Commonwealth Edison Company
Dresden Generating Station
6500 North Dresden Road
Morris, IL 60450
Tel 815-942-2920

ComEd

May 5, 1997

JSPLTR 97-0093

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Subject: Monthly Operating Data Report for April 1997
Dresden Nuclear Power Station
Commonwealth Edison Company
Docket Nos. 50-010, 50-237, and 50-249

Enclosed is the Dresden Nuclear Power Station Monthly Operating Summary Report for April 1997.

This information is supplied to your office as required by Technical Specification 6.9.A.5, in accordance with the instructions set forth in Regulatory Guide 1.16.

Sincerely,

J. Stephen Perry for
Site Vice President
Dresden Station

1/1
IE24

Enclosure

cc: NRC Region III Office
Illinois Dept. of Nuclear Safety, State of Illinois
NRC Senior Resident Inspector
UDI, Inc. - Washington, DC
File/Numerical

MONTHLY NRC

SUMMARY OF OPERATING EXPERIENCE,

PER REGULATORY GUIDE 1.16

FOR

DRESDEN NUCLEAR POWER STATION

COMMONWEALTH EDISON COMPANY

FOR APRIL 1997

UNIT	DOCKET	LICENSE
1	050-010	DPR-2
2	050-237	DPR-19
3	050-249	DPR-25

TABLE OF CONTENTS

APRIL 1997 NRC REPORT

1.0 Introduction

Dresden Nuclear Power Station is a three reactor generating facility owned and operated by the ComEd Company of Chicago, Illinois. Dresden Station is located at the confluence of the Kankakee and Des Plaines Rivers, in Grundy County, near Morris, Illinois.

Dresden Unit 1 is a General Electric Boiling Water Reactor with a design net electrical output rating of 200 megawatts electrical (MWe). The unit is being decommissioned with all nuclear fuel removed from the reactor vessel. Therefore, no Unit 1 operating data is provided in this report.

Dresden Units 2 and 3 are General Electric Boiling Water Reactors, each licensed at 2527 megawatts thermal. The gross outputs of Units 2 and 3 are 832 and 834 megawatts electrical, respectively, with design net electrical output ratings of 794 MWe each. The commercial service date for Unit 2 is 11 August 1970, and 30 October 1971 for Unit 3.

Waste heat is rejected to a man-made cooling lake using the Kankakee River for make-up and the Illinois River for blowdown.

The Architect-Engineer for Dresden Units 2 and 3 was Sargent and Lundy of Chicago, Illinois.

This report for APRIL 1997, was compiled by Gary A. Abrell of Dresden Regulatory Assurance Staff, telephone number (815) 942-2920, extension 3749.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR APRIL 1997

2.1 UNIT 2 MONTHLY OPERATING EXPERIENCE SUMMARY

Unit 2 was on system at the beginning of the period near full power.

On April 10, 1997 the auxiliary contact assembly was found damaged for the bus 23 to 23-1 feed breaker during an inspection of electrical breakers. At 2112 all the Merlin Gerin electrical breakers were declared inoperable and preparations began for a unit shutdown. At 2236, commenced a load drop at 100 MWE per hour for a unit shutdown. At 0857 on April 11, 1997 a manual scram was initiated at 150 MWE to complete the unit shutdown.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR APRIL 1997

2.2 UNIT 3 MONTHLY OPERATING EXPERIENCE SUMMARY

Unit 3 was shutdown during this period for refueling outage D3R14.

April 1, 1997, the Mode Switch was placed in REFUEL at 1245.

April 3, 1997, the first fuel bundle was removed from the core for sipping at 0210.

April 10, 1997, the core off-load was completed at 2104.

3.0 OPERATING DATA STATISTICS

3.1 OPERATING DATA REPORT - DRESDEN UNIT TWO

DOCKET No. 050-237
DATE MAY 1, 1997
COMPLETED BY G. A. ABRELL
TELEPHONE (815) 942-2920

OPERATING STATUS

1. REPORTING PERIOD: **APRIL 1997**
2. CURRENTLY AUTHORIZED POWER LEVEL (MWth): 2,527
   MAXIMUM DEPENDABLE CAPACITY (MWe NET): 772
   DESIGN ELECTRICAL RATING (MWe Net): 794
3. POWER LEVEL TO WHICH RESTRICTED (MWe Net): No restrictions
4. REASONS FOR RESTRICTIONS (IF ANY): See Section 2.1 of this report.

REPORTING PERIOD DATA			
PARAMETER	THIS MONTH	YEAR TO DATE	CUMULATIVE
HOURS IN PERIOD	719	2,879	234,215
TIME REACTOR CRITICAL	248	2,408	169,570
TIME REACTOR RESERVE SHUTDOWN (Hours)	0	0	0
TIME GENERATOR ON-LINE (Hours)	248	2,408	161,423
TIME GENERATOR RESERVE SHUTDOWN (Hours)	0	0	1
THERMAL ENERGY GENERATED (MWHt Gross)	615,786	5,593,068	337,296,969
ELECTRICAL ENERGY GENERATED (MWEHe Gross)	198,498	1,814,799	107,712,735
ELECTRICAL ENERGY GENERATED (MWEHe Net)	189,408	1,734,786	101,836,515
REACTOR SERVICE FACTOR (%)	34.5%	83.6%	72.4%
REACTOR AVAILABILITY FACTOR (%)	34.5%	83.6%	72.4%
GENERATOR SERVICE FACTOR (%)	34.5%	83.6%	68.9%
GENERATOR AVAILABLILITY FACTOR (%)	34.5%	83.6%	68.9%
CAPACITY FACTOR (Using MCD Net) (%)	34.1%	78.1%	56.3%
CAPACITY FACTOR (Using DER Net) (%)	33.2%	75.9%	54.8%
FORCED OUTAGE FACTOR (%)	66%	16.4%	12.9%

20. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS: NONE

21. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: May 2, 1997

3.0 OPERATING DATA STATISTICS

3.2 OPERATING DATA REPORT - DRESDEN UNIT THREE

DOCKET No. 050-249
DATE MAY 1, 1997
COMPLETED BY G. A. ABRELL
TELEPHONE (815) 942-2920

OPERATING STATUS

1. REPORTING PERIOD: **APRIL 1997**
2. CURRENTLY AUTHORIZED POWER LEVEL (MWth): 2,527
   MAXIMUM DEPENDABLE CAPACITY (MWe Net): 773
   DESIGN ELECTRICAL RATING (MWe Net): 794
3. POWER LEVEL TO WHICH RESTRICTED: No restriction
4. REASONS FOR RESTRICTIONS (IF ANY): See Section 2.2 of this report.

	REPORTING PERIOD DATA			
	PARAMETER	THIS MONTH	YEAR TO DATE	CUMULATIVE
5	HOURS IN PERIOD	719	2,879	223,535
6	TIME REACTOR CRITICAL	0	1395.8	157,253
7	TIME REACTOR RESERVE SHUTDOWN (Hours)	0.0	0	0
8	TIME GENERATOR ON-LINE (Hours)	0	1,356	149,880
9	TIME GENERATOR RESERVE SHUTDOWN (Hours)	0.0	0	0
10	THERMAL ENERGY GENERATED (MWHt Gross)	0	2,267,378	312,479,855
11	ELECTRICAL ENERGY GENERATED (MWEHe Gross)	0	729,667	100,002,797
12	ELECTRICAL ENERGY GENERATED (MWEHe Net)	-4,056	686,361	94,768,862
13	REACTOR SERVICE FACTOR (%)	0.0%	48.5%	70.3%
14	REACTOR AVAILABILITY FACTOR (%)	0.0%	48.5%	70.3%
15	GENERATOR SERVICE FACTOR (%)	0.0%	47.1%	67.0%
16	GENERATOR AVAILABLILITY FACTOR (%)	0.0%	47.1%	67.0%
17	CAPACITY FACTOR (Using MCD Net) (%)	-0.7%	30.9%	54.9%
18	CAPACITY FACTOR (Using DER Net) (%)	-0.7%	30.0%	53.4%
19	FORCED OUTAGE FACTOR (%)	0.0%	35.1%	14.3%

20. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS: In refuel outage D3R14.

21. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: June 2, 1997.

3.3 AVERAGE DAILY UNIT 2 POWER LEVEL

DOCKET No. 050-237
UNIT Dresden 2
DATE MAY 1, 1997
COMPLETED BY G. A. ABRELL
TELEPHONE (815) 942-2920

MONTH: April-97

DRESDEN 2

DAY	AVERAGE DAILY NET POWER LEVEL (MWe)	DAY	AVERAGE DAILY NET POWER LEVEL (MWe)
1	791	17	-4
2	790	18	-4
3	785	19	-4
4	784	20	-4
5	788	21	-4
6	756	22	-4
7	792	23	-5
8	792	24	-5
9	793	25	-5
10	787	26	-4
11	122	27	-4
12	-6	28	-4
13	-5	29	-5
14	-5	30	-6
15	-5		
16	-4		

(Note: negative values represent station loads)

3.4 AVERAGE DAILY UNIT 3 POWER LEVEL

DOCKET No. 050-249
UNIT Dresden 3
DATE MAY 1, 1997
COMPLETED BY G. A. ABRELL
TELEPHONE (815) 942-2920
MONTH: April-97

DRESDEN 3

DAY	AVERAGE DAILY NET POWER LEVEL (MWe)	DAY	AVERAGE DAILY NET POWER LEVEL (MWe)
1	-8	17	-4
2	-8	18	-3
3	-8	19	-3
4	-8	20	-3
5	-8	21	-4
6	-8	22	-4
7	-8	23	-5
8	-8	24	-5
9	-8	25	-5
10	-8	26	-4
11	-7	27	-4
12	-6	28	-4
13	-5	29	-4
14	-5	30	-5
15	-5		
16	-4		

(Note: Negative values represent station loads)

3.5 UNIT 2 SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OF APRIL 1997

NO	DATE	TYPE (1)	DURATION (HOURS)*	REASON (2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE(4)	COMPONENT CODE (5)	CORRECTIVE ACTIONS/ COMMENTS
3	970411	F	471	A	1	9708	EA	BKR	2.1

Year-to-date forced outage hours = 471
Cumulative forced outage hours = 23,406

TABLE KEY:

(1)
F: Forced
S: Scheduled

(2)
Reason:
A Equipment Failure (Explain)
B Maintenance or Test
C Refueling
D Regulatory Restriction
E Operator Training & Licensing Exam
F Administrative
G Operational Error
H Other (Explain)

(3)
Method:
1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)
5. Load Reduction

(4)
Exhibit G Instruction for Preparation of Data Entry Sheets for Licensee Event Reports (LER) File (NUREG-0161)

(5)
Exhibit I Same Source as Above.

3.6 UNIT 3 SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH APRIL 1997

NO.	DATE	TYPE(1)	DURATION (HOURS)*	REASON(2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE (4)	COMPONENT CODE (5)	CORRECTIVE ACTIONS/ COMMENTS
1	970328	S	719	C	1				2.2

Year-to-date forced outage hours = 734
Cumulative forced outage hours = 25,810

TABLE KEY:

(1)
F: Forced
S: Scheduled

(2)
Reason:
A. Equipment Failure (Explain)
B Maintenance or Test
C Refueling
D Regulatory Restriction
E Operator Training & Licensing Exam
F Administrative
G Operational Error
H Other (Explain)

(3)
Method:
1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)
5. Load Reduction

(4)
Exhibit G Instruction for Preparation of Data Entry Sheets for Licensee Event Reports (LER) File (NUREG-0161)

(5)
Exhibit I Same Source as Above.

4.0 UNIQUE REPORTING REQUIREMENTS

4.1 MAIN STEAM RELIEF AND/OR SAFETY VALVE OPERATIONS - UNIT 2 AND UNIT 3

None

4.2 OFF-SITE DOSE CALCULATION MANUAL (ODCM) CHANGES

None

4.3 MAJOR CHANGES TO THE RADIOACTIVE WASTE TREATMENT SYSTEMS

None

4.4 FAILED FUEL ELEMENT INDICATIONS

4.4.1 Unit 2

Unit 2 has no indications of fuel failures.

4.4.2 Unit 3

Unit 3 is defueled for D3R14. The leaking fuel element was identified by sipping and will not be reloaded.

5.0 TECHNICAL SPECIFICATION AMENDMENTS

5.1 Amendments to Facility License or Technical Specifications implemented during April 1997.

April 10, Received amendment 156 and 151 which permitted the removal of the Unit 3 24/48 VDC battery.

April 25, received amendment 158 and 153 which raised the minimum iodine removal efficiency of the Standby Gas Treatment Charcoal Filters from 90% to 97.5%. This was done because a recalculation of the secondary containment volume resulted in a smaller volume. The smaller volume, when used as a input to the calculation of post accident exposure to the control room operators, raised the predicted exposure above acceptable limits. This was offset by the improved iodine removal efficiency of the Standby Gas Treatment Charcoal Filters.

April 30, received amendment 157 and 152 which evaluated the Unreviewed Safety Question (USQ) associated with the use of containment overpressure to compensate for a deficiency in the net positive suction head for the emergency core cooling pumps following a design basis accident. The amendment raised the

maximum suppression chamber water temperature and ultimate heat sink temperature from 75° F to 95° F.