

March 2, 1988

Docket No. 50-368

Mr. T. Gene Campbell
Vice President, Nuclear
Operations
Arkansas Power and Light Company
Post Office Box 551
Little Rock, Arkansas 72203

Dear Mr. Campbell:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION, REACTOR COOLANT PUMP TRIP,
ARKANSAS NUCLEAR ONE, UNIT 2 (TAC. NO. 49676)

After reviewing your response (November 24, 1986, 2CAN118608) to Generic Letter 86-06, Implementation of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps", we find that we need additional information so that we may complete our review. Please provide answers to the enclosed questions within 50 days of the date of this letter.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,

15/

George F. Dick, Jr., Project Manager
Project Directorate - IV
Division of Reactor Project - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

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Mr. T. Gene Campbell
Arkansas Power & Light Company

Arkansas Nuclear One, Unit 2

CC:

Mr. J. Ted Enos, Manager
Nuclear Engineering and Licensing
Arkansas Power & Light Company
P. O. Box 551
Little Rock, Arkansas 72203

Mr. James M. Levine, Director
Site Nuclear Operations
Arkansas Nuclear One
P. O. Box 608
Russellville, Arkansas 72801

Mr. Nicholas S. Reynolds
Bishop, Liberman, Cook, Purcell
1200 Seventeenth Street, N.W.
Suite 700
Washington, D.C. 20036

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
Office of Executive Director for
Operations
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
1 Nuclear Plant Road
Russellville, Arkansas 72801

Ms. Greta Dicus, Director
Division of Environmental Health
Protection
Arkansas Department of Health
4815 West Markam Street
Little Rock, Arkansas 72201

Mr. Robert B. Borsum
Babcock & Wilcox
Nuclear Power Generation Division
Suite 220, 7910 Woodmont Avenue
Bethesda, Maryland 20814

Mr. Charles B. Brinkman, Manager
Washington Nuclear Operations
C-E Power Systems
7910 Woodmont Avenue
Suite 1310
Bethesda, Maryland 20814

Mr. Frank Wilson, Director
Division of Environmental Health
Protection
Department of Health
Arkansas Department of Health
4815 West Markham Street
Little Rock, Arkansas 72201

Honorable William Abernathy
County Judge of Pope County
Pope County Courthouse
Russellville, Arkansas 72801

ADDITIONAL QUESTIONS ON
ARKANSAS POWER AND LIGHT'S RESPONSE TO
GENERIC LETTER 86-06
FOR
ARKANSAS NUCLEAR ONE, UNIT 2

1. Arkansas Power & Light's (AP&L's) letter of November 24, 1986 did not clearly identify which of the criteria presented in CEN-268 was selected to trip the second set of pumps during a small break LOCA (SBLOCA) at Arkansas Nuclear One, Unit 2 (ANO-2). Identify the criterion selected and the setpoints used to determine when to trip the second set of pumps. Also, identify the pressure setpoint used to trip the first set of pumps if different from that recommended in CEN-268.
2. For the instrumentation identified in response to question 1, discuss how the effects of instrument uncertainty, as identified in AP&L's November 24, 1986 response to Generic Letter (GL 86-06 item 2) were included in determining the setpoints for pressure, subcooled margin, and secondary reactivity.
3. AP&L did not provide sufficient information in its November 24, 1986 response to GL 86-06 item 3 to determine how the uncertainties in the generic analysis presented in CEN-268 affect the results as they apply to ANO-2. Therefore, identify the ANO-2 plant specific features not representative of the reference plant used in the analyses presented in CEN-268. At a minimum discuss core power; decay heat; HPIS capacity; makeup flows; setpoints for reactor trip, safety injection, and accumulator injection and show that the values used in the generic analysis are either representative of those at ANO-2 or conservative. If a reference plant parameter is not representative for ANO-2, discuss how this was considered in determining the plant specific setpoints.
4. Part 2 of Item 4 of Generic Letter 86-06 requested the licensee to identify, "...procedures which provide direction for use of individual steam generators with and without operating RCPs." In its November 24, 1986 response, AP&L stated that because all four RCPs are not tripped for transients that cause use of individual steam generators and the first two pumps tripped are always in opposite loops, use of individual steam generators has no relationship to RCP trip criteria. Because the ANO-2 procedures will always allow one pump in a loop to be operating in situations where a single steam generator will be used, identifying all procedures and operator training that involve use of single steam generators is sufficient to comply with this item. The licensee is requested to provide this information.

5. In its response to item 4, AP&L stated that emergency operating procedure (EOP) 2202.01 requires the use of reactor coolant pump trip guidelines. Identify what situations, i.e., main steam line breaks, steam generator tube ruptures, small break LOCAs, etc., are covered by EOP 2202.01.