



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-304

ZION STATION UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37
License No. DPR-48

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated May 24, 1977 as supplemented September 27, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

7812180240

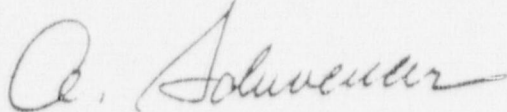
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C (2) of Facility License No. DPR-48 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 37, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 30, 1978

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. DPR-39

AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. DPR-48

DOCKET NOS. 50-295 AND 50-304

Revise Appendix A as follows:

Remove Pages

40
197
198

Insert Revised Pages

40
197
198

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p>C. Unit Startup</p> <ol style="list-style-type: none"> 1. Immediately prior to startup, the reactor coolant temperature shall be shown to be greater than the temperature above which the moderator temperature coefficient is always negative and greater than 500°F, except during low power physics tests. 2. When a reactor is approaching criticality, the shutdown banks shall be fully withdrawn in sequence (shutdown bank A,B,C,D) before any other rods are withdrawn. The control group rods shall be no further inserted than the limits shown by Figure 3.2.2 for Unit 1 and Figure 3.2-4 for Unit 2 for 4-loop operation and Figure 3.2-3 for Unit 1 and Figure 3.2-5 for Unit 2 for 3-loop operation when criticality is attained. <p>D. Power Operation</p> <ol style="list-style-type: none"> 1. When a reactor is critical, except for physics tests and control rod exercises, the shutdown rods shall be fully with- 	<p>4.2.1.B once a shift while remaining in this condition. During heatup, the boron concentration in the reactor coolant loops and pressurizer shall be sampled every 4 hours. The reactor coolant loop boron concentration must not decrease by more than 50 ppm between successive 4 hour samples. The pressurizer boron concentration must not be more than 200 ppm less than the reactor coolant loop boron concentration.</p> <p>C. Startup</p> <ol style="list-style-type: none"> 1. The Tavg of each reactor coolant loop shall be logged before attempting to bring a reactor critical. 2. Not Applicable. <p>D. Power Operation</p> <ol style="list-style-type: none"> 1. Rod operation shall be verified by partial movement of all rods every two weeks. Rods which have been exercised within the <p>Amendment No. 40, Unit 1 Amendment No. 37, Unit 2</p>

PROPOSED TECHNICAL SPECIFICATION SECTION 3.9.1.B

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p>3.9 <u>CONTAINMENT ISOLATION SYSTEMS (per unit)</u></p> <p><u>Applicability:</u></p> <p>Applies to containment isolation systems.</p> <p><u>Objective:</u></p> <p>To insure containment isolation within acceptable limits and define the operating status of the reactor containment for plant operation.</p> <p><u>Specification:</u></p> <p>1. Isolation valve seal water system</p> <p>A. The isolation valve seal water system shall be operable unless the reactor is in the cold shutdown condition except as specified in 3.9.1C.</p> <p>B. The isolation valve seal water tank shall be maintained at a minimum pressure of 68 psig and a minimum volume of 70 gallons.</p> <p>C. Any one header of the isolation valve seal water system may be inoperable for a period not to exceed four consecutive days during reactor operation to permit maintenance</p>	<p>4.9 <u>CONTAINMENT ISOLATION SYSTEM (per unit)</u></p> <p><u>Applicability:</u></p> <p>Applies to containment isolation systems.</p> <p><u>Objective:</u></p> <p>To establish the surveillance requirement for containment isolation.</p> <p><u>Specification:</u></p> <p>1. Isolation valve seal water system (Table 4.9-1)</p> <p>A. The isolation valve seal water system shall be tested every refueling outage. Performance will be acceptable if the five seal water headers are maintained at a minimum pressure of 47 psig for two hours following manual actuation. The seal water header isolation valves are closed throughout the test.</p> <p>B. The level and pressure of the isolation valve seal water tank shall be checked monthly.</p> <p>C. Not Applicable.</p>

PROPOSED TECHNICAL SPECIFICATION SECTION 3.9.2.E

LIMITING CONDITION FOR OPERATION	SURVEILLANCE REQUIREMENT
<p>3.9.2. Penetration pressurization systems</p> <p>A. The penetrations (electrical and mechanical) and liner weld channels shall be continuously pressurized above 47 psig unless the reactor is in the cold shutdown condition except as specified in 3.9.2B.</p> <p>B. Any one header of the nitrogen or air pressurization system may be inoperable for a period not to exceed four consecutive days during reactor operation to permit maintenance.</p> <p>C. The penetration air compressor system and either the instrument air system or the nitrogen system shall be operable except as specified in 3.9.2.D.</p> <p>D. One penetration air compressor may be inoperable for a period not to exceed 15 days to permit maintenance.</p> <p>E. The upper limit for long term uncorrected air consumption for the penetration pressurization systems shall be less than 0.2% of the containment volume per day at 47 psig (950 SCFH).</p>	<p>4.9 2. Penetration pressurization systems Table 4.9-2)</p> <p>A. The pressure of the four nitrogen and penetration pressurization headers shall be checked monthly.</p> <p>B. Immediately following maintenance on a pressurization header, the header pressure must be checked.</p> <p>C. The penetration air compressor shall be load tested (+ 10% of design pressure and capacity) during each refueling outage.</p> <p>D. Immediately following maintenance on a penetration compressor, that compressor shall be load tested.</p> <p>E. The air flow to the penetration pressurization headers shall be checked monthly.</p>