



Northern States Power Company

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May 7, 1997

10 CFR Part 50
Section 50.90

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-308 DPR-60

License Amendment Request Dated May 7, 1997
Amendment of Spent Fuel Pool Special Ventilation System Specifications

Attached is a request for changes to the Technical Specifications, Appendix A of the Operating Licenses, for the Prairie Island Nuclear Generating Plant. This request is submitted in accordance with the provisions of 10 CFR Part 50, Section 50.90. These requested amendments will maintain limitations on spent fuel handling relating to spent fuel pool special ventilation system operability. Limitations on crane operations in the spent fuel pool enclosure relating to spent fuel pool special ventilation system operability will be removed from the Technical Specifications. These changes are necessary to allow movement of loads over spent fuel stored in the spent fuel pool enclosure with the spent fuel pool special ventilation system inoperable.

Exhibit A contains a description of the proposed changes, the reasons for requesting the changes, and the supporting safety evaluation and significant hazards determination. Exhibit B contains current Prairie Island Technical Specification pages marked up to show the proposed changes. Exhibit C contains the revised Prairie Island Technical Specification pages incorporating the proposed changes.

Recently the Prairie Island plant staff reviewed the technical specification interpretations to assure that plant operations were in compliance with the technical specifications. Difficulties were identified with verbatim compliance with TS.3.8,

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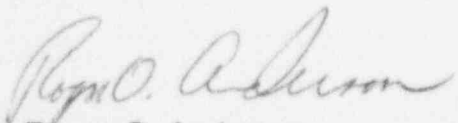


Applicability and Objectives and TS.3.8.D. It is our understanding that the underlying purpose of these specifications is to assure that the spent fuel pool special ventilation system is operable when heavy loads present a potential for damaging irradiated fuel stored in the spent fuel pools. License Amendments 91/84 incorporated these limitations into the Prairie Island Technical Specifications on October 27, 1989. Since License Amendments 91/84 were issued, NSP has installed single-failure-proof cranes and implemented procedures which implement the requirements of NUREG-0612, "Control of Heavy Loads at Nuclear Plants," for handling heavy loads over the spent fuel pools. *Prairie Island will assure that heavy loads do not present a potential for damaging irradiated fuel through use of: 1) a single-failure-crane with rigging and procedures which implement Prairie Island commitments to NUREG-0612; or, 2) spent fuel pool covers and their implementing plant procedures.* Compliance with the requirements of NUREG-0612, through use of a single-failure-proof crane, associated rigging and procedures, or through use of spent fuel pool covers, eliminates the necessity of crane operating limitations in TS.3.8.

In general, crane operating limitations are the subject of plant operating procedures and commitments, and are not contained in the Technical Specifications. Removal of crane operating restrictions from this Technical Specification will make this specification consistent with other Prairie Island Technical Specifications and consistent with NUREG-1431, Standard Technical Specifications, Westinghouse Plants.

Movement of heavy loads through the spent fuel pool enclosure is required to prepare for and implement the Unit 1 refueling outage scheduled to commence on October 18, 1997. Therefore, NSP requests that the proposed license amendment request be issued by October 11, 1997 in order to allow for movement of heavy loads over spent fuel stored in the spent fuel pool enclosure in preparation for the Unit 1 refueling outage.

In this submittal, NSP has made new Nuclear Regulatory Commission commitments which are identified in italics. If you have any questions related to this information in support of the subject license amendment request, please Dale Vincent at 612-388-1121.



Roger O. Anderson
Director,
Licensing and Management Issues

(attachments and copies listed on page 3)

Attachments:

Affidavit

Exhibit A, Description of the Proposed Changes, The Reasons for Requesting the Changes, and the Supporting Safety Evaluation/Significant Hazards Determination.

Exhibit B, Marked Up Technical Specification Pages

Exhibit C, Revised Technical Specification Pages

c: Regional Administrator - III, NRC
NRR Project Manager, NRC
Senior Resident Inspector, NRC
State of Minnesota
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