

DCW

June 26, 1997

Mr. H. G. Stanley  
Site Vice President  
Braidwood Nuclear Power Station  
Commonwealth Edison Company  
R.R. #1, Box 84  
Braceville, IL 60407

SUBJECT: NRC INSPECTION REPORT 50-456/97010(DRS)

Dear Mr. Stanley:

On June 2, 1997, the NRC completed an inspection at your Braidwood, Unit 1, reactor facility. The enclosed report presents the results of that inspection.

The inspection included a review of inservice inspection activities and the examination of steam generator tubes. Your efforts to evaluate tube integrity by assessing growth rates for outside diameter stress corrosion cracking using historical eddy current data reviews/comparisons and in situ pressure testing were indicative of an aggressive program. The decision to provide additional oversight of the contractors' activities and procedures for eddy current examination of the steam generator tubing was considered a program improvement.

In accordance with 10 CFR 2.790 of the Commission's regulations, a copy of this letter and the enclosed inspection report will be placed in the NRC Public Document Room.

We will gladly discuss any questions you have concerning this inspection.

Sincerely,

/s/ John M. Jacobson (for)

John A. Grobe, Acting Director  
Division of Reactor Safety

Docket No. 50-456

Enclosure: Inspection Report 50-456/97010(DRS)

See Attached Distribution



DOCUMENT NAME: G:\DRS\BRA97010.DRS (See Previous Concurrence)

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DATE	06/19/97	06/19/97		06/20/97	06/ /97

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June 26, 1997

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