

GENERAL ELECTRIC COMPANY
Sunnyvale, California 94806

Docket No. 50-231
RE: License DR-15

APR 28 1971



Dr. Peter A. Morris, Director
Division of Reactor Licensing
U. S. Atomic Energy Commission
Washington, D. C. 20545

Regulatory

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Dear Sir:

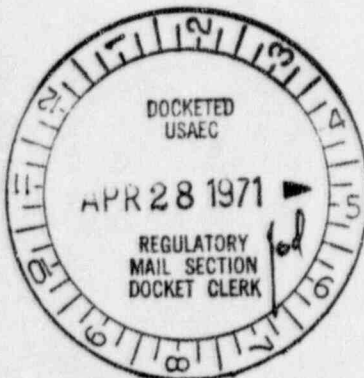
The attached Proposed Change No. 5 to the Technical Specifications is submitted in partial response to the request for additional information contained in your letter of April 19, 1971. Information on the remaining items outlined in your letter was discussed in a meeting with members of your staff on April 27, 1971, and will be documented in the near future.

The attached proposed specifications include limits on reactor primary sodium temperature heat-up and cool-down rates to provide assurance that the reactor vessel head bolt stresses remain below the ASME code allowable values. In addition, it defines an in-service inspection program to verify the adequacy of the proposed limits.

Very truly yours,

K. Cohen
for Karl Cohen

Karl Cohen, General Manager
Nuclear Energy Division
Sunnyvale, California 94806



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I. Introduction

In accordance with the provisions of 10 CFR 50, General Electric requests that the SEFOR Technical Specifications be amended by adding paragraphs 3.4.L, 3.4.M, 3.4.N, 3.4.O, 4.4.R, and 4.4.S, and the bases for these paragraphs, as given in Appendix I of this document.

II. Discussion

Analyses of the stress levels in the reactor vessel head bolts have indicated that the bolt stress levels will remain below the ASME code values when the temperature transient limits and pre-load limits proposed in the attached changes to the Technical Specifications are observed. The analyses were discussed with members of the DRL staff at a meeting in Bethesda on April 27, 1971. Documentation of this information is being prepared and will be submitted to DRL in the near future.

Surveillance requirements are included in the proposed specifications to provide assurance that operation of the reactor within the proposed limits does not result in damage to the outer head bolts. The first of the periodic examinations required by the proposed surveillance program will be performed on or before August 31, 1972. In addition, one of the three reactor vessel outer head bolts removed for inspection during March 1971 will be removed and examined for evidence of permanent elongation following the Core I sub-prompt testing and prior to Core I super-prompt testing.