AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346
UNIT Davis-Besse Unit 1
DATE October 6, 1978
COMPLETED BY E. Caba
TELEPHONE 419-259-5000, Ext. 251

MONTH September, 1978

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	663	17	881
2	665	18	880
3	663	19	881
4	667	20	881
5	665	21	884
6	659	22	889
7	654	23	889
8	653	24	885
9	620	25	884
10	258	26	885
11	458	27	885
12	720	28	48
13	799	29	0
14	801	30	0
15	816	31	-
16	878		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

7810130144

OPERATING DATA REPORT

DOCKET NO. 50-346
DATE October 6, 1978
COMPLETED BY E. Caba
TELEPHONE 419-259-5000, Ext. 251

OPERATING STATUS

1. Unit Name: Davis-Besse Unit 1
2. Reporting Period: September, 1978
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): to be determined
7. Maximum Dependable Capacity (Net MWe): to be determined
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720	6551	9556
12. Number Of Hours Reactor Was Critical	636.4	3553.6	5345.7
13. Reactor Reserve Shutdown Hours	19.1	38.9	422.6
14. Hours Generator On-Line	631.3	3246.4	4713.2
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,535,662	6,174,797	7,838,829
17. Gross Electrical Energy Generated (MWH)	518,725	2,098,898	2,623,347
18. Net Electrical Energy Generated (MWH)	488,193	1,922,139	2,351,957
19. Unit Service Factor	87.7%	49.6%	52.9%
20. Unit Availability Factor	87.7%	49.6%	52.9%
21. Unit Capacity Factor (Using MDC Net)	to be determined		
22. Unit Capacity Factor (Using DER Net)	74.8%	32.4%	32.9%
23. Unit Forced Outage Rate	9.9%	25.7%	25.1%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: October 2, 1978

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		8/12/77
INITIAL ELECTRICITY		8/28/77
COMMERCIAL OPERATION		11/21/77*
		12/19/77**
		1/23/78***
		7/31/78****

*Declared operational at 25%
**Declared operational at 40% (from 25%)
***Declared operational at 75% (from 40%)
****Declared operational at 100% (from 75%)

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH September, 1978

DOCKET NO. 50-346
UNIT NAME Davis-Besse Unit 1
DATE October 6, 1978
COMPLETED BY C. Alm
TELEPHONE 419-259-5000, Ext. 251

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
23	78 09 10	S	19.1	B	3	N/A	N/A	N/A	The turbine-generator was tripped to perform the Reactor Turbine Trip Test (TP 800.14).
24	78 09 28	F	69.6	A	3	N/A	IA	INSTRU	The reactor tripped because of reactor coolant system (RCS) low pressure. The RCS low pressure occurred when the steam generators were overfed during an attempt to stabilize plant parameters. The transient parameters were caused by the failure of the RCS flow transmitter for Loop 2 of the RCS.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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OPERATIONAL SUMMARY FOR SEPTEMBER 1978

9/1/78 - 9/8/78	Reactor power was maintained at 75 ± 1 percent with the generator gross load at 705 ± 10 MWe. During this time period, data was acquired for physics testing and preparations were initiated to perform unit transient testing.
9/9/78	The Unit Load Transient Test (TP 800.23) was initiated at 0955 hours. At 2130 hours, reactor power was returned to 75 percent and preparations were made to perform the Reactor Turbine Trip Test (TP 800.14).
9/10/78	The turbine-generator was tripped at 0931 hours as per TP 800.14. The reactor then tripped at 0932 hours because of reactor coolant system low pressure. Reactor criticality was re-established at 2324 hours.
9/11/78 - 9/12/78	The turbine-generator was synchronized on line at 0437 hours on September 11, 1978. Reactor power was then increased and attained 90 percent at 1750 hours on September 12, 1978. During this power increase, reactor power was maintained at 70 percent for eight hours and 75 percent for eighteen hours. The generator gross load at 90 percent reactor power was 835 ± 10 MWe.
9/12/78 - 9/15/78	Reactor power was maintained at 90 percent from 1750 hours on September 12, 1978 to 1528 hours on September 15, 1978. Reactor power was then increased and attained 97 percent at 1750 hours on September 15, 1978.
9/16/78 - 9/28/78	At 0750 hours on September 16, 1978, reactor power was increased and attained 100 percent full power at 0930 hours on September 16, 1978. This power level was maintained until 0225 hours on September 28, 1978 when the reactor tripped.
9/28/78 - 9/30/78	The reactor tripped at 0225 hours on September 28, 1978 because of reactor coolant system (RCS) low pressure. The RCS low pressure occurred when the steam generators were overfed during an attempt to stabilize plant parameters. The transient parameters were caused by the failure of the RCS flow transmitter for Loop 2 of the RCS. The unit outage continued the remainder of this month.

FACILITY CHANGE REQUESTS COMPLETED DURING SEPTEMBER, 1978

FCR NO: 77-207

SYSTEM: Safety Features Actuation System (SFAS)

COMPONENT: Spare Field Cable L432D/L434D

CHANGE, TEST, OR EXPERIMENT: On August 25, 1978, work was completed which lifted spare field cable L432D/L434D from Terminal Board 65, Terminals 6 and 7 in SFAS Channel 4 (Cabinet C 5756C), and labeled the cable as a spare. The applicable drawings already showed the cable as a spare.

REASON FOR FCR: Cable L432D/L434D was no longer an active SFAS cable. It was a spare cable attached to Terminal Board 65, Terminals 6 and 7 with non-conducting plastic screws. There was no electrical connection, and hence no effect on SFAS operation.

SAFETY EVALUATION: This FCR was reviewed by the unit architect/engineer, Bechtel Corporation, who determined that the subject terminations should indeed be disconnected and tagged as spare conductors in the SFAS Cabinet. There is no unreviewed safety issue when spare wires which were attached with insulated plastic screws are lifted and tagged as spares.