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June 16, 1997  
6730-97-2183

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Dear Sir:

SUBJECT: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Monthly Operating Report - May, 1997

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1, enclosed are two (2) copies of the Monthly Operating Data (Gray Book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Ms. Brenda DeMerchant, Oyster Creek Regulatory Affairs Engineer, at 609-971-4642.

Very truly yours,

for

Michael B. Roche  
Vice President & Director  
Oyster Creek

MBR/BDeM/gl

Enclosures

cc: Administrator, Region I (2 copies)  
NRC Project Manager  
NRC Resident Inspector



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PDR ADOCK 05000219  
R PDR

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## **MONTHLY OPERATING REPORT**

### **Licensee Event Reports -- May, 1997**

LER 97-005 was filed on May 8, 1997.

On April 8, 1997 it was identified that the most recent Standby Liquid Control System (SLCS) sodium pentaborate solution Boron-10 (B-10) enrichment surveillance determining the B-10 enrichment solution results were not received within 30 days of sampling. Technical Specification Section 4.2.E.5 states, "If not received within 30 days, notify NRC (within 7 days) of plans to obtain test results." This notification was not made.

The cause of this event was determined to be an inadequate procedure in that Technical Specification requirements were not included in permanent plant documents.

The safety significance of this event is considered minimal. When the results were received, the B-10 enrichment atom percent was within specification. Therefore, the SLCS was capable of performing its intended function at all times.

Corrective actions include revising appropriate plant documents to reflect the Technical Specification requirement.

LER 97-006 was filed on May 19, 1997.

On April 19, 1997, control rod scram time testing was performed on a reference sample of seven control rods for Scram Solenoid Pilot Valve (SSPV) performance monitoring based on BWR Owners' Group recommendations. The tests revealed that the 5% scram insertion times had increased over previous tests, and the average 5% insertion time for these seven rods exceeded complete core average Technical Specification time limit. The reactor was shutdown to address the degraded performance.

The apparent root cause of this event was the sticking of the Viton A diaphragms in the SSPVs. The safety significance of this event was determined to be minimal, because the small delay in 5% insertion time has a negligible effect on design basis accident and transient analyses.

While the reactor was shutdown, the diaphragms for all SSPVs were replaced with a new Viton 515 A-B material designed to prevent the sticking experienced with Viton A. Subsequent scram time tests have shown a marked improvement in 5% insertion time. An evaluation will be performed to determine the need for a new test program on the SSPVs.

AVERAGE DAILY POWER LEVEL  
NET MWe

Docket Number : 50-219  
Unit: Oyster Creek #1  
Report Date: June 5, 1997  
Compiled By: John D. Dougher  
Telephone Number: (609) 971- 2130

Month: May, 1997

DAY	MW	DAY	MW
1	0	16	625
2	0	17	631
3	0	18	633
4	0	19	629
5	53	20	631
6	143	21	634
7	307	22	629
8	624	23	637
9	632	24	632
10	635	25	631
11	635	26	630
12	632	27	633
13	631	28	634
14	632	29	632
15	631	30	630
		31	629

## OPERATING DATA REPORT

### OPERATING STATUS

1. Docket:	50-219		
2. Reporting Period:	May, 1997		
3. Utility Contact:	John D. Dougher	(609) 971- 2130	
4. Licensed Thermal Power (MWt):	1930		
5. Nameplate Rating (Gross MWe):	687.5 x 0.8 = 550		
6. Design Electrical Rating (Net MWe):	650		
7. Maximum Dependable Capacity (Gross MWe):	641		
8. Maximum Dependable Capacity (Net MWe):	619		
9. If changes occur above since last report, give reasons:	None		
10. Power Level to Which Restricted, If Any (Net MWe):	None		
11. Reason for Restriction, If Any:	None		
	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. Report Period Hours	744.0	3623.0	240527.0
13. Hours Rx Critical	675.5	3371.4	164960.2
14. Rx Reserve Shutdown Hours	0.0	0.0	918.2
15. Hours Generator On-Line	633.2	3326.5	161244.3
16. Ut Reserve Shutdown Hours	0.0	0.0	0.0
17. Gross Thermal Energy (MWh)	1158485	6273826	280187907
18. Gross Electric Eenergy (MWh)	390278	2148592	94077859
19. Net Electric Energy (MWh)	374941	2070272	90278838
20. Ut Service Factor	85.1	91.8	67.0
21. Ut Available Factor	85.1	91.8	67.0
22. Ut Capacity Factor (MDC Net)	81.4	92.3	61.2
23. Ut Capacity Factor (DER Net)	77.5	87.9	57.7
24. Ut Forced Outage Rate	14.9	8.2	9.8
25. Forced Outage Hours	110.8	296.6	17433.9
26. Shutdowns Scheduled Over Next 6 Months (Type,Date,Duration):			None
27. If Currently Shutdown, Estimated Startup Date:			N/A

Oyster Creek Station #1

Docket No. 50-219

Refueling Information - May 1997

Name of Facility: Oyster Creek Station #1

Scheduled date for refueling shutdown: September 1998

Scheduled date for restart following refueling: October 1998

Will refueling or resumption of operation thereafter require a  
Technical Specification change or other license amendment? No

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - Fuel design and performance analysis methods have been approved by the NRC.

The number of fuel assemblies	(a) in the core	=	560
	(b) in the spent fuel storage pool	=	2236
	(c) in new fuel storage vault	=	8

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present Licensed Capacity: 2645

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Full core discharge capacity to the spent fuel pool was lost after the 1996 refueling outage.

# UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-219  
 UNIT NAME: Oyster Creek  
 DATE: May 13, 1997  
 COMPLETED BY: David M. Egan  
 TELEPHONE: 609/971-4818

REPORT MONTH: June 9, 1997

No.	DATE	TYPE F: Forced S: Scheduled	DURATION (hours)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
3	970423	F	110.8	a	1	Plant manually shutdown per Tech Specs due to failure of CRD Scram Time Test. Plant returned to service on May 5, 1997 at 14:46.

## SUMMARY:

### (1) REASON

- |                                |                                 |
|--------------------------------|---------------------------------|
| a. Equipment Failure (Explain) | c. Operator Training & Lic Exam |
| b. Maintenance or Test         | f. Administrative               |
| c. Refueling                   | g. Operational Error (Explain)  |
| d. Regulatory Restriction      | h. Other (Explain)              |

### (2) METHOD

1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)



## Summary

### May, 1997

At the beginning of May, Oyster Creek was completing work from its second unscheduled outage in Cycle 16 (16U2). The plant shutdown on April 23, 1997, when it failed to meet the technical specification requirements for control rod scram time testing. 16U2 ended on May 5, 1997, at 1446 following repairs and preliminary testing of the CRDs. The plant returned to full power on May 3, 1997, following successful scram time testing of the control rods. The plant generated 390,278 MWh net electric, which was 81.4% of its MDC rated capacity.