



**Northeast
Nuclear Energy**

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Millstone Nuclear Power Station
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The Northeast Utilities System

JUN 12 1997

Docket No. 50-423
B16518

US Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D C. 20555

**Millstone Nuclear Power Station Unit 3
Facility Operating License Number NPF-49
Monthly Operating Report for May 1997**

In accordance with the reporting requirements of Technical Specification 6.9.1.5 for Millstone Unit No. 3, enclosed in Attachment 2 is the Monthly Operating Report for the month of May, 1997.

Should you have any questions regarding this submittal, please contact Mr. David Smith at (860) 437-5840.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

G.D. Hicks
Unit Director - Millstone Unit 3

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Attachments (2)

cc: H. J. Miller, Region 1 Administrator
W. D. Travers, Dr., Director Special Projects
A.C. Cerne, Senior Resident Inspector, Millstone Unit No. 3
J. W. Anderser, NRC Project Manager, Millstone Unit No. 3

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Docket No. 50-423
B16518

Attachment 1

Millstone Nuclear Power Station, Unit No. 3

NNECO's Commitments

June 1997

Enclosure
List of Regulatory Commitments

The following table identifies those actions committed to by NNECO in this document. Please notify the Manager - Nuclear Licensing at the Millstone Nuclear Power Station Unit No. 3 of any questions regarding this document or any associated regulatory commitments.

Commitment	Committed Date or Outage
NONE	N/A

Docket No. 50-423
B16518

Attachment 2

Millstone Nuclear Power Station, Unit No. 3

Facility Operating License No. NPF-49

May 1997 Monthly Operating Report

June 1997

REFUELING INFORMATION REQUEST
May 1997

1. Name of the facility: Millstone Unit 3
2. Scheduled date for next refueling outage: To be determined
3. Scheduled date for restart following refueling: To be determined
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
N/A.
5. Scheduled date(s) for submitting licensing action and supporting information:
None.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
None.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:
In Core: (a) 193 In Spent Fuel Pool: (b) 416
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:
Present storage capacity: 756.
No increase requested.
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming present license capacity:
End of Cycle 7.

DOCKET NO. 50-423
UNIT Millstone Unit 3
DATE 06/03/97
COMPLETED BY K. W. Emmons
TELEPHONE (860)447-1791
Ext. 6572

MONTH: May-97

DAY	AVG. DAILY POWER LEVEL (MWe-Net)	DAY	AVG. DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT NAME Millstone Unit 3
DATE 06/03/97
COMPLETED BY K. W. Emmons
TELEPHONE (860) 447-1791
Ext 6572

OPERATING STATUS

1. Docket Number 50-423
2. Reporting Period May-97
3. Utility Contact K. W. Emmons
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1253
6. Design Electrical Rating (Net MWe): 1153.6
7. Maximum Dependable Capacity (Gross MWe): 1184.20
8. Maximum Dependable Capacity (Net MWe): 1137.00
9. If Changes Occur in Capacity Ratings (Items Number 4 Through 8) Since Last Report, Give Reasons:
N/A

Notes:

10. Power Level To Which Restricted, If any (Net MWe): 0
11. Reasons For Restrictions, If Any: NRC Category III Facility; NRC Confirmatory Order requiring implementation of an independent corrective action verification program; NRC Order requiring a third-party review of the employee concerns program at Millstone Station; design basis verification response pursuant to 10CFR50.54(f).

	This Month	Yr.-To-Date	Cumulative
12. Hours In Reporting Period	744.0	3623.0	97367.0
13. Number Of Hours Reactor Was Critical	0.0	0.0	67080.1
14. Reactor Reserve Shutdown Hours	0.0	0.0	6525.8
15. Hours Generator On-Line	0.0	0.0	65912.4
16. Unit Reserve Shutdown Hours	0.0	0.0	0.0
17. Gross Thermal Energy Generated (MWH)	0.0	0.0	216937728.1
18. Gross Electrical Energy Generated (MWH)	0.0	0.0	74905103.1
19. Net Electrical Energy Generated (MWH)	-4864.1	-23899.6	71272819.9
20. Unit Service Factor	0.0	0.0	67.7
21. Unit Availability Factor	0.0	0.0	67.7
22. Unit Capacity Factor (Using MDC Net)	0.0	0.0	64.3
23. Unit Capacity Factor (Using DER Net)	0.0	0.0	63.5
24. Unit Forced Outage Rate	100.0	100.0	23.8
25. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Shutdown at time of this report.			

26. If Unit Shutdown At End Of Report Period, Estimated Date of Startup: To be determined.
27. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-423
UNIT NAME: Millstone Unit 3
DATE 06-03-97
COMPLETED BY K. W. Emmons
TELEPHONE (860) 447-1791 X 6572

REPORT MONTH: May 1997

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	License Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
96-01	03-30-96	F	744	B/D	4	96-006-00	BA	ISV	Valves inoperable due to original design deficiencies, in that an improper valve design did not meet GDC 57. Corrective action is to install vendor kit to modify valve disk to meet GDC 57. Continued shutdown: NRC Category III facility; NRC Confirmatory Order requiring independent corrective action verification; NRC Order requiring third-party review of Millstone Station employee concerns program; design basis verification for response to NRC pursuant to 10CFR50.54(f).

¹F: Forced
S: Scheduled

²Reason
A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & License Examination
F - Administrative
G - Operational Error (Explain)
H - Other (Explain)

³Method
1 - Manual
2 - Manual Scram
3 - Automatic Scram
4 - Continued from Previous Month
5 - Power Reduction (Duration = 0)
6 - Other (Explain)

⁴IEEE Standard 805-1984,
"Recommended Practices
for System Identification in
Nuclear Power Plants and
Related Facilities"

⁵IEEE Standard 803A-1983,
"Recommended Practices
for Unique identification in
Power Plants and Related
Facilities - Component
Function Identifiers"