaining operable PORV, 1RY-455A, were declared inoperable due to the incomplete surveillance. Limiting Condition for Operation Action Requirement (LCOAR), 1BWOS 4.4-1A, LCOAR Pressurizer Relief Valves, was entered.

At 1316 on December 24, 1987, BwVS 0.5-2.RY.1 was performed on both block valves and PORV 1RY-455A with satisfactory results and LCOAR 1BWOS 4.4-1a was exited.

Stable plant conditions were maintained throughout the event. Operator actions neither increased nor decreased the severity of the event.

This event is being reported pursuant to 10CFR50.73(a)(2)(i) - Any operation or condition prohibited by the plant's Technical Specifications.

C. CAUSE OF EVENT:

The cause of this event was overly restrictive procedural requirements which mandated both PORV's to be operable to perform the surveillance. Precaution number one of BwVS 0.5-2.RY.1 states, "Do not perform this test on one PORV and its respective block valve without the opposite PORV and block valve being operable in operating modes 1, 2, 3."

D. SAFETY ANALYSIS:

There were no safety consequences associated with the event as it occurred as no overpressure condition existed at the time. Under worst case conditions of an actual overpressure event, there would have been no impact on the plant or public safety as no credit is taken in the applicable accident analyses for the operation of the PORV's. The purpose of the PORV's is to minimize challenges to the safety valves, which were operable throughout the event.

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E. CORRECTIVE ACTIONS:

Corrective action to prevent recurrence shall be to revise BWVS 0.5-2.RY.1 such that precaution number one provides for LCOAR entry should a PORV be inoperable. The precaution shall be reworded to allow performance of the surveillance with a PORV inoperable and direct entry into 1BWOS 4.4-1a. This will be tracked to completion by Action Item 456-200-87-42101.

F. PREVIOUS OCCURRENCES:

There have been no previous licensee event reports due to restrictive surveillance procedures.

G. COMPONENT FAILURE DATA:

This event was not caused by component failure, nor did any associated components fail as a result of this event.



Commonwealth Edison
Braidwood Nuclear Power Station
Route #1, Box 84
Braceville, Illinois 60407
Telephone 815/458-2801

EEF/88-68

January 12, 1988

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

The enclosed Licensee Event Report from Braidwood Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73(a)(2) (i) which requires a 30 day written report.

This report is number 87-061-00; Docket No. 50-456.

Very truly yours,

E. E. Fitzpatrick 1/16/88

E. E. Fitzpatrick
Station Manager
Braidwood Nuclear Station

EEF/PMB/jab
(6466z)

Enclosure: Licensee Event Report No. 87-061-00

cc: NRC Region III Administrator
T. Tongue, NRC Resident Inspector
INPO Record Center
CECo Distribution List

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