

50-369



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 15, 1997

Mr. H. B. Barron
Vice President, McGuire Site
Duke Power Company
12700 Hagers Ferry Road
Huntersville, North Carolina 28078

SUBJECT: MCGUIRE NUCLEAR STATION - COMPLETION OF LICENSING ACTION FOR GENERIC LETTER 95-03, "CIRCUMFERENTIAL CRACKING OF STEAM GENERATOR TUBES," DATED APRIL 28, 1995 (TAC M92251 AND M92252)

Dear Mr. Barron:

On April 28, 1995, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter (GL) 95-03, "Circumferential Cracking of Steam Generator Tubes," to all holders of operating licenses or construction permits for pressurized-water reactors. The NRC issued GL 95-03 for three principal reasons:

- (1) Notify addressees about the safety significance of the recent steam generator tube inspection findings at Maine Yankee Atomic Power Station.
- (2) Request that all addressees implement the actions described within the generic letter.
- (3) Require that all addressees submit to the NRC a written response regarding implementation of the requested actions.

In addition, GL 95-03 alerted addressees to the importance of performing comprehensive examinations of steam generator tubes using techniques and equipment capable of reliably detecting the types of degradation to which the steam generator tubes may be susceptible. The staff also noted that the performance of steam generator tube examinations is controlled, in part, by Appendix B to Title 10, Part 50, of the *Code of Federal Regulations* (10 CFR Part 50).

In GL 95-03, the NRC staff also requested that licensees take the following actions:

- (1) Evaluate recent operating experience with respect to the detection and sizing of circumferential indications to determine the applicability to their plants.
- (2) On the basis of such evaluation, as well as past inspection scope and results, susceptibility to circumferential cracking, threshold of detection, expected or inferred crack growth rates, and other relevant factors, develop a safety assessment justifying continued operation until the next scheduled steam generator tube inspections.

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- (3) Develop plans for the next steam generator tube inspections as they pertain to the detection of circumferential cracking. The inspection plans should address, but not be limited to, scope (including sample expansion criteria, if applicable), methods, equipment, and criteria (including personnel training and qualification).

To document the outcome of these actions, the NRC staff requested that addressees prepare and submit the following:

- (1) A safety assessment justifying continued operation, predicated on the evaluation performed in accordance with requested actions 1 and 2 (above).
- (2) A summary of the inspection plans developed in accordance with requested action 3 (above) and a schedule for the next planned inspection.

In response to GL 95-03, Duke Power Company provided letters dated June 27, October 9, and October 25, 1995, for the McGuire Nuclear Station. These submittals provided the information requested by GL 95-03; therefore, the above TAC numbers are closed. For your information, the staff's findings regarding this issue are contained in NUREG-1604, "Circumferential Cracking of Steam Generator Tubes."

If you have any questions regarding this matter, please contact me at 301-415-1484.

Sincerely,

ORIGINAL SIGNED BY:

Victor Nerses, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-370

cc: See next page

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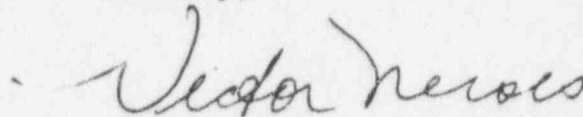
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Victor Nerses, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-370

cc: See next page

McGuire Nuclear Station
Units 1 and 2

cc:

Mr. Paul R. Newton
Duke Power Company, PB05E
422 South Church Street
Charlotte, North Carolina 28242-0001

County Manager of Mecklenberg County
720 East Fourth Street
Charlotte, North Carolina 28202

Mr. Michael T. Cash
Regulatory Compliance Manager
McGuire Nuclear Station
Duke Power Company
12700 Hagers Ferry Road
Huntersville, North Carolina 28078-8985

J. Michael McGarry, III, Esquire
Winston and Strawn
1400 L Street, NW.
Washington, DC 20005

Senior Resident Inspector
c/o U. S. Nuclear Regulatory
Commission
12700 Hagers Ferry Road
Huntersville, North Carolina 28078

Mr. T. Richard Puryear
Owner's Group (NCEMC)
Duke Power Company
4800 Concord Road
York, South Carolina 29745

Dr. John M. Barry
Mecklenberg County
Department of Environmental
Protection
700 N. Tryon Street
Charlotte, North Carolina 28202

Mr. Dayne H. Brown, Director
Department of Environmental,
Health and Natural Resources
Division of Radiation Protection
P. O. Box 27687
Raleigh, North Carolina 27611-7687

Ms. Karen E. Long
Assistant Attorney General
North Carolina Department of
Justice
P. O. Box 629
Raleigh, North Carolina 27602

Mr. G. A. Copp
Licensing - EC050
Duke Power Company
526 South Church Street
Charlotte, North Carolina 28242-0001

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, S.W., Suite 23T85
Atlanta, Georgia 30303

Elaine Wathen, Lead REP Planner
Division of Emergency Management
116 West Jones Street
Raleigh, North Carolina 27603-1335

Mr. H. B. Barron
Vice President, McGuire Site
Duke Power Company
12700 Hagers Ferry Road
Huntersville, North Carolina 28078