

10 CFR 50.73

**Virginia Electric and Power Company**  
North Anna Power Station  
P. O. Box 402  
Mineral, Virginia 23117

June 17, 1997

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

NAPS.GSS  
Docket Nos. 50-338  
License Nos. NPF-4

Dear Sirs:

Pursuant to North Anna Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Unit 1.

Report No. 50-338/97-004-00

This Report has been reviewed by the Station Nuclear Safety Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Very truly yours,

*Dated for*

W. R. Matthews  
Station Manager

Enclosure:

Commitments contained in this report: None

cc: US Nuclear Regulatory Commission  
Region II  
Atlanta Federal Center  
61 Forsyth Street, SW, Suite 23T85  
Atlanta, Georgia 30303

Mr. M. J. Morgan  
NRC Senior Resident Inspector  
North Anna Power Station



9706250045 970617  
PDR ADOCK 05000338  
S PDR

## LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (1-6-F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

North Anna Power Station Unit 1

DOCKET NUMBER (2)

05000338

PAGE (3)

1 OF 3

TITLE (4)

Hi-Hi Alarm Setpoint for 1-RM-RMS-160 Found Out of Tolerance Due to Personnel Error

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCUMENT NUMBER
05	24	97	97	004	00	06	17	97	FACILITY NAME	05000
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
6			20.2201(b)		20.2203(a)(2)(v)		X		50.73(a)(2)(i)	50.73(a)(2)(viii)
POWER LEVEL (10)			20.2203(a)(1)		20.2203(a)(3)(i)				50.73(a)(2)(ii)	50.73(a)(2)(x)
0 %			20.2203(a)(2)(i)		20.2203(a)(3)(ii)				50.73(a)(2)(iii)	73.71
			20.2203(a)(2)(ii)		20.2203(a)(4)				50.73(a)(2)(iv)	OTHER
			20.2203(a)(2)(iii)		50.36(c)(1)				50.73(a)(2)(v)	Specify in Abstract below
			20.2203(a)(2)(iv)		50.36(c)(2)				50.73(a)(2)(vii)	or in NRC Form 366A

## LICENSEE CONTACT FOR THIS LER (12)

NAME

W. R. Matthews, Station Manager

TELEPHONE NUMBER (Include Area Code)

(540) 894-2101

## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

## SUPPLEMENTAL REPORT EXPECTED (14)

YES		NO		EXPECTED SUBMISSION DATE	MONTH	DAY	YEAR
(If yes, complete EXPECTED SUBMISSION DATE)							

## ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On May 24, 1997 with Unit 1 defueled, it was identified that the as-found Hi-Hi alarm setpoint for the containment gaseous radiation monitor (1-RM-RMS-160) had been incorrectly set above the allowable limits of Technical Specifications (TS) 3.3.3.1 during defueling operations. TS requires the containment gaseous radiation monitoring instrumentation channel Hi-Hi setpoint to be less than or equal to 3.6E+3 cpm for Mode 6. The as-found Hi-Hi setpoint was 2.5E+4 cpm. This event is reportable pursuant to 10CFR50.73 (a)(2)(i)(B) for a condition prohibited by TS.

The cause of this event is attributed to a personnel error whereby the 2.7E+3 cpm Hi-Hi setpoint stated on the Process Radiation Monitor Setpoint Record sheet was misread as 27E+3 cpm.

This event did not pose any safety implications. The containment Particulate Radiation Monitor and the Manipulator Radiation Monitor remained OPERABLE during this event. Abnormal Procedure, O-AP-30, "Fuel Failure During Handling" would also initiate a manual isolation of containment ventilation if a Hi radiation alarm is received from these containment radiation monitors. Therefore, the health and safety of the public were not affected at any time during this event.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

FACILITY NAME (1) North Anna Power Station Unit 1	DOCKET 05000338	LER NUMBER (6)			PAGE (3) 2 OF 3
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
		97	004	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

1.0 Description of the Event

During the performance of Periodic Test Procedure 1-PT-38.1.6, "Containment Radio Gas Monitor (RMS-160) Functional Test" on May 24, 1997 with Unit 1 defueled, it was identified that the as-found Hi-Hi alarm setpoint for containment gaseous radiation monitor 1-RM-RMS-160 (EIS System IL, Component MON) had been incorrectly set above the allowable limits of Technical Specifications (TS) 3.3.3.1 during defueling operations. TS requires the radiation monitoring instrumentation channel Hi-Hi setpoint for 1-RM-RMS-160 to be less than or equal to  $3.6\text{E}+3$  cpm for Mode 6. The recorded as-found setpoint of  $2.5\text{E}+4$  cpm exceeded this Technical Specifications allowable value. The desired setpoint value should have been  $2.7\text{E}+3$  cpm in accordance with the Process Radiation Monitor Setpoint Record obtained from the Radiation Monitoring Setpoint Book. The misreading of the decimal point during the performance of Instrument Calibration Procedure ICP-RMS-1-RM-160, "Containment Area Radio Gas Radiation Monitor Calibration" on May 12, 1997 caused the initial Hi-Hi alarm setpoint value to be set at 27000 cpm ( $27\text{E}+3$ ) in lieu of 2700 cpm ( $2.7\text{E}+3$ ).

2.0 Significant Safety Consequences and Implications

This event did not pose any safety implications. The containment Particulate Radiation Monitors (1-RM-RMS-159) and the Manipulator Radiation Monitor (1-RM-RMS-162) remained OPERABLE during this event. A Hi-Hi radiation signal from these containment radiation monitors would automatically trip the containment purge supply and exhaust fans and close the containment ventilation valves, thus isolating containment. Abnormal Procedure, 0-AP-30, "Fuel Failure During Handling" would also initiate a manual isolation of containment ventilation if a Hi radiation alarm is received from these containment radiation monitors. During the performance of Instrument Calibration Procedure ICP-RMS-1-RM-160, the Hi alarm setpoint was set correctly at  $1.5\text{E}+3$ . Therefore, the health and safety of the public were not affected at any time during this event.

This event is reportable pursuant to 10CFR50.73 (a)(2)(i)(B) for a condition prohibited by Technical Specifications.

3.0 Cause of the Event

The cause of this event is attributed to personnel error whereby the  $2.7\text{E}+3$  cpm Hi-Hi setpoint specified on the Process Radiation Monitor Setpoint Record sheet was misread as  $27\text{E}+3$  cpm. This caused the alarm setpoint value to be set at 27000 cpm in lieu of 2700 cpm.

# LICENSEE EVENT REPORT (LER)

## TEXT CONTINUATION

FACILITY NAME (1) North Anna Power Station Unit 1	DOCKET 05000338	<table border="1"> <tr> <th colspan="3" data-bbox="1031 217 1128 244">LER NUMBER (6)</th></tr> <tr> <th data-bbox="1031 244 1128 287">YEAR</th><th data-bbox="1128 244 1274 287">SEQUENTIAL NUMBER</th><th data-bbox="1274 244 1372 287">REVISION NUMBER</th></tr> <tr> <td data-bbox="1031 287 1128 325">97</td><td data-bbox="1128 287 1274 325">004</td><td data-bbox="1274 287 1372 325">00</td></tr> </table>	LER NUMBER (6)			YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	97	004	00	PAGE (3)  3 OF 3
LER NUMBER (6)												
YEAR	SEQUENTIAL NUMBER	REVISION NUMBER										
97	004	00										

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

### 4.0 Immediate Corrective Actions

Instrument Calibration Procedure ICP-RMS-1-RM-160 was reperformed and the Hi-Hi alarm setpoint was set correctly to 2.7 E+3. Periodic Test Procedure 1-PT-38.1.6 "Containment Radio Gas Monitor (RMS-160) Functional Test" was performed again with results satisfactory.

### 5.0 Additional Corrective Actions

Instrumentation and Control Supervisor coached department personnel on the importance of self-check and procedural compliance.

### 6.0 Actions to Prevent Recurrence

Procedure Action Requests were processed to revise Units 1 and 2 Instrument Calibration Procedures and Periodic Test Procedures for the Containment Gaseous and Particulate Radiation Monitors and the Manipulator Radiation Monitor to include independent verification. This independent verification requires a qualified individual to independently verify that the values transcribed from the Radiation Monitoring Setpoint Book have been correctly recorded into the Instrument Calibration procedure.

### 7.0 Similar Events

None

### 8.0 Additional Information

During this period Unit 2 was operating at 100 percent and was not affected by this event.