

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Rancho Seco Nuclear Generating Station										DOCKET NUMBER (2) 0 5 0 0 0 3 1 2 1 OF 0 3										PAGE (3) 1								
TITLE (4) Surveillance Procedures Not Performed by Technical Specification Date Due to Personnel Error																												
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES None										DOCKET NUMBER(S) 0 5 0 0 0									
1	2	1	5	8	7	8	7	0	4	6	0	0	0	1	1	3	8	8	0 5 0 0 0									
OPERATING MODE (9) N			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)																									
POWER LEVEL (10) 01010			20.402(b)						20.405(c)						50.73(a)(2)(iv)						73.71(b)							
			20.405(a)(1)(i)						50.36(c)(1)						50.73(a)(2)(v)						73.71(c)							
			20.405(a)(1)(ii)						50.36(c)(2)						50.73(a)(2)(vi)						OTHER (Specify in Abstract below and in Text, NRC Form 365A)							
			20.405(a)(1)(iii)						50.73(a)(2)(i)						50.73(a)(2)(viii)(A)													
			20.405(a)(1)(iv)						50.73(a)(2)(ii)						50.73(a)(2)(viii)(B)													
			20.405(a)(1)(v)						50.73(a)(2)(iii)						50.73(a)(2)(ix)													
LICENSEE CONTACT FOR THIS LER (12)																												
NAME Paul Lavelly, Supervisor, Independent Investigation/Reviews															TELEPHONE NUMBER 911 6415 21-13 1211													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																												
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS									
SUPPLEMENTAL REPORT EXPECTED (14)															EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR									
YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO <input type="checkbox"/>																												
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)																												
<p>On November 26, 1987, Decay Heat System Valves HV-20001 and HV-20002 were returned to operable status. The valves were energized, stroked, and left in the open position.</p> <p>Surveillance Procedure (SP) 203.06C/D requires verification of valve stroke time and position indication following a cold shutdown of 72 hours or greater, provided it has been three months since the valves were last exercised. The valves had been scheduled for the quarterly performance of SP.203.06C/D on September 18, 1987, but the procedure was not performed because the valves were not in service at that time. SP.203.06C/D should have been performed on November 26, 1987 when the valves were returned to service. The procedure was not performed at that time. On December 15, 1987, the routine quarterly surveillance SP.31B (SP.31B superseded SP.203.06C/D) was successfully performed. Between November 26, 1987 and December 15, 1987 the valves did not meet the operability requirements of the Technical Specifications. The valves were in the open position and were functional; therefore, the Decay Heat System was functional during the period that the valves were technically inoperable.</p> <p>A search of previously submitted LERs disclosed four similar occurrences. LERs 85-08, 86-06, 87-12, and 87-45 related to Technical Specification required surveillances missed due to scheduling problems.</p>																												
<div style="display: flex; justify-content: space-between;"> 8801220052 880113 PDR ADOCK 05000312 S DCD IE22 16 </div>																												

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Rancho Seco Nuclear Generating Station	05000312	87	046	000	2	OF	03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

DESCRIPTION

On November 26, 1987, Decay Heat System Valves HV-20001 and HV-20002 were returned to operable status. The valves were energized, stroked, and left in the open position.

Surveillance Procedure (SP) 203.06C/D requires verification of valve stroke time and position indication following a cold shutdown of 72 hours or greater, provided it has been three months since the valves were last exercised. The valves had been scheduled for the quarterly performance of SP.203.06C/D on September 18, 1987, but the procedure was not performed because the valves were not in service at that time. SP.203.06C/D should have been performed on November 26, 1987 when the valves were returned to service. The procedure was not performed at that time. On December 15, 1987, the routine quarterly surveillance SP.31B (SP.31B superseded SP.203.06C/D) was successfully performed. Between November 26, 1987 and December 15, 1987 the valves did not meet the operability requirements of the Technical Specifications. The valves were in the open position and were functional; therefore, the Decay Heat System was functional during the period that the valves were technically inoperable. The plant was never in a degraded mode.

The failure to perform Surveillance Procedure SP.203.06C/D by the scheduled date of September 18, 1987, and the failure to perform SP.203.06C/D before the valves were returned to service placed the plant in a condition which violated 10 CFR 50.73.a.2.i(B) and Technical Specification 4.2.2.1.

PLANT OPERATING CONDITIONS BEFORE THE EVENT

The plant has been in continuous cold shutdown since December 29, 1985.

CAUSE

The cause of the failure to perform SP.203.06C/D on Valves HV-20001 and HV-20002 prior to returning the valves to service was personnel error.

During the interval that the valves were not considered to be operable, the Operations group was provided a notice stating that SP.203.06C/D was scheduled to be performed by September 18, 1987. SP 203.06C/D was not performed because the valves were not operable, thereby precluding the necessity for performance of the surveillance procedure. When the valves were returned to service the required surveillance SP.203.06C/D was overlooked.

THE ENERGY INDUSTRY IDENTIFICATION SYSTEM (EIIS) COMPONENT AND SYSTEM IDENTIFIER

The EIIS System Identifier for the Class 1 Decay Heat System is BP. The EIIS Component Identifier for the valves is ISV.

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TEXT (If more space is required, use additional NRC Form 365A's) (17)

METHOD OF DISCOVERY

Failure to perform the Technical Specification surveillance for Valves HV-20001 and HV-20002 was discovered by the Surveillance Procedure Implementation Review portion of the Operational Readiness Review Inspection.

SAFETY CONSEQUENCES

Valves HV-20001 and HV-20002 were technically inoperable for 19 days. During this period the valves were in the open position, as verified by flow measurement from FE-26003 and FE-26004, thereby providing decay heat removal capability.

CORRECTIVE ACTIONS

By April 1, 1988 a matrix addressing surveillance procedures relative to plant mode will be in place. This matrix will be incorporated into Administrative Procedure AP.303 "Surveillance Program." This, in conjunction with the weekly surveillance schedule, the daily overdue list, and the failed list will preclude recurrence of missed or neglected surveillances.

PREVIOUS SIMILAR EVENTS

A search of previously submitted LERs disclosed four similar occurrences. LERs 85-08, 86-06, 87-12, and 87-45 were related to Technical Specification required surveillances missed due to scheduling problems.



SMUD

SACRAMENTO MUNICIPAL UTILITY DISTRICT ☐ 6201 S Street, P.O. Box 15830, Sacramento CA 95852-1830, (916) 452-3211
AN ELECTRIC SYSTEM SERVING THE HEART OF CALIFORNIA

GCA 87-917
JAN 13 1988

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

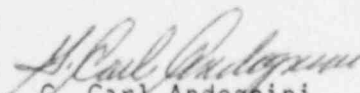
Docket No. 50-312
Rancho Seco Nuclear Generating Station
License No. DPR-54
**LICENSEE EVENT REPORT 87-046 SURVEILLANCE PROCEDURES NOT PERFORMED BY
TECHNICAL SPECIFICATION REQUIRED DATE DUE TO PERSONNEL ERROR**

Dear Sirs:

In accordance with the requirements of 10 CFR Part 50.73 the Sacramento Municipal Utility District hereby submits Licensee Event Report Number 87-046.

Please contact me if you have any questions. Members of your staff with questions requiring additional information or clarification may contact Mr. Paul Lavelly at (916) 452-3211, extension 4674.

Sincerely,


G. Carl Andognini
Chief Executive Officer,
Nuclear

Attachment

cc w/atch:

G. Kalman, NRC, Bethesda (2)
A. D'Angelo, NRC, Rancho Seco
J. B. Martin (2)
INPO

IE22
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