

Commonwealth Edison Company
Quad Cities Generating Station
22710 206th Avenue North
Cordova, IL 61242-9740
Tel 309-654-2241



LWP-97-052

May 6, 1997

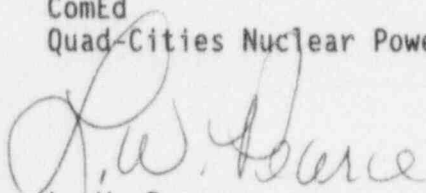
U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of April, 1997.

Respectfully,

ComEd
Quad-Cities Nuclear Power Station


L. W. Pearce
Station Manager

LWP/dak

Enclosure

cc: A. Beach, Regional Administrator
C. Miller, Senior Resident Inspector

160041

STMGR\05297.LWP



9705160227 970430
PDR ADOCK 05000254
R PDR

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

APRIL 1997

COMMONWEALTH EDISON COMPANY

AND

MIDAMERICAN ENERGY COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

TABLE OF CONTENTS

- I. Introduction
- II. Summary of Operating Experience
 - A. Unit One
 - B. Unit Two
- III. Plant or Procedure Changes, Tests, Experiments, and Safety Related Maintenance
 - A. Amendments to Facility License or Technical Specifications
 - B. Facility or Procedure Changes Requiring NRC Approval
 - C. Tests and Experiments Requiring NRC Approval
- IV. Licensee Event Reports
- V. Data Tabulations
 - A. Operating Data Report
 - B. Average Daily Unit Power Level
 - C. Unit Shutdowns and Power Reductions
- VI. Unique Reporting Requirements
 - A. Main Steam Relief Valve Operations
 - B. Control Rod Drive Scram Timing Data
- VII. Refueling Information
- VIII. Glossary

I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors and Steam Turbine/Generators, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Midamerican Energy Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Kristal Sirles and Debra Kelley, telephone number 309-654-2241, extensions 3070 and 2240, respectively.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One began the month of April 1997 on-line. On April 6, 1997 at 0855 hours, a load drop was performed to adjust the setpoint after receiving a high level alarm from the 1D3 Feedwater Heater. On April 9, 1997 at 1330 hours a shutdown was initiated due to problems with 4KV Breaker Auxiliary Switches. At 2344 hours the Main Turbine was taken off-line and Forced Outage Q1F41 commenced. On April 10, 1997 at 0006 hours the reactor was manually scrammed. A design change was implemented to repair the cracked mounting channels in the 4KV Breaker Auxiliary Switches. After successful testing, the reactor went critical on April 24, 1997 at 0334 hours. Unit One exited Forced Outage Q1F41 on April 24, 1997 at 1252 hours when the generator was synchronized to the grid. A few other load drops were performed, however the average daily power level remained at 80% or greater.

B. Unit Two

Quad Cities Unit Two was shutdown the entire month of April due to Refuel Outage Q2R14. Start-up is scheduled for May 27, 1997.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS,
AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements of 10CFR50.73.

UNIT 1

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of occurrence</u>
97-003	3/26/97	Secondary Containment Volume Less than that Described in FSAR. Downgraded to PIR 1-97-060
97-008	4/2/97	High Pressure Coolant Injection (HPCI) inoperable due to Turning Gear Failure.
97-010	4/7/97	Train "B" Control Room HVAC System Inoperable due to Loss of refrigerant.
97-011	4/9/97	The Residual Heat Removal Service Water System was made inoperable due to uncertainties related to 4 KV Air Magne-Blast horizontal Gas (AMHG) circuit breakers, which had experienced cracks in the auxiliary switches mounting channels.
97-012	4/10/97	It is physically possible for the Unit 1 & 2 Refuel Bridge Monorail Auxiliary Hoists to perform core alterations without a Refuel Bridge Position Interlock actuation. This is outside the design basis which requires the Refuel Position Interlocks for associated equipment to be operable when performing core alterations.
97-013	4/16/97	RCIC Hi Temperature Isolation Switch
97-014	4/16/97	Safety Relief Valve not Tested.

NOTE: Sequence of LER Numbers is out of order due to reuse of numbers.

UNIT 2

Licensee Event
Report Number

Date

Title of occurrence

There were no LER's for Unit 2 during the month of April.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT One

DATE May 6, 1997

COMPLETED BY Kristal Sirles

TELEPHONE (309) 654-2241

OPERATING STATUS

0000 040197

1. REPORTING PERIOD: 2400 043097 GROSS HOURS IN REPORTING PERIOD: 719

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-NET): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	379.50	2260.10	166760.30
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	3421.90
7. HOURS GENERATOR ON LINE	345.80	2171.50	161700.40
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	909.20
9. GROSS THERMAL ENERGY GENERATED (MWH)	727408.80	4946385.60	352119268.10
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	232367.00	1599937.00	114021367.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	221261.00	1527940.00	107682909.00
12. REACTOR SERVICE FACTOR	52.78	78.50	75.94
13. REACTOR AVAILABILITY FACTOR	52.78	78.50	77.50
14. UNIT SERVICE FACTOR	48.09	75.43	73.64
15. UNIT AVAILABILITY FACTOR	48.09	75.43	74.05
16. UNIT CAPACITY FACTOR (Using MDC)	40.02	69.01	63.77
17. UNIT CAPACITY FACTOR (Using Design MWe)	39.00	67.26	62.15
18. UNIT FORCED OUTAGE RATE	51.91	24.57	7.76

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): N/A

20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP: N/A

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): N/A

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT Two

DATE May 6, 1997

COMPLETED BY Kristal Sirles

TELEPHONE (309) 654-2241

OPERATING STATUS

0000 040197

1. REPORTING PERIOD: 2400 043097 GROSS HOURS IN REPORTING PERIOD: 719

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-NET): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	1408.30	161994.85
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	2985.80
7. HOURS GENERATOR ON LINE	0.00	1407.70	157657.05
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	702.90
9. GROSS THERMAL ENERGY GENERATED (Mv/H)	0.00	3477281.50	343149370.02
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.00	1118808.00	110135945.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	0.00	1076518.00	104396239.00
12. REACTOR SERVICE FACTOR	0.00	48.92	74.30
13. REACTOR AVAILABILITY FACTOR	0.00	48.92	75.67
14. UNIT SERVICE FACTOR	0.00	48.90	72.31
15. UNIT AVAILABILITY FACTOR	0.00	48.90	72.63
16. UNIT CAPACITY FACTOR (Using MDC)	0.00	48.62	62.26
17. UNIT CAPACITY FACTOR (Using Design MWe)	0.00	47.39	60.69
18. UNIT FORCED OUTAGE RATE	0.00	0.00	11.08

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): N/A

20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP: 5/27/97

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): N/A

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE May 6, 1997
COMPLETED BY Kristal Sirles
TELEPHONE (309) 654-2241

MONTH April, 1997

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>781</u>
2.	<u>785</u>
3.	<u>785</u>
4.	<u>784</u>
5.	<u>736</u>
6.	<u>648</u>
7.	<u>561</u>
8.	<u>636</u>
9.	<u>599</u>
10.	<u>- 7</u>
11.	<u>- 6</u>
12.	<u>- 6</u>
13.	<u>- 6</u>
14.	<u>- 6</u>
15.	<u>- 6</u>
16.	<u>- 6</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>- 6</u>
18.	<u>- 6</u>
19.	<u>- 6</u>
20.	<u>- 7</u>
21.	<u>- 7</u>
22.	<u>- 7</u>
23.	<u>- 6</u>
24.	<u>- 7</u>
25.	<u>30</u>
26.	<u>141</u>
27.	<u>535</u>
28.	<u>758</u>
29.	<u>783</u>
30.	<u>782</u>
31.	<u></u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE May 6, 1997
COMPLETED BY Kristal Sirles
TELEPHONE (309) 654-2241

MONTH April, 1997

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>-8</u>
2.	<u>-8</u>
3.	<u>-8</u>
4.	<u>-8</u>
5.	<u>-8</u>
6.	<u>-8</u>
7.	<u>-8</u>
8.	<u>-8</u>
9.	<u>-8</u>
10.	<u>-6</u>
11.	<u>-5</u>
12.	<u>-6</u>
13.	<u>-6</u>
14.	<u>-6</u>
15.	<u>-6</u>
16.	<u>-6</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>-6</u>
18.	<u>-5</u>
19.	<u>-6</u>
20.	<u>-6</u>
21.	<u>-6</u>
22.	<u>-6</u>
23.	<u>-6</u>
24.	<u>-7</u>
25.	<u>-8</u>
26.	<u>-8</u>
27.	<u>-8</u>
28.	<u>-8</u>
29.	<u>-8</u>
30.	<u>-8</u>
31.	<u></u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME One

COMPLETED BY Kristal Sirles

DATE May 6, 1997

REPORT MONTH April, 1997

TELEPHONE 309-654-2241

[illegible]

APPENDIX D

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Two

COMPLETED BY Kristal Sirles

DATE May 6, 1997

REPORT MONTH April 1997

TELEPHONE 309-654-2241

[illegible]

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram timing for the reporting period. All scram timing as performed with reactor pressure greater than 800 PSIG.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING
INFORMATION REQUEST

1. Unit: Q1 Reload: 14 Cycle: 15
2. Scheduled date for next refueling shutdown: 3/28/98
3. Scheduled date for restart following refueling: 5/29/98
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

No
5. Scheduled date(s) for submitting proposed licensing action and supporting information:

5/9/98
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

Approx. 216 SPC 9X9IX Fuel Bundles Q1R15 will be loaded.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 1933
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3657
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2002

QUAD CITIES REFUELING
INFORMATION REQUEST

1. Unit: 02 Reload: 14 Cycle: 15
2. Scheduled date for next refueling shutdown: 3/1/97
3. Scheduled date for restart following refueling: 5/27/97
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

Yes
5. Scheduled date(s) for submitting proposed licensing action and supporting information:

Submittals began August 1996; ongoing process.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

216 Siemens 9X9IX Power Corporation Fuel Bundles will be loaded during Q2R14.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 0
 - b. Number of assemblies in spent fuel pool: 3667
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3897
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2002

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLRT	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRs
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCIMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RPS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRS
SDV	- Scram Discharge Volume
SRM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center